POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NO. 3 NUCLEAR POWER PLANT
P. O. BOX 215 BUCHANAN, N. Y. 10511
TELEPHONE: 914-739-8200

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December 8, 1980
IP-FG-10896

Docket No. 50-286
License No. DPR-64

Boyce H. Grier, Director
Office of Inspection & Enforcement
Region 1
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pa. 19406

Dear Mr. Grier:

This letter is to inform you of a recent site event of which, although it is not reportable under the Technical Specifications as a Licensee Event Report, we feel you should be made aware.

On November 9, 1980, while the reactor was in the cold shutdown condition, a retest for a modification to the Engineered Safety Features System (ESFS) was performed. During the course of the retest it was observed that two Westinghouse MG-6 relays in Train A, (i.e., Containment Isolation Phase B and Containment Ventilation Isolations) failed to latch mechanically upon receipt of a safety injection activation signal. The relays return to their normal mode upon the removal of the safety injection signal, should they fail to latch mechanically. It is worth noting that these relays actuated as required upon receipt of a safeguards signal.

Upon examination of the relays in question, it was observed that the latch mechanisms associated with relays CB-1 and V-1 (Westinghouse Style MG-6, Model No. 289B36A11) were found to be out of adjustment. An adjustment screw on the latch mechanism was then reset to provide mechanical latch-up upon receipt of the activation signal, as required.

These relays are tested routinely on a monthly basis, and the testing procedure has been modified such that each latching mechanism is checked for operability.

As this condition does not on its own prevent the satisfactory operation of the ESFS, this item does not qualify as a reportable occurrence. However, as the performance of the ESFS reset is of particular importance to the operation of the plant, this event is reported for your information.

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For purposes of statistical comparison and convenience of storage, this report has been encapsulated onto a standard Licensee Event Report (LER) form, enclosed herein as Attachment 1. With the exception of Item 17 (LER/RO Report Number) all categories have been completed as usual.

Very truly yours,

S. S. Zulla
Resident Manager

FG/bam

cc: Director of Nuclear Reactor Regulation
Attn: William McDonald, Director (3 Copies)
Office of Management Information and Program Control
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Office of Inspection and Enforcement (30 Copies)
c/o Distribution Services Branch, DDC, ADM
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. George T. Berry
Power Authority of the State of New York
10 Columbus Circle
New York, New York 10019

Resident Inspector T. Rebelowski
George Wilverding, NYO
J. P. Bayne, NYO
James Davis (SRC), NYO

Nuclear Safety Analysis Center
Electric Power Research Institute
P. O. Box 10412
Palo Alto, California 94303

(7-77)

U.S. NUCLEAR REGULATORY COMMISSION

Attachment 1

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 NYIPS3 (2) 00-00000-00 (3) 41111 (4) (5)
LICENSEE CODE — LICENSE NUMBER — LICENSE TYPE — CAT

CON'T 01 REPORT SOURCE L (6) 0500286 (7) 110980 (8) 120980 (9)
DOCKET NUMBER — EVENT DATE — REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 WHILE AT COLD SHUTDOWN, DURING THE PERFORMANCE OF A RETEST FOR A
03 MODIFICATION TO THE ENGINEERED SAFETY FEATURES SYSTEM, TWO WESTINGHOUSE
04 MG-6 RELAYS IN TRAIN A FAILED TO LATCH MECHANICALLY AFTER RECEIPT OF THE
05 SAFETY INJECTION ACTIVATION SIGNAL. THIS CONDITION DOES NOT ON ITS OWN
06 PREVENT THE SATISFACTORY OPERATION OF THE ENGINEERED SAFETY FEATURES
07 SYSTEM AND IS THEREFORE NOT REPORTABLE UNDER THE TECHNICAL
08 SPECIFICATIONS.

09 SYSTEM CODE SF (11) CAUSE CODE E (12) CAUSE SUBCODE A (13) COMPONENT CODE RELAYX (14) COMP. SUBCODE A (15) VALVE SUBCODE Z (16)

(17) LER/RO REPORT NUMBER: EVENT YEAR — SEQUENTIAL REPORT NO. — OCCURRENCE CODE — REPORT TYPE — REVISION NO.

ACTION TAKEN E (18) FUTURE ACTION Z (19) EFFECT ON PLANT Z (20) SHUTDOWN METHOD Z (21) HOURS (22) 0000 ATTACHMENT SUBMITTED N (23) NPRD-4 FORM SUB. N (24) PRIME COMP. SUPPLIER N (25) COMPONENT MANUFACTURER W120 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 THE LATCH MECHANISMS ASSOCIATED WITH RELAYS CB-1 AND V-1 (WESTINGHOUSE
11 STYLE MG-6, MODEL NO. 289B36A11) WERE FOUND TO BE OUT OF ADJUSTMENT.
12 THESE WERE THEN READJUSTED TO PROVIDE MECHANICAL LATCH-UP FEATURE UPON
13 SAFETY INJECTION ACTIVATION SIGNAL, AS REQUIRED. NO SIMILAR EVENTS HAVE
14 BEEN REPORTED TO DATE.

15 FACILITY STATUS G (28) % POWER 000 (29) OTHER STATUS (30) NA METHOD OF DISCOVERY B (31) DISCOVERY DESCRIPTION (32) PERFORMANCE OF RETEST 00319

16 ACTIVITY RELEASED Z (33) CONTENT OF RELEASE Z (34) AMOUNT OF ACTIVITY (35) NA LOCATION OF RELEASE (36) NA

17 PERSONNEL EXPOSURES NUMBER 000 (37) TYPE Z (38) DESCRIPTION (39) NA

18 PERSONNEL INJURIES NUMBER 000 (40) DESCRIPTION (41) NA

19 LOSS OF OR DAMAGE TO FACILITY (43) TYPE Z (42) DESCRIPTION NA

20 PUBLICITY ISSUED N (44) DESCRIPTION (45) NA NRC USE ONLY

NAME OF PREPARER FLOYD W. GUMBLE PHONE: 914-739-8200, X217

GPO 917-926