



William J. Cahill, Jr.
Executive Vice President
and Chief Nuclear Officer

October 13, 1994
IPN-94-127

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Subject: **Indian Point 3 Nuclear Power Plant**
Docket No. 50-286
Station Blackout

- References:
1. NYPA letter, J.C. Brons to the NRC, "Response to 10 CFR Part 50.63, Loss of All Alternating Current Power - Station Blackout," dated April 17, 1989 (IPN-89-025).
 2. NYPA letter, J.C. Brons to the NRC, "Station Blackout Supplementary Information," dated March 29, 1990 (IPN-90-018).
 3. NYPA letter, R.E. Beedle to the NRC, "Response to Additional Information Concerning Implementation of the Station Blackout Rule," dated August 19, 1991 (IPN-91-031).
 4. NYPA letter, R.E. Beedle to the NRC, "Response to Safety Evaluation Recommendations," dated January 29, 1992 (IPN-92-006).
 5. NRC letter, N.F. Conicella to R.E. Beedle, "Safety Evaluation of the Indian Point 3 Response to the Station Blackout Rule," dated December 23, 1991.
 6. NRC letter, N.F. Conicella to R.E. Beedle, "Supplemental Safety Evaluation of Response to the Station Blackout Rule," dated June 9, 1992.

Dear Sir:

This letter documents that all NRC Safety Evaluation (SE) and Supplemental Safety Evaluation (SSE) recommendations concerning Station Blackout have been addressed and completed.

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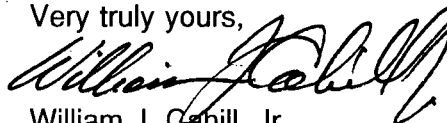
On July 21, 1988 the Nuclear Regulatory Commission (NRC) amended its regulations in 10 CFR Part 50 to include a new section, 10 CFR 50.63, "Loss of All Alternating Current Power." 10 CFR 50.63 requires that each light water cooled nuclear plant be able to withstand and recover from a Station Blackout (SBO) of a specified duration. Regulatory Guide (RG) 1.155 provides guidance acceptable to the NRC for satisfying the requirements of 10 CFR 50.63. RG 1.155 states that the NRC has determined that NUMARC 87-00 is also acceptable to the NRC for meeting these requirements.

The Authority has evaluated the Indian Point 3 Nuclear Power Plant against the requirements of the SBO rule using the guidance from NUMARC 87-00. Detailed results of this evaluation are discussed in References 1 through 4. The NRC SE dated December 23, 1991 (Reference 5) and the NRC SSE dated June 9, 1992 (Reference 6) responded to the referenced Authority letters and closed a number of issues. The SSE dated June 9, 1992 also contained NRC final recommendations, including a recommendation to revise the coping duration categorization from four to eight hours. The Authority revised its analysis using an eight hour coping duration category and addressed all remaining recommendations accordingly.

As a result of this analysis, the Authority has determined that Indian Point 3 is capable of attaining a SBO coping duration category of eight hours with no modifications to the plant. In conclusion, the Authority's implementation of the recommendations of the NRC SSE (Reference 6) completes all of the actions required by 10 CFR 50.63.

This letter contains no new commitments. If you have any questions please contact Mr. P. Kokolakis.

Very truly yours,



William J. Cahill, Jr.
Executive Vice President and
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Nuclear Generation

cc: see next page

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