(iii) Summary description of action(s) taken to prevent a recurrence.

5. Liquid Holdup Tanks

a. The quantity of radioactive material contained in each of the following unprotected outdoor tanks shall be limited to less than or equal to 10 curies, excluding tritium and dissolved or entrained noble gases.

a. Refueling Water Storage Tank
b. Primary Water Storage Tank
c. 13, 14 Waste Distillate Storage Tanks
d. Outside temporary tank

b. With the quantity of radioactive material in any of the above listed tanks exceeding the above limit, immediately suspend all additions of radioactive material to the tank, take action within 48 hours to reduce the tank contents to within the limit, and describe the events leading to this condition in the next Annual Radioactive Effluent Release Report.

B. RADIOACTIVE GASEOUS EFFLUENTS

1. Gaseous Effluent Dose Rate

a. The dose rate due to radioactive materials released in gaseous effluents from the site to areas at and beyond the site boundary (see Figure 5.1-1) shall be limited to the following:

(i) for noble gases: Less than or equal to 500 mrem/yr to the total body and less than or equal to 3000 mrem/yr to the skin, and

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4.2 INSERVICE INSPECTION AND TESTING

Applicability

Applies to the inservice inspection of Quality Group* A, B, and C components and the inservice testing of pumps and valves whose function is required for safety.

Objective

To provide assurance of the continued integrity and/or operability of those structures, systems, and components to which this specification is applicable.

Specifications

4.2.1 Inservice Testing

Inservice testing of pumps and valves whose function is required for safety shall be performed in accordance with the applicable edition and addenda of Section XI of the ASME Boiler and Pressure Vessel Code as required by 10 CFR 50, Section 50.55a(g), except where specific written relief pursuant to 10 CFR 50, Section 50.55a has been granted.

4.2.2 Inservice Inspection

Inservice inspection of Quality Group* A, B, and C components shall be performed in accordance with the applicable edition and addenda of Section XI of the ASME Boiler and Pressure Vessel Code as Required by 10 CFR 50, Section 50.55a(g), except where specific written relief pursuant to 10 CFR 50, Section 50.55a has been granted.

* Quality Group classification is in accordance with Revision 3 of Regulatory Guide 1.26.

monitoring program. In lieu of a Licensee Event Report and pursuant to Specification 6.9.1.6, identify the cause of the unavailability of samples and identify the new location(s) for obtaining replacement samples in the next Annual Radioactive Effluent Release Report and also include in the report a revised figure(s) and table for the ODCM reflecting the new location(s).

B. LAND USE CENSUS

1. A land use census shall be conducted and shall identify within a distance of 8 km (5 miles) the location, in each of the 16 meteorological sectors, of the nearest milk animal, the nearest residence and the nearest garden* of greater than 50 m^2 (500 ft^2) producing broad leaf vegetation. (For elevated releases as defined in Regulatory Guide 1.111, Revision 1, July 1977, the land use census shall also identify within a distance of 5 km (3 miles) the locations, in each of the 16 meteorological sectors, of all milk animals and all gardens of greater than 50 m^2 producing broad leaf vegetation.)

2. The land use census shall be conducted during the growing seasons at least once per calendar year using that information which will provide the best results, such as by a door-to-door survey, aerial survey, or by consulting local agriculture authorities. The results of the land use census shall be included in the Annual Radiological Environmental Operating Report pursuant to Specification 6.9.1.5.

3. With a land use census identifying a location(s) that yields a calculated dose or dose commitment greater than the values currently being calculated in Specification 4.10.B.4, in lieu of a Licensee Event Report,

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* Broad leaf vegetation sampling of at least three different kinds of vegetation may be performed at the site boundary in each of two different direction sectors with the highest predicted D/Qs in lieu of the garden census. Specifications for broad leaf vegetation sampling in Table 4.11-1.4c shall be followed, including analysis of control samples.

identify the new location(s) in the next Annual Radioactive Effluent Release Report, pursuant to Specification 6.9.1.6.

4. With a land use census identifying a location(s) that yields a calculated dose or dose commitment (via the same exposure pathway) a factor of 2 greater than at a location from which samples are currently being obtained in accordance with Specification 4.11.A, add the new location(s) to the radiological environmental monitoring program within 30 days. The sampling location(s), excluding the control station location, having the lowest calculated dose or dose commitment(s), via the same exposure pathway, may be deleted from this monitoring program after (October 31) of the year in which this land use census was conducted. In lieu of a Licensee Event Report and pursuant to Specification 6.9.1.6, identify the new location(s) in the next Annual Radioactive Effluent Release Report and also include in the report a revised figure(s) and table for the ODCM reflecting the new location(s).

C. INTERLABORATORY COMPARISON PROGRAM

1. Analyses shall be performed on radioactive materials supplied as part of an Interlaboratory Comparison Program that has been approved by the Commission.

2. With analyses not being performed as required in Specification 4.11.C.1 above, report the corrective actions taken to prevent a recurrence to the Commission in the Annual Radiological Environmental Operating Report pursuant to Specification 6.9.1.5.

3. The Interlaboratory Comparison Program shall be described in the ODCM. A summary of the results obtained as part of the above required Interlaboratory Comparison Program shall be included in the Annual Radiological Environmental Operating Report pursuant to Specification 6.9.1.5.

Composition

6.5.1.2 The Station Nuclear Safety Committee shall, as a minimum, be composed of technically competent individuals, approved by the Vice President-Nuclear Power, in the following disciplines:

Chairman:	Senior Manager *
Member:	Engineering
Member:	Operations
Member:	Maintenance
Member:	Instrument and Control
Member:	Radiation Protection
Member:	Reactor Engineering

* This senior manager shall be a technically competent person experienced in the field of nuclear energy, shall be appointed by and report directly to the Vice President-Nuclear Power for the SNSC function and shall be independent of the Plant Manager.

6.5.1.2.1 In addition, other technically competent individuals may be appointed by the SNSC Chairman to serve as SNSC members.

Alternates

6.5.1.3 Alternate members shall be appointed in writing by the SNSC Chairman to serve on a temporary basis, and must have qualifications similar to the member being replaced.

Meeting Frequency

6.5.1.4 The SNSC shall meet at least once per calendar month and as convened by the SNSC Chairman or his designated alternate.

Quorum

6.5.1.5 A quorum of the SNSC shall consist of the Chairman or his designated alternate and four members. No more than two alternate members shall be included in the quorum.

Responsibilities

6.5.1.6 The Station Nuclear Safety Committee shall be responsible for:

a. review of (1) all procedures required by Specification 6.8 and changes thereto, and (2) any other proposed procedures or changes thereto as determined by the Chairman of SNSC to affect nuclear safety,

b. review of all proposed tests and experiments that affect nuclear safety,

c. review of all proposed changes to the Technical Specifications,

d. review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety,

e. investigation of all violations of the Technical Specifications and preparation and forwarding of a report covering evaluation and recommendations to prevent recurrence to the Vice President-Nuclear Power and to the Chairman of the Nuclear Facilities Safety Committee,

f. review of facility operations to detect potential nuclear safety hazards,

g. performance of special reviews and investigations and the issuance of reports thereon as required by the Chairman of the Nuclear Facilities Safety Committee,

h. review of any unplanned, radioactive release, including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President-Nuclear Power and to the Nuclear Facility Safety Committee, and

i. review of changes to the Process Control Program and the Offsite Dose Calculation Manual.

e. The Facility Fire Protection Program and implementing procedures at least once per 24 months.

f. A fire protection and loss prevention inspection and audit shall be performed utilizing either qualified offsite licensee personnel or an outside fire protection firm at least once per 12 months.

g. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at least once per 36 months.

h. The radiological environmental monitoring program and the results thereof at least once per 12 months.

i. The Offsite Dose Calculations Manual and implementing procedures at least once per 24 months.

j. The Process Control Program and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.

k. The performance of activities required by the Quality Assurance Program to meet the provisions of Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975 at least once per 12 months.

l. Any other area of facility operation considered appropriate by the NFSC or the Executive Vice President, Central Operations.

these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any specific details required in license conditions based on other commitments shall be included in this report.

6.9.1.2 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

ANNUAL RADIATION EXPOSURE REPORT[1]

6.9.1.3 Routine reports of occupational radiation exposure data during the previous calendar year shall be submitted no later than April 30 of each year.

6.9.1.4 The annual radiation exposure reports shall provide a tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions[2], e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty

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1 A single submittal may be made for a multiple-unit station. The submittal should combine those sections that are common to all units at the station.

2 This tabulation supplements the requirements of 10 CFR Part 20.407.

Table 6.2-1

Minimum Shift Crew Composition**

License Category	During Operations Involving Core Alterations	During Cold Shutdown or Refueling Periods	At All Other Times
Senior Operator License	2*	1	2***
Operator License	1	1	2
Non-Licensed	(As Required)	1	2
Watch Engineer	1	(None Required)	1

* Includes individual with SRO license supervising fuel movement as per Specification 6.2.2(e).

** Shift crew composition may be one less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

*** When the unit is in an operational mode other than cold shutdown or refueling, at least one Licensed Senior Reactor Operator must be in the Control Room at all times.