

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Indian Point Unit 3										DOCKET NUMBER (2) 0 5 0 0 0 2 8 6 1 OF 0 2										PAGE (3) 1 OF 0 2																					
TITLE (4) REFUELING WATER STORAGE TANK LOW LEVEL ALARMS BELOW TECHNICAL SPECIFICATION ALARM ACTUATION RANGE.																																									
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																							
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES						DOCKET NUMBER(S)								
0 5			0 6			8 7			8 7			0 0			0 7			0 0			0 6			0 5			8 7									0 5 0 0 0 0					
OPERATING MODE (9) N						THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)																73.71(b) 73.71(c) OTHER (Specify in Abstract below and in Text, NRC Form 366A)																			
POWER LEVEL (10) 0 1 0 1 0						20.402(b) 20.405(a)(1)(i) 20.405(a)(1)(ii) 20.405(a)(1)(iii) 20.405(a)(1)(iv) 20.405(a)(1)(v)						20.405(c) 90.36(e)(1) 90.36(e)(2) 90.73(a)(2)(i) 90.73(a)(2)(ii) 90.73(a)(2)(iii)						90.73(a)(2)(iv) 90.73(a)(2)(v) 90.73(a)(2)(vi) 90.73(a)(2)(vii)(A) 90.73(a)(2)(vii)(B) 90.73(a)(2)(viii)																							
LICENSEE CONTACT FOR THIS LER (12)																																									
NAME Maryanne F. Paluck - Assistant Reactor Engineer																TELEPHONE NUMBER 9 1 4 7 3 6 - 8 3 4 4																									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																									
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC				CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC																					
X		B		P L I T		B O 8 0		Y																																	
X		B		P L I T		F 1 8 0		Y																																	
SUPPLEMENTAL REPORT EXPECTED (14)																EXPECTED SUBMISSION DATE (15)		MONTH		DAY		YEAR																			
YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO <input type="checkbox"/>																																									

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On May 6, 1987, while performing Surveillance Test 3PC-R10 "Refueling Water Storage Tank (RWST) Level Calibration" with the reactor at cold shutdown for a refueling outage, it was noted that Transmitter LT-920 and indicator LIC-921 did not meet the As Found test acceptance criteria. Specifically, calibration points on LT-920 and the Low Level Alarm set-point on LIC-921 were above applicable test limits. This resulted in actual Low Level Alarm set-points for the Refueling Water Storage Tank being below the required Technical Specification (TS) actuation range of 98,100 gallons to 100,850 gallons. The affected instruments were immediately recalibrated to meet the test acceptance criteria. This event was caused by instrument drift.

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PDR ADOCK 05000286
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Indian Point, Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 8 6 8 7 — 0 0 7 — 0 0 0 2 OF 0	LER NUMBER (3)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 388A's) (17)

On May 6, 1987, with the reactor in a cold shutdown condition for a refueling outage, Surveillance Test 3PC-R10 "Refueling Water Storage Tank (RWST) Level Calibration" did not meet the test acceptance criteria for the As Found condition of Level Transmitter LT-920 (codes X, BP, LIT, F180, Foxboro No. E-11-GM-SAA1) and Level Indicator LIC-921 (codes X, BP, LIT, B080, Barton No. 288) because of instrument drift. Technical Specification (TS) 3.3.A.1.b. states that whenever Reactor Coolant System (RCS) average temperature is above 200 degrees Fahrenheit, one Refueling Water Storage Tank low level alarm must be operable and set to alarm between 98,100 gallons and 100,850 gallons of water in the tank. For RCS temperature greater than 350 degrees Fahrenheit, the alarms are required to be operable as per Technical Specification 3.3.A.3.k. Low level alarm set-points on bistables LC-920A, LC-920B (from transmitter LT-920) and Level Transmitter LIC-921 were calculated to be at 92,609 gallons, 91,582 gallons, and 65,421 gallons, respectively, all below the specified range. Although this condition was discovered with the plant at cold shutdown, the potential exists for having operated outside of Technical Specification limits when the plant was above 200 degrees F. Therefore, this event is reportable under 10CFR 50.73 (a)(2)(i)(B).

It should be noted, however, that two of the three set-points still provided for an average of about 6,000 gallons margin above the minimum design level of 86,093 gallons required in the RWST by the Final Safety Analysis Report. Above the minimum design level, there is adequate water in the RWST for the recirculation phase of core cooling following a design basis loss-of-coolant accident.

The transmitters were re-calibrated to meet the test acceptance criteria. No similar event has been reported in an LER to date.

Indian Point 3
Nuclear Power Plant
P.O. Box 215
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914 739.8200



June 5, 1987
IP3-WAJ-038Z
IP3-JJA-161H

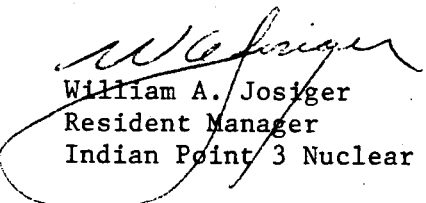
Docket No. 50-286
License No. DPR-64

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

The attached Licensee Event Report LER 87-007-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in Paragraph 50.73 (a) (2) (i).

Very truly yours,


William A. Josiger
Resident Manager
Indian Point 3 Nuclear Power Plant

JJA:sn:07
Attachment

cc: Mr. William Russell
Regional Administrator
Region 1
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

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1100 Circle 75 Parkway
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