

ATTACHMENT I

PROPOSED TECHNICAL SPECIFICATION CHANGES

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT UNIT NO. 2

DOCKET NO. 50-247

JUNE 1998

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operating procedures. If the requirements of 3.3.E.1 are not satisfied within an additional 48 hours, the reactor shall be placed in the cold shutdown condition utilizing normal operating procedures.

- a. One of the three operable component cooling pumps may be out of service provided the pump is restored to operable status within 14 days. Component Cooling Pump 22 may be out of service if Emergency Diesel Generator 22 is out of service or if no emergency diesel generator is out of service.
- b. An additional component cooling pump may be out of service provided a second pump is restored to operable status within 24 hours.
- c. One auxiliary component cooling pump may be out of service provided the pump is restored to operable status within 24 hours and the other pump is operable.
- d. One component cooling heat exchanger or other passive component may be out of service for a period not to exceed 48 hours provided the system may still operate at design accident capability.

F. SERVICE WATER SYSTEM

1. DESIGNATED ESSENTIAL HEADER

- a. The reactor shall not be above 350°F unless three service water pumps with their associated piping and valves are operable on the designated essential header.
- b. When the reactor is above 350°F and one of the three service water pumps or any of its associated piping or valves is found inoperable, and an essential service water header that meets the requirements of 3.3.F.1.a. cannot be restored within 12 hours, the reactor shall be placed in the hot shutdown condition within the next 6 hours and subsequently cooled below 350°F using normal operating procedures.

Composition

6.5.2.2 The Committee shall have a permanent voting membership of at least 5 persons of which a majority are independent of the Nuclear Power organization and shall include technically competent persons from departments of Consolidated Edison having a direct interest in nuclear plant design, construction, operation or in nuclear safety. In addition, persons from departments not having a direct interest in nuclear plant design, construction, operation or nuclear safety may serve as members of the Committee if experienced in the field of nuclear energy. The Chairman and Vice Chairman will be senior officials of the Company experienced in the field of nuclear energy.

The Chairman of the Nuclear Facilities Safety Committee, hereafter referred to as the Chairman, shall be appointed by the Senior Vice President, Nuclear Operations.

The Vice Chairman shall be appointed by the Senior Vice President, Nuclear Operations. In the absence of the Chairman, he will serve as Chairman.

The Secretary shall be appointed by the Chairman of the Committee.

Committee members from departments having a direct interest in nuclear plant design, construction and operation or in nuclear safety shall be designated by the Vice President of the Company, who is responsible for the functioning of the department subject to the approval of the Chairman. Committee members from other departments may be appointed by the Chairman with the concurrence of the Vice President of that department.

- e. The Facility Emergency Plan and implementing procedures at least once per 12 months.
- f. The Facility Security Plan and implementing procedures at least once per 12 months.
- g. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- h. A fire protection and loss prevention inspection and audit shall be performed utilizing either qualified offsite licensee personnel or an outside fire protection firm at least once per 12 months.
- i. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at least once per 36 months.
- j. The radiological environmental monitoring program and the results thereof at least once per 12 months.
- k. The Offsite Dose Calculations Manual and implementing procedures at least once per 24 months.
- l. The Process Control Program and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.
- m. The performance of activities required by the Quality Assurance Program to meet the provisions of Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975 at least once per 12 months.
- n. Any other area of facility operation considered appropriate by the NFSC or the Senior Vice President, Nuclear Operations.

Authority

6.5.2.9 The NFSC shall report to and advise the Senior Vice President, Nuclear Operations on those areas of responsibility in Specifications 6.5.2.7 and 6.5.2.8.

Records

6.5.2.10 Records of NFSC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NFSC meeting shall be prepared, approved and forwarded to the Senior Vice President, Nuclear Operations and to Senior Company Officers concerned with nuclear facilities within 30 days following each meeting.
- b. Reports of reviews encompassed by Specifications 6.5.2.7 e, f, g and h above, shall be prepared, approved and forwarded to the Senior Vice President, Nuclear Operations and to Senior Company Officers concerned with nuclear facilities within 30 days following completion of the review.
- c. Audit reports encompassed by Specification 6.5.2.8 above, shall be forwarded to the Senior Company Officers concerned with nuclear facilities and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE EVENT ACTION

6.6.0 A Reportable Event is defined as any of the conditions specified in 10 CFR 50.73a(2).

6.6.1 The following actions shall be taken in the event of a Reportable Event:

- a. A report shall be submitted to the Commission pursuant to the requirements of 10 CFR 50.73.
- b. Each Licensee Event Report submitted to the Commission shall be submitted to the NFSC Chairman and the Vice President-Nuclear Power and be reviewed by the SNSC.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The Safety Limit Violation Report shall be reported to the Commission, the Vice President-Nuclear Power and to the NFSC Chairman immediately.
- c. The Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

systems or structures, and (3) corrective action taken to prevent recurrence.

- d. The Safety Limit Violation Report shall be submitted to the Commission, the NFSC Chairman and the Vice President-Nuclear Power within 10 days of the violation.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained covering the activities referenced below:

- a. The requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7-1976 and Appendix A of Regulatory Guide 1.33, Revision 2 except as provided in 6.8.2 and 6.8.3 below.
- b. Process Control Program implementation.
- c. Offsite Dose Calculation Manual implementation.
- d. Quality Assurance Program for effluent and environmental monitoring using the guidance in Regulatory Guide 1.21, Revision 1, April 1974 and Regulatory Guide 4.1, Revision 1, April 1975.
- e. Fire Protection Program implementation.

6.8.2 Each procedure and administrative policy of Specification 6.8.1 above, and any changes to them shall be reviewed and approved for implementation in accordance with a written administrative control procedure approved by the Plant Manager, or appropriate Department Manager, with the concurrence of the Station Nuclear Safety Committee and the Vice President, Nuclear Power. The administrative control procedure required by this specification shall, as a minimum, require that:

- a. Each proposed procedure/procedure change involving safety-related components and/or operation of same receives a pre-implementation review by the SNSC except in case of an emergency.
- b. Each proposed procedure/procedure change which renders or may render the Updated Final Safety Analysis Report or subsequent safety analysis reports inaccurate and those which involve or may involve potential unreviewed safety questions are approved by the SNSC prior to implementation.

- c. The approval of the Nuclear Facilities Safety Committee shall be sought if, following its review, the Station Nuclear Safety Committee finds that the proposed procedure/procedure change either involves an unreviewed safety question or if it is in doubt as to whether or not an unreviewed safety question is involved.

6.8.3 A mechanism shall exist for making temporary changes and they shall only be made by approved management personnel in accordance with the requirements of ANSI 18.7-1976. The change shall be documented, and reviewed by the SNSC and approved by the Plant Manager, or appropriate Department Manager, within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

- a. A program which will ensure the capability to obtain and analyze samples of reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere under accident conditions. The program shall include the following:

- (i) training of personnel,

- (ii) procedures for sampling and analysis, and

- (iii) provisions for maintenance of sampling and analysis equipment.

6.9 REPORTING REQUIREMENTS

Routine Reports and Reportable Occurrences

- 6.9.1. In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Regional Administrator, Region I unless otherwise noted.

ATTACHMENT II
SAFETY ASSESSMENT

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT UNIT NO. 2
DOCKET NO. 50-247
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Description of Proposed Changes:

- 1) TS section 6.5.2.2, 6.5.2.8.n, 6.5.2.9, 6.5.2.10a, and 6.5.2.10b: remove the words “Senior Vice President, Central Operations ” and replace with “Senior Vice President, Nuclear Operations”.
- 2) TS section 6.8.1 : Replace “USAEC Regulatory Guide 1.33 (issued November 1972)” with “Regulatory Guide 1.33, Revision 2”. This change is required to reflect our response to the NRC’s March 23, 1980 letter requesting additional information on the IP2 Quality Assurance program, as transmitted by Reference 4.
- 3)TS section 6.8.1 and 6.8.3: Replace “ANSI N18.7-1972” with “ANSI N18.7-1976”. This change is required to reflect our response to the NRC’s March 23, 1980 letter requesting additional information on the IP2 Quality Assurance program, as transmitted by Reference 4.
- 4) TS section 3.3.E: replace page 3.3-7, originally provided via Reference 3, with the new page 3.3-7 provided herein. This change is required to correct the pagination of text.

Basis for No Significant Hazards Consideration Determination

The proposed changes do not involve a significant hazards consideration because these changes are administrative in nature and provide clarification. Accordingly, the Safety Assessment provided as Attachment II to Reference 1 remains in effect.