

POWER AUTHORITY OF THE STATE OF NEW YORK

10 COLUMBUS CIRCLE NEW YORK, N. Y. 10019
(212) 397-6200

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June 17, 1980
IPN-80-57

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Mr. Boyce H. Grier
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Region 1
631 Park Avenue
King of Prussia, Pa. 19406

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
I. E. Bulletin No. 80-06
Engineered Safety Feature Reset Controls

Dear Sir:

The purpose of this letter is to respond to I. E. Bulletin No. 80-06, "Engineered Safety Feature (ESF) Reset Controls", dated March 13, 1980.

Attachment 1 provides the responses to your questions contained in the subject Bulletin.

Very truly yours,

J. P. Bayne
Senior Vice President
Nuclear Generation

cc: Mr. T. Rebelowski
Resident Manager
U. S. Nuclear Regulatory Commission
P. O. Box 38
Buchanan, New York 10511

Subscribed and sworn to this 17 day of
June, 1980

Notary Public

RUTH G. ZAPP
Notary Public, State of New York
Qualified In Nassau County
Commission Expires March 30, 1981

ATTACHMENT 1

ENGINEERED SAFETY FEATURE RESET CONTROLS

POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT 3 NUCLEAR POWER PLANT
DOCKET NO. 50-286
JUNE 17, 1980

1. Review the drawings for all systems serving safety-related functions at the schematic level to determine whether or not upon the reset of an ESF actuation signals, all associated safety-related equipment remains in its emergency mode.

Response:

- 1) Schematic drawings for all systems serving safety-related functions were reviewed and it was determined that all safety-related equipment remains in its emergency mode upon the reset of an ESF actuation signal except for the following listed items.

<u>Component/System</u>	<u>Remarks</u>
Valve SOV 1170 (Train 1) Valve SOV 1171 (Train 2) Containment Cooling Water	Valve opens on SI signal and will close on SI reset.
Valve SOV 1276 (Train 1) Valve SOV 1276A (Train 2) D/G Cooling Water	Valve opens on SI signal and will close on SI reset
5 - Air Units (Train 1) 5 - Air Units (Train 2) Containment Recirculation	Dampers positions as required on SI signal and will be repositioned on SI reset.
Diesel Generator No. 31 Diesel Generator No. 32 Diesel Generator No. 33 Emergency Power	In the event SI is reset and either an overcurrent or reverse power condition exists at time of reset, generator breaker will trip. This is a design feature and as such is acceptable.
Valve 1851A (Train 1) Valve 1851B (Train 2) Boron Injection Tank Recirculation Iso. Valve	Valve closes on SI signal and will open on SI reset.
Valve 876A (Train 1) Valve 876B (Train 2) Containment Spray	Valve opens 2 minutes after Containment Spray (CS) actuation and will close on CS reset.

2. Verify the actual installed instrumentation and controls at the facility are consistent with the schematics reviewed in Item 1 above by conducting a test to demonstrate that all equipment remains in its emergency mode upon removal of the actuating signal and/or manual resetting of the various isolating or actuation signals. Provide a schedule for the performance of the testing in your response to this Bulletin.

Response:

Valves associated with the Containment Isolation (CI) system were modified and tested in response to NUREG-0578 during the 1979 refueling outage and demonstrated proper operation as per the schematic diagrams. Copies of test reports are available. Additionally, test procedures will be reviewed and ensured that the other systems are tested as required. If this documentation does not exist, the system will be tested at the next test interval for the particular component.

3. If any safety-related equipment does not remain in its emergency mode upon reset of an ESF signal at your facility, describe proposed system modification, design change, or other corrective action planned to resolve the problem.

Response:

The existing operating procedures require the operator to place the equipment in such a mode that the resetting action will not affect the ESF system operation.

4. Report in writing within 90 days, the results of your review and include a list of all devices which respond as discussed in item 3 above, actions taken or planned to assure adequate equipment control, and a schedule for implementation of corrective action. This information is requested under the provisions of 10 CFR 50.54(f). Accordingly, you are requested to provide within the time period specified above, written statements of the above information, signed under oath or affirmation. Reports shall be submitted to the Director of the appropriate NRC Regional Office and a copy shall be forwarded to the NRC Office of Inspection and Enforcement, Division of Reactor Operations Inspection, Washington, D.C. 20555.

Response:

The data listed in Items 1, 2 and 3 fulfill the requirements of the requested report.