



June 27, 1994
IPN-94-076

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
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Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
**Generic Letter 92-01, Revision 1: Reactor
Vessel Structural Integrity (TAC No. M83473)**

- References:
- 1) NRC letter, N. F. Conicella to W. J. Cahill, Jr., dated April 20, 1994, "Generic Letter (GL) 92-01, Revision 1, 'Reactor Vessel Structural Integrity,' - Indian Point Nuclear Generating Unit No. 3."
 - 2) NYPA letter, W. A. Josiger to NRC, dated April 12, 1994 (IPN-94-044), "Generic Letter 92-01, Revision 1, Additional Information Concerning NRC Evaluation of Pressurized Thermal Shock."
 - 3) NYPA letter, R. E. Beedle to NRC, dated November 24, 1993 (IPN-93-149), "Generic Letter 92-01, Revision 1, Response to Request for Additional Information."
 - 4) WCAP-8047, "American Electric Power Service Corp. Donald C. Cook Unit No. 1 Reactor Vessel Radiation Surveillance Program," S. E. Yanichko and D. J. Lege, March 1973.

Dear Sir:

The Nuclear Regulatory Commission (NRC) recently requested that the Authority verify the accuracy of reactor vessel integrity data for Indian Point 3. This data was provided by the NRC in Reference 1, Enclosures 1 and 2. It was also requested in Reference 1 that the Authority submit a schedule for providing additional data regarding the use of a generic unirradiated upper-shelf energy (USE) value. This letter provides the Authority's response to these requests.

The Authority identified two discrepancies in the information contained in Reference 1, Enclosures 1 and 2. The first discrepancy involves the heat number for Plate B2803-2, which is C-1397-3, rather than C-1397-2 (listed in Reference 1, Enclosures 1 and 2). The second concerns the chemistry factor (CF) for Plate B2803-3 (listed in Reference 1, Enclosure 1). In Reference 2, the Authority provided information in support of a CF value of 158.7°F.

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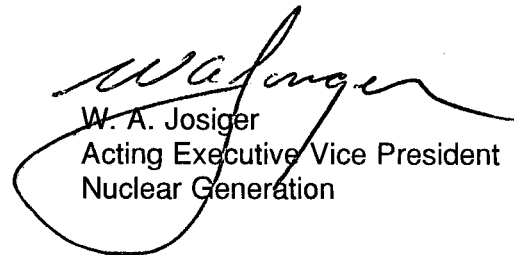
The additional data requested in Reference 1 pertains to the unirradiated USE values of the Intermediate Shell Axial Welds (2-042A,B,C), the Intermediate to Lower Shell Girth Weld (9-042), and the Lower Shell Axial Welds (3-042A,B,C). The value provided by the Authority for all welds (120 ft-lbs) was derived from surveillance weld material, which was fabricated from a different heat of weld wire than that which was used for the beltline welds. As a result, the NRC has requested that the Authority either use unirradiated USE values from welds with heat numbers that are identical to those of the beltline welds, or perform a statistical analysis utilizing industry data from similar welds to obtain acceptable values.

As stated in Reference 3, the same weld wire that was used to fabricate the Lower Shell Girth Weld (9-042) was used for the D. C. Cook Unit 1 surveillance weld. Based upon preirradiation data for the D. C. Cook Unit 1 weld (Reference 4), the unirradiated USE for Weld 9-042 is 111 ft-lbs, and the resulting end-of-life USE is 70 ft-lbs. These values should replace those that are currently listed in Enclosure 2 of Reference 1.

The Authority currently has no data for welds with the same heat number as the Intermediate Shell Axial Welds (2-042A,B,C) and the Lower Shell Axial Welds (3-042A,B,C). The Authority anticipates that this data, or the industry data necessary to perform a statistical analysis, will be accessible when the Reactor Vessel Integrity Database (RVID) becomes publicly available. As the RVID is currently scheduled for availability at the end of 1994, the Authority expects to complete its analysis for these welds by June 30, 1995.

Attachment I lists the Authority's commitment contained in this submittal. If you have any questions, please contact Mr. P. Kokolakis.

Very truly yours,



W. A. Josiger
Acting Executive Vice President
Nuclear Generation

att: as stated
cc: see next page

cc: U.S. Nuclear Regulatory Commission
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COMMITMENTS ASSOCIATED WITH IPN-94-076

Comm. No.	Commitment Description	Due Date
IPN-94-076-01	The Authority expects to complete its analysis for the Intermediate Shell Axial Welds (2-042A,B,C) and the Lower Shell Axial Welds (3-042A,B,C) by June 30, 1995, pending completion of the Reactor Vessel Integrity Database (RVID).	June 30, 1995 (pending completion of the RVID).