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LICENSEE EVENT	REPORT	(LER) TEXT	CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 -

YEAR SEQUENTIAL NUMBER REVISION NUMBER Indian Point, Unit 3 0 5 0 0 2 8 6 8 7 - 0 1 0 - 0 0 0 2 0F 0	FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)		
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At 2336 hours on September 3, 1987, with the reactor subcritical, the Reactor Operator was in the process of manually inserting control rods in order to shutdown for turbine maintenance. During the course of this shutdown a reactor trip and Safety Injection (SI) actuation occurred automatically. Just prior to 2336 hours, No. 32 static inverter was taken out of service by opening its output breaker to diagnose an instability problem. No. 32 static inverter normally supplies power to instrument bus No. 32. At this time, however, power was being supplied to instrument bus 32 through the No. 32 static inverter maintenance bypass switch. An operator, while troubleshooting No. 32 static inverter, did not recognize that the output breaker on No. 32 static inverter was opened when he opened the maintenance bypass switch to take final voltage readings. This action caused instrument bus 32 voltage to drop to zero.

The loss of power to instrument bus 32 (Protection Channel I) caused associated reactor protection relays to de-energize and initiated a reactor trip via the Nuclear Instrumentation System (NIS) Intermediate Range 35 High Flux signal. All equipment functioned properly for the reactor trip. Approximately three minutes after the reactor trip a "High Steam Flow" SI actuation occurred and all equipment functioned properly for the SI. No water was injected into the reactor because the Reactor Coolant System (RCS) was at normal operating pressure.

This "High Steam Flow" SI actuation signal requires that steam flow be greater than a programmed set-point for steam flow on at least one of two transmitters in at least two of four steam lines, concurrent with either low steam generator pressure (less than 616 psig in at least two of four steam generators) or low Taverage (less than 542°F in at least two of four RCS loops). The first half of the logic was satisfied when the loss of power to instrument bus 32 caused all SI relays associated with Protection Channel I steam flow transmitters to de-energize. The remaining half of the logic was satisfied when the loss of power to instrument bus 32 caused the SI logic relay associated with low Taverage for loop 1 to de-energize coincident with actual RCS loop 2 Taverage decreasing below the low set-point of 542°F. Power was restored to instrument bus 32 by reclosing the maintenance bypass switch on No. 32 static inverter at approximately 2353 hours.

In order to preclude recurrence, a discussion of this event will be incorporated into the Licensed Operator requalification program. Instrument bus power supplies (normal and backup) and inverter operations will also be emphasized at this time.

No similar events have been reported in an LER to date. After turbine repairs were made, the generator was synchronized to the bus on September 5, 1987 at 1125 hours. This event is reportable under 10CFR50.73 (a)(2)(iv) as an engineered safety feature actuation.

NRC Form 366A

Indian Point 3 Nuclear Power Plant P.O. Box 215 Buchanan, New York 10511 914 739.8200



NewYork Power Authority

October 5, 1987 IP3-87-059Z IP3-JJA-216H

Docket No. 50-286 License No. DPR-64

Document Control Desk U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Dear Sir:

The attached Licensee Event Report LER 87-010-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in Paragraph 50.73 (a) (2) (iv).

Very tryly yours, K.r Cursel

William A. Josiger Resident Manager Indian Point 3 Nuclear Power Plant

JJA:sn:LER3:10 Attachment

cc: Mr. William Russell Regional Administrator Region 1 U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, Pennsylvania 19406

> INPO Records Center Suite 1500 1100 Circle 75 Parkway Atlanta, Georgia 30339