NRC Form 366 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Indian Point, Unit 3	0 5 0 0 0 2 8 6	1 OF 0 3

TITLE (4) CHANGE IN LOW TEMPERATURE OVERPRESSURIZATION PROTECTION SYSTEM (LTOPS) SET-POINT CURVE NOT COMPLETELY REFLECTED IN PLANT DOCUMENTATION

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 5	0 5	8 7	8 7	0 0 6	0 0	0 6	0 1	8 7		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9): N

POWER LEVEL (10): 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)
	20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)
	20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		
	20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
	20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (AREA CODE)
Floyd W. Gumble - Nuclear Reactor Engineer	9 1 4 6 8 1 - 6 7 2 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 5, 1987, an engineering review of a routine plant cooldown on May 2, 1987 was conducted. It was determined that during this cooldown the Reactor Coolant System was operating at a pressure above the set-point established for the Power Operated Relief Valves (PORV) in the Low Temperature Overpressurization Protection System (LTOPS). (At no time during this event did RCS pressure exceed the pressure/temperature limits described in Figure 3.1-2 of the Technical Specifications.) Further investigation determined that the LTOPS had not been recalibrated and that the plant documentation had not been updated after a recent LTOPS Technical Specification (TS) amendment was put into effect. Hence, the operators were cooling down the plant in accordance with cooldown graphs that did not reflect the new TS curve. A review of plant parameters during cooldown confirmed that a pressurizer bubble of sufficient size existed throughout the period when the LTOPS was required operable (RCS temperature below 326 degrees F) to compensate for the RCS pressure exceeding the trip set-point. The plant documentation associated with the LTOPS set-point curve was not updated due to an administrative error. To preclude repetition of this event, a mechanism has been established and is being formalized to incorporate plant Technical Specifications amendments into plant operating documents

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

On May 5, 1987, an engineering review of a routine plant cooldown was conducted. It focused on a recent shutdown of the plant on May 2, 1987 for a scheduled refueling outage. The shutdown and cooldown were performed in accordance with established procedures. The engineering review revealed that , for a period of approximately one hour, while the Reactor Coolant System (RCS) temperature was descending from 326 to 318 degrees Fahrenheit, the RCS pressure was greater than the Technical Specification (TS) set-point for the Low Temperature Overpressurization Protection System (LTOPS). At no time during this event did RCS pressure exceed the pressure/temperature limits described in figure 3.1-2 of the TS.

Investigation determined that the LTOPS had not been recalibrated and the plant's Graphs Book had not been updated after LTOPS TS amendment was put into effect. Hence, the operators were cooling down the plant in accordance with cooldown graphs that did not reflect the new TS curve. In fact, the cooldown procedures and the LTOPS calibration continued to reflect Technical Specification Figure 3.1-2, which is the standard cooldown curve, exclusive of the LTOPS.

A review of plant parameters during cooldown confirmed that a pressurizer bubble of sufficient size existed throughout the period when the LTOPS was required operable (RCS temperature below 326 degrees F) to compensate for the RCS pressure exceeding the trip set-point. The TS allows operation at an RCS pressure above the PORV set-point curve (assuming the LTOPS is inoperable) provided that pressurizer level is low enough to compensate for any pressure spikes caused by design basis mass or heat input transients. Review of Control Room strip charts of RCS pressure, RCS average temperature and pressurizer level showed that this condition was met at all times.

The Control Room records for all low temperature operation since the new LTOPS curve came into effect were reviewed. This included one other cooldown on March 27th and one heat-up on April 2, 1987. For both of these cases, there was a bubble of sufficient size in the pressurizer whenever RCS pressure exceeded the LTOPS curve, and at no time were the pressure/temperature limits described in Figure 3.1-2 exceeded.

At no time was the plant outside design basis with respect to low temperature reactor vessel protection. Since RCS pressure was above the LTOPS TS set-point curve, this event is reportable under 10CFR50.73 (a)(2)(i)(B).

The operating and calibration procedures associated with the LTOPS set-point curve were not updated due to an administrative error. All of the other aspects of this TS amendment (including Reactor Coolant Pump starts, operations at low power, charging and Safety Injection Pump restrictions, and LTOPS operability requirements) were correctly incorporated into plant documentation.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

To preclude repetition of this event, a mechanism has been established and is being formalized to track the incorporation of TS amendments into plant procedures. A Quality Assurance Department special audit will be conducted prior to plant heat-up to confirm that this TS amendment has been fully incorporated.

No similar events have been reported to date.

Indian Point 3
Nuclear Power Plant
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NewYorkPower Authority

June 1, 1987
IP3-WAJ-030Z
IP3-JJA-137H

Docket No. 50-286
License No. DPR-64

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

The attached Licensee Event Report LER 87-006-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in Paragraph 50.73 (a) (2) (i) (B).

Very truly yours,

William A. Josiger
Resident Manager
Indian Point 3 Nuclear Power Plant

JJA:sn:06
Attachment

cc: Mr. William Russell
Regional Administrator
Region 1
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

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