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April 5, 1995

Re: Indian Point Unit No. 2  
Docket No. 50-247

Document Control Desk  
US Nuclear Regulatory Commission  
Mail Station P1-137  
Washington, DC 20555

SUBJECT: Response to Request for Additional Information (RAI)  
Pertaining to Licensee's April 13, 1994 Steam Generator  
Tube Acceptance Criteria Technical Specification  
Amendment Request

By letter dated February 15, 1995, the Nuclear Regulatory Commission requested additional information to complete its review of the Consolidated Edison Company of New York (Con Edison) proposed amendment to Technical Specifications involving steam generator tube sleeving for Indian Point Station, Unit 2.

Enclosure 1 of this letter contains the Con Edison response to the RAI and Enclosure 2 contains reconciled Technical Specification page changes consistent with our submittal of April 13, 1994.

By September 1, 1995, Con Edison will submit a license amendment request for those items contained in RAI item Nos. 3 and 4.

Should you or your staff have any questions regarding this matter, please contact Mr. Charles W. Jackson, Manager, Nuclear Safety and Licensing.

Very truly yours,



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cc: Mr. Thomas T. Martin  
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ENCLOSURE 1

CON EDISON RESPONSE TO  
NRC REQUEST FOR ADDITIONAL INFORMATION  
STEAM GENERATOR TUBE SLEEVING

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.  
INDIAN POINT UNIT NO. 2  
DOCKET NO. 50-247  
APRIL, 1995

CONSOLIDATED EDISON RESPONSE TO NRC  
REQUEST FOR ADDITIONAL INFORMATION  
REQUIRED FOR THE REVIEW OF  
STEAM GENERATOR TUBE SLEEVING  
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

Item No. 1:

Provide a commitment to use enhanced and improved Eddy Current (ET) inspection techniques as they are developed and verified for use.

Response:

Consolidated Edison commits to using enhanced and improved Eddy Current (ET) inspection techniques as they are developed and verified for use. Consolidated Edison believes that the use of the bobbin/CECCO C5 or C3 eddy current probe currently satisfies such a commitment.

CONSOLIDATED EDISON RESPONSE TO NRC  
REQUEST FOR ADDITIONAL INFORMATION  
REQUIRED FOR THE REVIEW OF  
STEAM GENERATOR TUBE SLEEVING  
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

Item No. 2:

Provide a commitment to perform post-weld heat treatment of installed laser weld sleeves.

Response:

Consolidated Edison commits to perform post-weld heat treatment of installed laser welded sleeves.

CONSOLIDATED EDISON RESPONSE TO NRC  
REQUEST FOR ADDITIONAL INFORMATION  
REQUIRED FOR THE REVIEW OF  
STEAM GENERATOR TUBE SLEEVING  
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

Item No. 3

Provide a commitment to perform additional confirmatory testing to establish the design life of the sleeves and to confirm that leakage detection requirements will be met.

Response:

Consolidated Edison commits to performing additional confirmatory testing to establish the design life of the sleeves and to confirm that leakage detection requirements will be met. Such testing is underway by Westinghouse in response to similar commitments made for the Commonwealth Edison Plants.

CONSOLIDATED EDISON RESPONSE TO NRC  
REQUEST FOR ADDITIONAL INFORMATION  
REQUIRED FOR THE REVIEW OF  
STEAM GENERATOR TUBE SLEEVING  
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

Item No. 4:

Provide a commitment to amend the license requirement for inservice inspection and primary-to-secondary leakage limits to account for the installation of sleeves into the steam generators.

Response:

Consolidated Edison commits to amending the license requirements to provide for appropriate inservice inspection for any steam generator tubes containing sleeves.

Consolidated Edison also commits to amending the license requirement for primary-to-secondary leakage limits to account for any installation of sleeves into the steam generators. Until such time as the license amendment is issued, Consolidated Edison will administratively limit primary-to-secondary leakage to no more than 150 gpd for any steam generator containing sleeves.

CONSOLIDATED EDISON RESPONSE TO NRC  
REQUEST FOR ADDITIONAL INFORMATION  
REQUIRED FOR THE REVIEW OF  
STEAM GENERATOR TUBE SLEEVING  
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

Item No. 5:

Parent tubes associated with Hybrid Expansion Joint (HEJ) mechanical sleeves installed previously in some plants have experienced cracking. Discuss the root cause of the cracking and the modifications to the sleeving process that will be made to prevent cracking in new installations.

Response:

Consolidated Edison does not intend to use HEJ mechanical sleeves at this time, therefore, this technical specification change will only apply to laser welded sleeves.