

Composition

6.5.2.2 The Committee shall have a permanent voting membership of at least 5 persons of which a majority are independent of the Nuclear Power organization and shall include technically competent persons from departments of Consolidated Edison having a direct interest in nuclear plant design, construction, operation or in nuclear safety. In addition, persons from departments not having a direct interest in nuclear plant design, construction, operation or nuclear safety may serve as members of the Committee if experienced in the field of nuclear energy. The Chairman or Vice Chairman will be a senior official of the Company. The Chairman and Vice Chairman will be experienced in the field of nuclear energy.

The Chairman of the Nuclear Facilities Safety Committee, hereafter referred to as the Chairman, shall be appointed by the Senior Vice President, Nuclear Operations.

The Vice Chairman shall be appointed by the Senior Vice President, Nuclear Operations. In the absence of the Chairman, he will serve as Chairman.

The Secretary shall be appointed by the Chairman of the Committee.

Committee members from departments of Consolidated Edison shall be designated by the NFSC Chairman with the concurrence of the appropriate department Vice President. Committee members from outside Consolidated Edison shall be designated by the NFSC Chairman with the concurrence of the Senior Vice President, Nuclear Operations.

- d. The Safety Limit Violation Report shall be submitted to the Commission, the NFSC Chairman and the Vice President-Nuclear Power within 10 days of the violation.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained covering the activities referenced below:

- a. The requirements and recommendations of Sections 5.2 and 5.3 of ANSI N18.7-1976 and Appendix A of Regulatory Guide 1.33, Revision 2 except as provided in 6.8.2 and 6.8.3 below.
- b. Process Control Program implementation.
- c. Offsite Dose Calculation Manual implementation.
- d. Quality Assurance Program for effluent and environmental monitoring using the guidance in Regulatory Guide 1.21, Revision 1, April 1974 and Regulatory Guide 4.1, Revision 1, April 1975.
- e. Fire Protection Program implementation.

6.8.2 Each procedure and administrative policy of Specification 6.8.1 above, and any changes to them shall be reviewed and approved for implementation in accordance with a written administrative control procedure approved by the Plant Manager, or appropriate Department Manager, with the concurrence of the Station Nuclear Safety Committee and the Vice President, Nuclear Power. The administrative control procedure required by this specification shall, as a minimum, require that:

- a. Each proposed procedure/procedure change involving safety-related components and/or operation of same receives a pre-implementation review by the SNSC except in case of an emergency.
- b. Each proposed procedure/procedure change which renders or may render the Updated Final Safety Analysis Report or subsequent safety analysis reports inaccurate and those which involve or may involve potential unreviewed safety questions are approved by the SNSC prior to implementation.

ATTACHMENT II
SAFETY ASSESSMENT

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT UNIT NO. 2
DOCKET NO. 50-247
OCTOBER 1998

Description of Proposed Changes:

1) TS 6.5.2.1.j: delete the words “nuclear power plant” from the top of page 6-7. This typographical error was introduced during document word processing.

2) TS 6.5.2.2: In the last sentence of the first paragraph revise: “The Chairman and Vice Chairman will be senior officials of the Company experienced in the field of nuclear energy.” to read: “The Chairman or Vice Chairman will be a senior official of the Company. The Chairman and Vice Chairman will be experienced in the field of nuclear energy.”

Further: Replace the last paragraph with the following: “Committee members from departments of Consolidated Edison shall be designated by the NFSC Chairman with the concurrence of the appropriate department Vice President. Committee members from outside Consolidated Edison shall be designated by the NFSC Chairman with the concurrence of the Senior Vice President Nuclear Operations.” These changes are required to provide flexibility in retaining industry experts (not Company officials) as either the Chairman, or Vice Chairman, and as NFSC committee members.

3) TS 6.7.1.c: delete the following: “systems or structures, and (3) corrective action taken to prevent recurrence.” from the top of page 6-12. This typographical error was introduced during document word processing.

4) TS 6.8.1.a: change “5.1” to “5.2”. This typographical change is required to accurately reflect the specific section of the referenced ANSI Standard. ANSI N18.7-1972 refers to section 5.1, while ANSI N18.7-1976 refers to section 5.2.

Basis for No Significant Hazards Consideration Determination

The proposed changes do not involve a significant hazards consideration because:

1. There is no significant increase in the probability or consequences of an accident previously evaluated.

The proposed amendment is administrative in nature. It involves a change in 1) the Nuclear Facilities Safety Committee (NFSC) Chairman or Vice Chairman to allow the services of an individual other than a senior official of the Company, and 2) allowing NFSC membership by other than Con Edison employees. In either case, concurrence by the Senior Vice President, Nuclear Operations is required.

These changes do not affect possible initiating events for accidents previously evaluated or alter the configuration or operating of the facility. The Limiting Safety Systems Settings and Safety Limits specified in the current Technical Specifications remain unchanged. Therefore, the proposed changes to the subject Technical Specification would not increase the probability or consequences of an accident previously evaluated.

2. The possibility of a new or different kind of accident from any accident previously evaluated has not been created.

As stated above, the proposed changes are administrative in nature. The safety analysis of the facility remains complete and accurate. There are no physical changes to the facility, and the plant conditions for which the design basis accidents have been evaluated are still valid. The operating procedures and emergency procedures are unaffected. Consequently, no new failure modes are introduced as a result of the proposed changes. Therefore, the proposed changes will not initiate any new or different kind of accident.

3. There has been no significant reduction in the margin of safety.

The proposed changes are administrative in nature. Since there are no changes to the operation of the facility or physical design the Updated Final Safety Analysis Report (UFSAR) design basis, accident assumptions, or Technical Specification Bases are not affected. Therefore, the proposed changes will not result in a reduction in the margin of safety.

The proposed changes has been reviewed by both the Station Nuclear Safety Committee (SNSC) and the Con Edison Nuclear Facility Safety Committee (NFSC). Both Committees concur that the proposed changes do not represent a significant hazards consideration.