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**William J. Cahill, Jr.**  
Chief Nuclear Officer

March 3, 1995  
IPN-95-027

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555

Subject: **Indian Point 3 Nuclear Power Plant**  
**Docket No. 50-286**  
**1994 ECCS Evaluation Changes**

- References:
1. Westinghouse letter, J. R. Gasperini to L. M. Hill, dated February 3, 1995, "10 CFR 50.46 Notification and Reporting Information."
  2. NYPA letter, W. J. Cahill, Jr. to NRC, dated December 5, 1994 (IPN-94-152), "Emergency Core Cooling System Evaluation Changes."

Dear Sir:

This letter describes changes to the emergency core cooling system (ECCS) evaluation model, and how these changes affect the peak cladding temperature (PCT). This letter satisfies the annual reporting requirements of 10 CFR 50.46(a)(3)(ii).

The Authority has reviewed the small and large break loss of coolant accident (LOCA) evaluation model changes for 1994 (Reference 1). There are no permanent changes in the large break LOCA analysis. The following permanent changes in the small break LOCA analysis are not significant, as defined by 10 CFR 50.46(a)(3)(i), and therefore were not previously reported in Reference 2.

<u>1994 10 CFR 50.46 Model Assessments</u>	<u>Change in PCT</u>
Containment Spray During Small Break LOCA	+20°F
Boiling Heat Transfer Correlation Error	-6°F
Steam Line Isolation Logic Error	+18°F

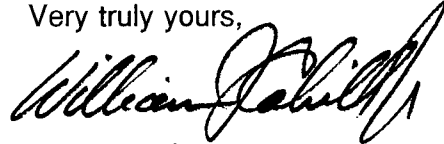
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The Authority has reviewed the previously described changes and has determined that the Indian Point 3 PCT is well below the maximum fuel cladding temperature of 2200°F, and that Indian Point 3 continues to comply with 10 CFR 50.46.

No commitments are being made by the Authority in this submittal. If you have any questions, please contact Ms. C. D. Faison.

Very truly yours,



William J. Cahill, Jr.  
Chief Nuclear Officer

cc: Regional Administrator  
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