PMBelCOL PEmails

From: Spink, Thomas E [tespink@tva.gov]
Sent: Monday, January 25, 2010 8:14 AM
To: Hughes, Brian; Comar, Manny
Cc: Dorothy Boland; Fred Redwanz
Subject: Courtesy Copy - BLN COLA Submittal #5 Roadmap
Attachments: BLN Submittal 5 ROADMAP January 2010.pdf

Please find attached as a courtesy a copy the Roadmap for COLA Submittal #5. The paper copy should arrive on January 25, 2010.

If you have any questions, or if I can be of assistance, please let me know.

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Tennessee Valley Authority, 1101 Market Street, LP 5A, Chattanooga, Tennessee 37402-2801

January 22, 2010

10 CFR 52.79

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

In the Matter of)
Tennessee Valley Authority)

Docket No. 52-014 and 52-015

BELLEFONTE COMBINED LICENSE APPLICATION - UPDATE ROADMAP SUBMITTAL No. 5

This letter provides information supporting the recent Tennessee Valley Authority (TVA) update of the application for a combined license for Bellefonte Units 3 and 4. Enclosed is a "roadmap" of the changes included in the recent update, along with an explanation of the information contained in the roadmap.

If you should have any questions, please contact Tom Spink at 1101 Market Street, LP5A, Chattanooga, Tennessee 37402-2801, by telephone at (423) 751-7062, or via email at tespink@tva.gov.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on this 22nd day of January 2010

Andrea L. Sterdis

Manager, New Nuclear Licensing and Industry Affairs
Nuclear Generation Development & Construction

Enclosure
cc: See Page 2

Enclosure
TVA letter dated January 22, 2010
Update Roadmap

2010 S-COLA Update Roadmap Format Explanation **(by** columns)

Change **ID #** [unique identifier for tracking purposes]

COLA REP [identifies the change as STD (Standard) or BLN (Plant-Specific)]

COLA Part A [Part 1 (Pt 01) through 11 (Pt 11)]

COLA Chapter A [e.g., FSAR 01 to FSAR 19] {generally used only for Part 2}

Section / Page A [page numbers (if identified) are specific to document to be Revised, i.e., Rev 1

Change Summary [Short description of change...]

Basis for Change [the Source of the change...]

Attachment

Bellefonte Units 3 & 4 Subsequent Combined License Application

Update Roadmap

(This cover and Pages 1 to 76)

NuStart's COLA Tracking Management (CTM) : COLA Changes | **AP - BLN Submittal #5 ROADMAP January 2010**

JAN-12-2010
9:57 AM

AP - BLN Submittal #5 ROADMAP January 2010

Technology is not ESBWR AND ...

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
Pt 02						**344 COLA Changes**
5130	BLN,STD	Pt 02	FSAR 01	01.01.05	Remove first sentence that states "Reference 1 to DCD Chapter 1 discusses the generic construction plan and startup schedule for the AP1000."	WEC DCD Rev 17 conforming change
5883	BLN	Pt 02	FSAR 01	01.01.T / T1.1-201 SNM	COLA Part 2, FSAR Chapter 13, Section 1.1, Table 1.1-201, will be revised to add a new acronym of SNM for "Special Nuclear Material"	Conforming change associated with COL-SER-OI-Ch01 response to OI 01.05-01 item 1
4927	BLN,STD	Pt 02	FSAR 01	01.02.03	COLA Part 2, FSAR Chapter 1, Section 1.2.3 will be added with an LMA of BLN DEP 18.8-1, to read: 1.2.3 PLANT ARRANGEMENT DESCRIPTION ---------- Add the following information at the end of the first paragraph of DCD Subsection 1.2.3. Figure 1.2-201 replaces DCD Figure 1.2-18 to reflect the relocation of the Operations Support Center. ----------	Editorial
4834	BLN	Pt 02	FSAR 01	01.04	1. COLA Part 2, FSAR Chapter 1, Subsection 1.4.1 (BLN SUP 1.4-2) will be revised to read: Not all participants have been identified at this time. In particular, the AP1000 NSSS provider, architect-engineer, and constructor have not yet contracted. This section of the FSAR will be revised to include information identifying the NSSS provider, the architect-engineer, and the constructor following the establishment of contracts for these purposes. This information will include descriptions of the technical qualifications of the NSSS provider, the architect-engineer, and the constructor, and address the division of responsibility among them and the operator.	RAI LTR 148 response to RAI 01-13 item 1
5974	BLN	Pt 02	FSAR 01	01.06.T / T1.6-201	COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, will be revised to include the ADAMS number for the Westinghouse APP-GW-GL-700 from: ML TBD To read: ML083230868	Editorial
6070	BLN	Pt 02	FSAR 01	01.06.T / T1.6-201	1. COLA Part 2, FSAR Chapter 1, Table 1.6-201 as revised by VEGP R-COLA letter dated October 16, 2009, Update of NEI 07-03A References, will be revised.	COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 1 SNC Letter ND-09-1770
6085	BLN	Pt 02	FSAR 01	01.06.T / T1.6-201	COLA Part 2, FSAR Chapter 1, Table 1.6-201, title for NEI 07-08A References, will be revised from: "Generic FSAR Template Guidance for Ensuring Occupational Radiation Exposures Are As Low As Is Reasonably Achievable (ALARA)" To read: "Generic FSAR Template Guidance for Ensuring that Occupational Radiation Exposures Are As Low As Is Reasonably Achievable (ALARA)"	Editorial revision to COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 1 SNC Letter ND-09-1770

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
6420	BLN	Pt 02	FSAR 01	01.06.T / T1.6-201	COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, will be revised to include the ADAMS number for NEI 07-08A from: ML(tbd) To read: ML093220164	Editorial
5069	BLN,STD	Pt 02	FSAR 01	01.06.T / T1.6-201	1. COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, will be revised to read: NEI 06-13A(b) Template for an Industry Training 2 13.2 March 2009 ML090910554 Program Description	SUPERSEDED by Qb 5932 - BLN-VOL-LTR-004 response to NEI 06-13 item 1
5932	BLN	Pt 02	FSAR 01	01.06.T / T1.6-201	1. COLA Part 2, FSAR Chapter 1, Table 1.6-201 as revised by TVA R-COLA letter dated May 11, 2009, Update of NEI 06-13 References.	COL-SER-CI-Ch12 response to CI 12.01.01 item 1 SNC Letter #ND-09-1529
6477	BLN	Pt 02	FSAR 01	01.06.T / T1.6-201	COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, NEI 06-13A title, will be revised to read: Template for an Industry Training Program Description	Editorial
5072	BLN,STD	Pt 02	FSAR 01	01.06.T / T1.6-201	3. COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, footnote a, will be revised to read: a) NEI 07-02A Revision 0 includes the approved Revision 3 template, the NRC safety evaluation, and corresponding responses to the NRC Request for Additional Information. Only the approved template is incorporated by reference. The rest of the document is referenced but not incorporated into the FSAR.	SUPERSEDED by Qb 5933 - BLN-VOL-LTR-004 response to NEI 06-13 item 3
5070	BLN,STD	Pt 02	FSAR 01	01.06.T / T1.6-201	2. COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, will be revised to add new footnote b to read: b) NEI 06-13A Revision 2 includes the approved Revision 1 template, the NRC safety evaluation, and corresponding responses to the NRC Request for Additional Information. Only the approved template is incorporated by reference. The rest of the document is referenced but not incorporated into the FSAR.	SUPERSEDED by Qb 5933 - BLN-VOL-LTR-004 response to NEI 06-13 item 2
5933	BLN	Pt 02	FSAR 01	01.06.T / T1.6-201	2. COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, footnote a) as revised by, and footnote b) as added by, TVA R-COLA letter dated May 11, 2009, Update of NEI 06-13 References, will be revised to read (note that footnote b) is entirely deleted): a) The NRC-accepted NEI documents identified by the A in the document number include the accepted template, the NRC safety evaluation, and corresponding responses to the NRC Requests for Additional Information. Only the accepted template is incorporated by reference. The remainder of the document is referenced but not incorporated into the FSAR.	COL-SER-CI-Ch12 response to CI 12.01.01 item 2 SNC Letter #ND-09-1529
6071	BLN	Pt 02	FSAR 01	01.06.T / T1.6-201	2. COLA Part 2, FSAR Chapter 1, Table 1.6-201 footnote will be revised to read: (A) Denotes NRC approved document.	COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 2 SNC Letter ND-09-1770
5097	BLN	Pt 02	FSAR 01	01.08	1. COLA Part 2, FSAR Chapter 1, Section 1.8, will be revised to include the following new paragraph at the end of the section with a left margin annotation (LMA) of BLN SUP 1.8-3: DCD Table 1.8-1 presents interface items for the AP1000. FSAR section(s) addressing these interface items are tabulated in Table 1.8-203.	RAI LTR 156 response to RAI 01-014 item 1
5131	BLN	Pt 02	FSAR 01	01.08.T / T1.8-201	COLA Part 2, FSAR Chapter 1, Section 1.8, Table 1.8-201, will be revised to include the new BLN Departure added in response to BLN-RAI-LTR-129 via Supplement to read: BLN DEP 2.3-1 In Revision 17 of the DCD the EAB atmospheric dispersion coefficients were revised to values that	Conforming change based on BLN-RAI-LTR-129 Supp response

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					did not bound the BLN site-specific values. This departure provides the evaluation associated with the BLN site-specific parameter values. Table 1.9-202, 6.2.6 15.0 15A.3.1.3 Table 2.0-201 15.0.3.2 15A.3.3 Table 2.0-202 Table 15.0-201 Table 15A-201 6.2.5.1.2 15.6.5.3.1.2 Table 15A-202 6.5 15.6.5.3.7.3 16.1 Table 6.5-201 Table 15.6-201 TS 5.5.8 Table 14.3-202 Table 15.6-202 TS B3.6.1 TS B3.6.2	
4940	BLN,STD	Pt 02	FSAR 01	01.08.T / T1.8-202 19.59.10-4	1. COLA Part 2, FSAR Chapter 1, Table 1.8-202, COL ITEM 19.59.10-4 will be changed to read: Implement Severe Accident Management Guidance	RAI LTR 152 response to RAI 19-21 item 1
5284	BLN	Pt 02	FSAR 01	01.08.T / T1.8-203	2. COLA Part 2, FSAR Chapter 1, Section 1.8, will be revised to include the following new table with an LMA of BLN SUP 1.8-3: TABLE 1.8-203 SUMMARY OF FSAR DISCUSSIONS OF AP1000 PLANT INTERFACES	RAI LTR 156 response to RAI 01-014 item 2
5418	BLN	Pt 02	FSAR 01	01.08.T / T1.8-203 01.01	1. COLA Part 2, FSAR Chapter 1, Section 1.8, Table 1.8-203, will be revised to remove line item 1.1, Post accident Radio-Iodine sampling capability per NUREG 0737, and its associated footnote (2) which read: 1.1 Post accident Radio-Iodine sampling capability per NUREG 0737 Requirement of AP1000 Combined License applicant program (2) Note 2 – Westinghouse has determined that this item has been fully addressed by the DCD. Thus, item 1.1 is not addressed by the COLA.	COL-SER-OI-Ch01 response to OI 01.04-01 item 1
5648	BLN	Pt 02	FSAR 01	01.08.T / T1.8-203 03.03	COLA Part 2, FSAR Chapter 1, Table 1.8-203, item 3.3, will be revised to remove the following reference under the Section or Subsection column - DCD 3.7.4.2	Conforming change to reflect BLN COL-SER-OI-01 change 4
5419	BLN	Pt 02	FSAR 01	01.08.T / T1.8-203 03.06	2. COLA Part 2, FSAR Chapter 1, Section 1.8, Table 1.8-203, line item 3.6 will be revised from: 3.6 Specific depth of waterproofing Requirement of AP1000 Onsite implementation 2.5.4.1, DCD 3.4.1.1.1 To read: 3.6 Specific depth of waterproofing Requirement of AP1000 Onsite implementation 2.5.4.1	COL-SER-OI-Ch01 response to OI 01.04-01 item 2
5480	BLN	Pt 02	FSAR 01	01.08.T / T1.8-203 08.03	COLA Part 2, FSAR Chapter 1, Table 1.8-203, item 8.3, will be revised to add the following reference under the Section or Subsection column - 8.2.1.2.2	Conforming change to reflect RAI LTR 149 response to Verbal Request
5420	BLN	Pt 02	FSAR 01	01.08.T / T1.8-203 18.04	3. COLA Part 2, FSAR Chapter 1, Section 1.8, Table 1.8-203, will be revised to remove line items 18.4 and 18.5 related to human factors evaluations (see WEC response to DCD RAI-SRP18-COLP-20): 18.4 Final coordination and integration of human system interface areas within a AP1000 Combined License applicant 18.2,	COL-SER-OI-Ch01 response to OI 01.04-01 item 3

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					specific AP1000 consistent with Human Factors evaluations Interface program DCD 18.2, DCD 18.8 Final coordination and integration of Combined 18.5 Combined License applicant facilities AP1000 License applicant 18.2, with those of a specific AP1000 Interface program DCD 18.2, consistent with Human Factors DCD 18.8 evaluations	
5825	BLN	Pt 02	FSAR 01	01.09.01.01	1. Revise the following text in FSAR Subsection 1.9.1.1 to read: . . . One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design (see Notes at the end of Appendix 1AA). . . .	COL-SER-OI-Ch01 S1 response to OI 01.04-02 item 1
5826	BLN	Pt 02	FSAR 01	01.09.01.02	2. Revise the following text in FSAR Subsection 1.9.1.2 to read: . . . One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design (see Notes at the end of Appendix 1AA). . . .	COL-SER-OI-Ch01 S1 response to OI 01.04-02 item 2
5827	BLN	Pt 02	FSAR 01	01.09.01.03	3. Revise the following text in FSAR Subsection 1.9.1.3 to read: . . . One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design (see Notes at the end of Appendix 1AA). . . .	COL-SER-OI-Ch01 S1 response to OI 01.04-02 item 3
5828	BLN	Pt 02	FSAR 01	01.09.01.04	4. Revise the following text in FSAR Subsection 1.9.1.4 to read: . . . One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design (see Notes at the end of Appendix 1AA). . . .	COL-SER-OI-Ch01 S1 response to OI 01.04-02 item 4
4892	BLN,STD	Pt 02	FSAR 01	01.09.05.01.05	1. COLA Part 2, FSAR Chapter 1, will be revised to include the following new Subsection 1.9.5.1.5 (with an LMA of STD SUP 1.9-3): 1.9.5.1.5 Station Blackout Add the following text to the end of DCD Subsection 1.9.5.1.5. Training and procedures to mitigate a 10 CFR 50.63 "loss of all alternating current power" (or station blackout (SBO)) event are implemented in accordance with Sections 13.2 and 13.5, respectively. As recommended by NUMARC 87-00 (Reference 201), the SBO event mitigation procedures address response (e.g., restoration of onsite power sources), ac power restoration (e.g., coordination with transmission system load dispatcher), and severe weather guidance (e.g., identification of actions to prepare for the onset of severe weather such as an impending tornado), as applicable. The AP1000 is a passive design and does not rely on offsite or onsite ac sources of power for at least 72 hours after an SBO event, as described above. In addition, there are no nearby large power sources, such as a gas turbine or black start fossil fuel plant, that can directly connect to the station to mitigate the event. Restoration from an SBO event will be contingent upon ac power being made available from any one of the transmission lines described in Section 8.2 or any one of the standby diesel generators.	RAI LTR 025 S1 response to RAI 08.01-002 item 1 SER with Open Items Confirmatory Item 8.1-1
4893	BLN,STD	Pt 02	FSAR 01	01.09.06	2. COLA Part 2, FSAR Chapter 1, will be revised to include the following new Subsection 1.9.6 (with no LMA):	RAI LTR 025 S1 response to RAI 08.01-002 item 2 SER with Open Items

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					1.9.6 References Add the following text to the end of DCD Subsection 1.9.6. 201. NUMARC 87-00, Guidelines and Technical Bases for NUMARC Initiatives Addressing Station Blackout at Light Water Reactors, Revision 1, August 1991.	Confirmatory Item 8.1-1
5934	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 1.008	3. COLA Part 2, FSAR Chapter 1, Table 1.9-201, Regulatory Guide 1.8 will be revised to read: 1.8 Qualification and Training of Personnel for Nuclear Power Plants (Rev. 3, May 2000) — 12.1 (NEI 07-08) Appendix 12AA Appendix 12AA (NEI 07-03A) 13.1.1.4 13.1.3.1 13.2 (NEI 06-13A) 16 (TS 5.3.1)	COL-SER-CI-Ch12 response to CI 12.01.01 item 3 SNC Letter #ND-09-1529
6072	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 1.008	3. COLA Part 2, FSAR Chapter 1, Table 1.9-201, Regulatory Guide 1.8 will be revised from "12.1 (NEI 07-08)" to read "12.1 (NEI 07-08A)" in the FSAR Chapter, Section, or Subsection column.	COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 3 SNC Letter ND-09-1770
5640	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 1.016	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Revision 1, will be revised to remove RG 1.16 from the table.	This Regulatory Guide withdrawn by NRC on 8-11-2009 via 74 FR 40244.
2598	BLN,STD	Pt 02	FSAR 01	01.09.T / T1.9-201 1.068	2. COLA Part 2, FSAR. Chapter 1, Table 1.9-201 will be revised to include additional FSAR Chapter, Section, or Subsection references for RG 1.68 of 14.2.1, 14.2.3, 14.2.8, and 14.2.11.2, to go with the existing 16 (TS Bases 3.1.8)	SUPERSEDED by Qb 4855 - RAI LTR 139 response to RAI 14.02-012, item 2
4855	BLN,STD	Pt 02	FSAR 01	01.09.T / T1.9-201 1.068	6. COLA Part 2, FSAR. Chapter 1, Table 1.9-201, as shown in letter 139, will be revised to read: Regulatory Guides — FSAR Chapter, Section, or Subsectiona 1.68 Initial Test Program for Water-Cooled Nuclear Power Plants (Rev. 3, March 2007) — 14.2.1 14.2.3 14.2.5.2 14.2.8 16 (TS Bases 3.1.8)	RAI LTR 139 S1 response to RAI 14.02-012, item 6 SER with Open Items Confirmatory Item 14.2-1 & 2
6299	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 1.091	For RG 1.91, add cross reference to Table 19.58-201	Consistency
5897	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 1.097	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, will be revised for Regulatory Guide 1.97 (retaining STD LMA) from: 1.97 Criteria for Accident Monitoring Instrumentation For Nuclear Power Plants (Rev. 4, June 2006) — Appendix 12AA (NEI 07-03) 16 (TS bases 3.3.3) To read: 1.97 Criteria for Accident Monitoring Instrumentation For Nuclear Power Plants (Rev. 4, June 2006) — Not referenced; see Appendix 1AA 1.97 Instrumentation For Light-Water-Cooled Nuclear Power Plants to Assess Plant Environs Conditions — Table 7.5-201 Appendix 12AA 16 (TS Bases 3.3.3)	Consistency

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					During and Following an Accident (Rev. 3, May 1983)	
5935	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 1.097	4. COLA Part 2, FSAR Chapter 1, Table 1.9-201, Regulatory Guide 1.97, will be revised to read: 1.97 Criteria For Accident Monitoring Instrumentation For Nuclear Power Plants (Rev. 4, June 2006) Not referenced 1.97 Criteria For Accident Monitoring Instrumentation For Nuclear Power Plants (Rev. 3, May 1983) Appendix 12AA 16 (TS Bases 3.3.3)	SUPERSEDED by Qb 5897 - COL-SER-CI-Ch12 response to CI 12.01.01 item 4 SNC Letter #ND-09-1529
3474	BLN,STD	Pt 02	FSAR 01	01.09.T / T1.9-201 1.101	1. COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Revision 1, will be revised to address RG 1.101, Revisions 3, 4 and 5.	RAI LTR 142 response to RAI 01-11, item 1
5951	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 1.101	COLA Part 2, Chapter 1, Section 1.9, Table 1.9-201, RG 1.101, Revs. 5 and 4 from "Not referenced" to read "Not referenced; see Appendix 1AA"	Editorial
5890	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 1.133	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Revision 1, will be revised to change the FSAR Chapter, Section, or Subsection reference for RG 1.133 from "DCD discussion only; see DCD Table 1.9-1" to read "Not referenced; see Appendix 1AA"	Editorial
5953	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 1.135	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Revision 1, will be revised for RG 1.135 from: "DCD discussion only; see DCD Table 1.9-1" To read: "Not referenced; see Appendix 1AA"	This Regulatory Guide withdrawn by NRC on 8-6-2009 via 74 FR 39349.
5892	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 1.152	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Revision 1, will be revised to change the FSAR Chapter, Section, or Subsection reference for RG 1.152 from "DCD discussion only; see DCD Table 1.9-1" to read "Not referenced; see Appendix 1AA"	Editorial
5936	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 1.206	5. COLA Part 2, FSAR Chapter 1, Table 1.9-201, Regulatory Guide 1.206, FSAR crossreference column entry, will be revised to add "Appendix 12AA (NEI 07-03A)"	COL-SER-CI-Ch12 response to CI 12.01.01 item 5 SNC Letter #ND-09-1529
5482	BLN,STD	Pt 02	FSAR 01	01.09.T / T1.9-201 4.15	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Revision 1, will be revised to address RG 4.15, Revisions 1 and 2.	Editorial
5937	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 8	6. COLA Part 2, FSAR Chapter 1, Table 1.9-201, Division 8 Regulatory Guides will be revised.	COL-SER-CI-Ch12 response to CI 12.01.01 item 6 SNC Letter #ND-09-1529
6073	BLN	Pt 02	FSAR 01	01.09.T / T1.9-201 8	4. COLA Part 2, FSAR Chapter 1, Table 1.9-201, Regulatory Guides 8.2, 8.7, 8.8, 8.9, 8.10, 8.13, 8.15, 8.27, 8.28, 8.29, 8.34, 8.35, 8.36, and 8.38 will be revised from "12.1 (NEI 07-08)" to read "12.1 (NEI 07-08A)" in the FSAR Chapter, Section, or Subsection column.	COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 4 SNC Letter ND-09-1770
3480	BLN	Pt 02	FSAR 01	01.09.T / T1.9-202 Sh09	1. COLA Part 2, FSAR Chapter 1, Table 1.9-202, sheet 9 of 27, will be revised To read: 6.2.6 Containment Leakage Testing Exception See Notes d, e, and f. The Exception is taken to the guidance in SRP Section II, item 4 in that the proposed acceptable containment leakage rate is less than 0.10% of containment air by weight.	RAI LTR 129S response to RAI 15.00.03-001, item 1

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
5132	BLN	Pt 02	FSAR 01	01.09.T / T1.9-202 Sh09	Revise the change in Qb 3480 to have an LMA of BLN SUP 1.9-4	Conforming change to match RAI LTR 129S response to RAI 15.00.03-001, item 1
5364	BLN	Pt 02	FSAR 01	01.09.T / T1.9-202 Sh09	Revise date of SRP 6.5.4 to read "12/1988"	Editorial
5408	BLN	Pt 02	FSAR 01	01.09.T / T1.9-202 Sh25	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-202, will be revised to add separator bars around the conformance listing for 17.1 Quality Assurance During the Design and Construction Phases, Rev. 2, 07/1981, and an LMA added to the item that reads "BLN SUP 1.9-5"	Editorial
5311	BLN	Pt 02	FSAR 01	01.10.03	COLA Part 2, FSAR Chapter 1, Subsection 1.10.3 will be revised to add the following new paragraph at the end of the subsection (under the LMA of STD SUP 1.10-1): The above discussed controls to eliminate or mitigate construction hazards that could potentially impact operating unit SSCs important to safety are in place when there is an operating nuclear unit on the site.	COL-SER-OI-Ch01 response to OI 01.04-04
4897	BLN,STD	Pt 02	FSAR 01	01.B	COLA Part 2, FSAR Chapter 1, Appendix 1B, will be revised To read: Rather, the severe accident mitigation design alternatives are addressed in the Environmental Report.	Editorial
5642	BLN	Pt 02	FSAR 01	01AA RG 1.016	COLA Part 2, FSAR Chapter 1, Appendix 1AA, Revision 1, conformance statement for Regulatory Guide 1.16 will be deleted in its entirety.	This Regulatory Guide withdrawn by NRC on 8-11-2009 vai 74 FR 40244.
3475	BLN,STD	Pt 02	FSAR 01	01AA RG 1.033	2. COLA Part 2, FSAR Chapter 1, Appendix 1AA, Revision 1, conformance statement for Regulatory Guide 1.33 will be revised to include a reference to NEI 06-14A in the exception statement.	RAI LTR 142 response to RAI 01-11, item 2
3842	BLN,STD	Pt 02	FSAR 01	01AA RG 1.053	COLA Part 2, FSAR Chapter 1, Appendix 1AA, Revision 1, title for Regulatory Guide 1.53 will be revised to replace "Nuclear Power Plan Protection" with "Safety"	Editorial
5894	BLN	Pt 02	FSAR 01	01AA RG 1.097	COLA Part 2, FSAR Chapter 1, Appendix 1AA, conformance statement for Regulatory Guide 1.97 will be revised from: "This guidance is completely within the scope fo the DCD." To read: "Conformance with this Regulatory Guide for programmatic and/or operational aspects is documented below. General Exception Portable equipment outside the DCD scope conforms to Revision 3 of this Regulatory Guide for consistency with DCD scope since Revision 4 indicates that partial implementation is not advised.	FSAR Revision 1 included conformance (See Table 7.5-201) for equipment outside the DCD scope.
3476	BLN,STD	Pt 02	FSAR 01	01AA RG 1.101	3. COLA Part 2, FSAR Chapter 1, Appendix 1AA, Revision 1, conformance statement for Regulatory Guide 1.101 will be revised to address Revisions 3, 4, and 5.	RAI LTR 142 response to RAI 01-11, item 3
4871	BLN,STD	Pt 02	FSAR 01	01AA RG 1.133	Revision items 1 through 5 were provided with the original response dated January 27, 2009, and will be incorporated into the COL application in a future revision. 6. COLA Part 2, FSAR Chapter 1, Appendix 1AA, conformance statement for Regulatory Guide 1.133 will be revised to read: C.2b Conforms Procedures are addressed in Section 13.5 C.3a Conforms Procedures are addressed in Section 13.5 C.4g Conforms Procedures are addressed in Section 13.5 C.4h Conforms Procedures are addressed in Section 13.5	RAI LTR 142 S1 response to RAI 01-011, item 6 SER with Open Items Confirmatory Item 4.4-1

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					C.4i Conforms ALARA is addressed in Chapter 12 and Section 13.5 C.4j Conforms Training is addressed in Section 13.2 C.6 Exception See position for Regulatory Guide 1.16	
5957	BLN	Pt 02	FSAR 01	01AA RG 1.133	COLA Part 2, FSAR Chapter 1, Appendix 1AA, conformance statement C.6 for Regulatory Guide 1.133 will be revised to read: C.6 Exception Regulatory Guide 1.16 has been withdrawn	Regulatory Guide 1.16 withdrawn by NRC on 8-11-2009 via 74 FR 40244. Revises BLN RAI LTR 142 S1 response to RAI 01-011, item 6 SER with Open Items Confirmatory Item 4.4-1
5955	BLN	Pt 02	FSAR 01	01AA RG 1.135	COLA Part 2, FSAR Chapter 1, Appendix 1AA, conformance statement for Regulatory Guide 1.135 will be revised To read: Conformance of the design aspects is as stated in the DCD. The programmatic and/or operational aspects are not applicable since this guidance was withdrawn by NRC (74 FR 39349, 08/06/2009).	This RG withdrawn by NRC (See 74 FR 39349, 08/06/2009).
4995	BLN,STD	Pt 02	FSAR 01	01AA RG 1.152	COLA Part 2, FSAR Chapter 1, Appendix 1AA, Revision 1, conformance statement for Regulatory Guide 1.152 will be revised to reflect the latest revisions to the Security regulations To read: General Exception The Cyber Security Program is based on March 2009 revisions of the 10 CFR 73.54 regulations in lieu of Revision 2 of this Regulatory Guide.	Editorial
4824	BLN,STD	Pt 02	FSAR 01	01AA RG 1.180	COLA Part 2, FSAR Chapter 1, Appendix 1AA, Revision 1, conformance statement for Regulatory Guide 1.180 will be revised to reflect that the DCD addresses the design aspects of Revision 1 of the RG.	WEC DCD Rev 17 conformance change
3477	BLN,STD	Pt 02	FSAR 01	01AA RG 8.006	4. COLA Part 2, FSAR Chapter 1, Appendix 1AA, Revision 1, conformance statement for Regulatory Guide 8.6 will be revised to inlcude justification for the identified exception.	RAI LTR 142 response to RAI 01-11, item 4
5829	BLN	Pt 02	FSAR 01	01AA Note	5. Revise FSAR Appendix 1AA Note (at the end of the Appendix) to read (the # may vary for R-COLA and S-COLA as appropriate): Note #. Above stated general alternatives regarding the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD is provided to preserve the finality of the certified design. Further, each stated conformance with the programmatic and/or operational aspects is only to the extent that a design change or departure from the approved DCD is not required to implement those programmatic and/or operational aspects. As the operational and programmatic aspects become more fully defined (for example, during the preparation, approval, or initial implementation of plant procedures), there exists a potential that a conflict could be identified between the design as certified in the DCD and the programmatic and/or operational aspects of the guidance. In such cases, the design certification (rule) becomes the controlling factor, and the design conformance to the Regulatory Guide is per the revision stated in the DCD.	COL-SER-OI-Ch01 S1 response to OI 01.04-02 item 5
5830	BLN	Pt 02	FSAR 01	01AA Note	7. Revise FSAR Appendix 1AA Note (at the end of the Appendix) to include the following additional note (the # may vary for the R-COLA and S-COLA as appropriate): Note #. A "Criteria Section" entry of "General" indicates a scope for the conformance statement of "all regulatory guide positions related to programmatic and/or operational aspects." Thus, an associated conformance statement of "Conforms" indicates that the applicant "complies with all regulatory guide positions related to programmatic and/or operational aspects."	COL-SER-OI-Ch01 S1 response to OI 01.04-02 item 7
5148	BLN	Pt 02	FSAR 02	02.00 LOT	COLA Part 2, FSAR Chapter 2, List of Tables for Table 2.3-328 to separate "atReceptor" and insert "Each" to read "...at Each Receptor Location"	Editorial
2222	BLN	Pt 02	FSAR 02	02.00.T / T2.0-201 Sh01	COLA Part 2, FSAR Chapter 2, Table 2.0-201, Air Temperature entries for the Site Characteristic will be revised.	SUPERSEDED by Qb 2602 - RAI LTR 133 response to

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
						RAI 02.03.01-12, item 1
2602	BLN	Pt 02	FSAR 02	02.00.T / T2.0-201 Sh01	1. COLA Part 2, FSAR Chapter 2, Table 2.0-201, Air Temperature entries for the Site Characteristic will be revised To read: Maximum Safety(b) 107.7°F dry bulb / 79.6°F coincident wet bulb (100-year return) 83.5°F wet bulb (noncoincident) (100-year return) Minimum Safety (b) -12.2°F (100-year return) Maximum Normal(c) 94°F dry bulb / 75°F coincident wet bulb (1% seasonal exceedance) 78°F wet bulb (noncoincident) (1% seasonal exceedance) Minimum Normal(c) 20°F (99% seasonal exceedance) c) Maximum and minimum normal values are the ASHRAE equivalent 1 percent seasonal exceedance magnitudes.	RAI LTR 133S response to RAI 02.03.01-012, item 1 Note c) change SUPERSEDED by Qb 5478.
5133	BLN	Pt 02	FSAR 02	02.00.T / T2.0-201 Sh06	COLA Part 2, FSAR Chapter 2, Table 2.0-201, BLN Site Characteristic value for the Rain parameter is revised to remove the 5-min criteria from 17.6 in/hr (3.3 in/5 min) to read 17.6 in/hr	Editorial
3481	BLN	Pt 02	FSAR 02	02.00.T / T2.0-201 Sh06	2. COLA Part 2, FSAR Chapter 2, Table 2.0-201, sheet 6 of 7, will be revised To read: AP1000 DCD BLN Site BLN FSAR BLN Within Site Site Parameter(a) Characteristic Reference Parameter Atmospheric Dispersion Values - X/Q(f) Site Boundary (0-2 hr) <=5.1 x 10-4 sec/m3 5.85 x 10-4 sec/m3 Table 2.3-319 No(j)	RAI LTR 129S response to RAI 15.00.03-001, item 2
5323	BLN	Pt 02	FSAR 02	02.00.T / T2.0-201 Sh06	1. COLA Part 2, FSAR Chapter 2, Section 2.0, Table 2.0-201, BLN Site Characteristic for the Atmospheric Dispersion Value for Site Boundary (annual average) is revised from: 0.28 x 10-5 sec/m3 To read: 0.14 x 10-5 sec/m3	BLN-VOL-LTR 007 item 1
5478	BLN	Pt 02	FSAR 02	02.00.T / T2.0-201 Sh07	COLA Part 2, FSAR Chapter 2, Table 2.0-201, revision to Note c) identified in RAI LTR 133S response to RAI 02.03.01-012, item 1 (Qb 2602), was not correctly based on Rev 1 and is superseded by the Rev 1 change of Qb 2078. Note c) should continue to read as it does in January 2009 revision.	SUPERSEDES part of Qb 2602 from RAI LTR 133S response to RAI 02.03.01-012, item 1
3482	BLN	Pt 02	FSAR 02	02.00.T / T2.0-201 Sh07	3. COLA Part 2, FSAR Chapter 2, Table 2.0-201, sheet 7 of 7, will be revised To read: j) This Site Characteristic evaluated on a site specific basis. See Section 15.0, Subsection 15.6.5, and Appendix 15A, Subsection 15A.3.3.	RAI LTR 129S response to RAI 15.00.03-001, item 3
5324	BLN	Pt 02	FSAR 02	02.00.T / T2.0-202	2. COLA Part 2, FSAR Chapter 2, Section 2.0, Table 2.0-202 values for the FSAR X/Q's will be revised from the current Revision 1 to read as shown in Attachment 1. The column heading for the "Fuel Building Rail Bay Door" on sheet 3 of 4 is also revised to read "Radwaste Building Truck Staging Area Door." The Notes are not revised and remain as shown in Revision 1 of the FSAR.	BLN-VOL-LTR 007 item 2

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5147	BLN	Pt 02	FSAR 02	02.01.01.02.01	COLA Part 2, FSAR Chapter 2, Section 2.1.1.2.1 is renumbered to 2.1.1.3 and the Table of Contents for 2.1.1.2.1 is revised to 2.1.1.3 to match the renumbering.	Editorial
5149	BLN	Pt 02	FSAR 02	02.01.03.03.02.03	COLA Part 2, FSAR Chapter 2, Subsection 2.1.3.3.2.3 is revised in the last sentence to remove the duplicate "of these workers"	Editorial
3834	BLN	Pt 02	FSAR 02	02.02.02.02.09	1. COLA Part 2, FSAR Chapter 2, Subsection 2.2.2.2.9 will be revised To read: Norfolk Southern Railroad Company (NSRC) owns and operates a railroad line that runs through the city of Scottsboro, Alabama, and the town of Hollywood, Alabama, approximately 3 mi. northwest of the site. Any material registered with the federal government as a hazardous material that is legally allowed to be transported via American railroads could potentially be transported at some point along the rails that are situated near the BLN site. Items that may be legally transported on the rails near the site include many types of hazardous materials and other industrial chemicals. Table 2.2-208 lists the top 50 commodities shipped through Hollywood, Alabama, between June 30, 2006 and June 30, 2007. NIOSH IDLH for the reported materials are provided in Tables 2.2-205, 2.2-206, and 2.2-207 (Reference 219).	RAI LTR 132 S1 response to RAI 02.02.03-008, item 1
3835	BLN	Pt 02	FSAR 02	02.02.02.06	2. COLA Part 2, FSAR Chapter 2, Subsection 2.2.2.6 will be revised To read: NSRC owns and operates a railroad line that runs through the city of Scottsboro, Alabama, and the town of Hollywood, Alabama. This railroad line is the main line in northern Alabama running from Memphis, Tennessee, through Huntsville, Alabama, to Chattanooga, Tennessee (Reference 201). At its closest point, the line runs about 3 mi. northwest of the BLN site center point. On average, 40 trains per day pulling an average of 75 cars use this rail line and travel at speeds up to 50 mph. This line is used for freight service only; no passenger trains use this line (Reference 222). As stated in Subsection 2.2.2.2.9, any material registered with the federal government as a hazardous material that is legally allowed to be transported via American railroads could potentially be transported at some point along the rails that are situated near the BLN site. Items that may be legally transported on the rails near the site include many types of hazardous materials and other industrial chemicals. Table 2.2-208 lists the top 50 commodities shipped through Hollywood, Alabama, between June 30, 2006 and June 30, 2007. NIOSH IDLH for the reported materials are provided in Tables 2.2- 205, 2.2-206, and 2.2-207 (Reference 219).	RAI LTR 132 S1 response to RAI 02.02.03-008, item 2
6000	BLN	Pt 02	FSAR 02	02.02.03.01.01	COLA Part 2, FSAR Chapter 2, Subsection 2.2.3.1.1, will be revised to relocate the following paragraphs from their current location: Eight paragraphs under the heading Spill Frequency on the Tennessee and Associated Rivers, and eight paragraphs under the heading Quantified Risk of Detonation. To follow the paragraph shown below. The nearest transportation route to the BLN is the Guntersville Reservoir. Its nearest bank is located 0.65 miles from the site. An assessment was performed to evaluate potential hazards represented by flammable and explosive cargo transported via barge past the BLN on the Guntersville Reservoir. An initial screening of commodities included in cargo shipped via the Guntersville Reservoir past the BLN site was conducted to identify those materials that warranted more detailed evaluation, that is, "commodities of interest." This initial screening of the hazardous commodities eliminated all but two requiring further analysis for potential adverse impact to the BLN site from waterway transportation (barge) accidents. These two commodities are styrene and ethanol. Commodities are screened out based on their physical properties. The primary physical parameter is the commodities' flash point. The National Fire Protection Association Hazard Identification System (NFPA 704) (Reference 237) is used. Only commodities with flammability hazards classified as three or four (serious hazard and severe hazard, respectively) are considered.	COL-SER-OI-Ch02 response to OI 02.02.03-001
5151	BLN	Pt 02	FSAR 02	02.02.03.01.01.03	COLA Part 2, FSAR Chapter 2, Subsection 2.2.3.1.1.3 is revised in the first sentence to remove the	Editorial

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					extra period after "BLN site" and before "boundary."	
3970	BLN	Pt 02	FSAR 02	02.02.03.01.01.04	1. COLA Part 2, FSAR Chapter 2, Subsection 2.2.3.1.1.4, will be revised to read: "… The plant gas system provides hydrogen, carbon dioxide, and nitrogen gases to the plant systems as required. The effects of the plant gas system on main control room habitability are addressed in DCD Section 6.4 including explosive gases and burn conditions for those gases. For explosions, the plant gas system is designed for conformance with Regulatory Guide 1.91 (DCD Subsection 9.3.2.3). There are no solid material explosion, confined, unconfined vapor explosion, toxic gas release event hazards identified for the Bellefonte nuclear site from hazardous chemicals that are outside the scope of the DCD identified in the Table 6.4-202."	RAI LTR 137 response to 02.02.03-10, item 1
3836	BLN	Pt 02	FSAR 02	02.02.03.01.03	3. COLA Part 2, FSAR Chapter 2, Subsection 2.2.3.1.3 will be extensively revised.	RAI LTR 132 S1 response to RAI 02.02.03-008, item 3
5374	BLN	Pt 02	FSAR 02	02.02.03.01.03.02.01	COLA Part 2, FSAR Chapter 2, Subsection 2.2.3.1.3.2.1 will be revised To read: The quantity of toxic materials used at Maple Industries is...	Editorial
3837	BLN	Pt 02	FSAR 02	02.02.03.01.03.02.02.01	4. COLA Part 2, FSAR Chapter 2, Subsection 2.2.3.1.3.2.2.1 will be revised To read: Barge shipment frequency statistics on barge traffic for 2004 were provided by the US Army Corp of Engineers Waterborne Commerce Statistics Center. The chemical screening methodology described in Regulatory Guide 1.78 is applied to the commodities transported by barge traffic. As a result of the screening, all chemicals except ethyl alcohol screen from further review as potential hazards to control room personnel. The mass utilized for the hazardous release calculation is 5,140,000 lbs (2,331,465 kg). The screening factors of distance from source to control room HVAC intake, meteorological conditions, total mass of the release, control room model characteristics, and IDLH values of the hazardous material do not eliminate ethyl alcohol from the potential to exceed the NIOSH IDLH threshold value.	RAI LTR 132 S1 response to RAI 02.02.03-008, item 4
3838	BLN	Pt 02	FSAR 02	02.02.03.01.03.02.02.02	5. COLA Part 2, FSAR Chapter 2, Section 2.2.3.1.3.2.2.2 will be revised To read: State Highway 72 is located approximately 1.5 miles west of the BLN site. Available State Highway 72 commodity flow information and rural highway risk analysis are used to perform a bounding analysis of traffic on Highway 72. The traffic is analyzed in accordance with the methodology in Regulatory Guide 1.78. Chemicals, with the exception of nitrogen, are screened from further review. Nitrogen is an asphyxiating gas and is identified as a potential hazard to control room personnel. The mass utilized for the hazardous release calculation is 58,500 lbs. The screening factors utilized include distance from source to control room HVAC intake, meteorological conditions, total mass of the release, control room model characteristics, and IDLH values of the hazardous material.	RAI LTR 132 S1 response to RAI 02.02.03-008, item 5
5975	BLN	Pt 02	FSAR 02	02.02.03.01.03.02.02.02	COLA Part 2, FSAR Chapter 2, Subsection 2.2.3.1.3.2.2.2, Local Highways, is revised To read: "State Highway 72 is located approximately 1.5 miles northwest of the BLN site."	Editorial
3839	BLN	Pt 02	FSAR 02	02.02.03.01.03.02.02.03	6. COLA Part 2, FSAR Chapter 2, Section 2.2.3.1.3.2.2.3 will be revised To read: A Norfolk Southern rail line is located approximately 2.5 miles west of the BLN site, running northeast to southwest in a line parallel to the Tennessee River. The screening method described in Regulatory Guide 1.78 is applied to the rail traffic listed in Section 2.2 and in Table 2.2-208. The release mass of a toxic chemical is calculated based on the size of a commercially available rail tanker. The screening factors of distance from source to control room HVAC intake, meteorological conditions, total mass of the release, control room model characteristics, and IDLH values of the hazardous material do not eliminate chlorine, anhydrous ammonia, propylene oxide, and hydrogen fluoride from the potential to exceed the NIOSH IDLH threshold value.	RAI LTR 132 S1 response to RAI 02.02.03-008, item 6
3840	BLN	Pt 02	FSAR 02	02.02.03.01.03.03	7. COLA Part 2, FSAR Chapter 2, Section 2.2.3.1.3.3 will be revised To read: As indicated above, the identified stationary industrial sources and mobile sources within the	RAI LTR 132 S1 response to RAI 02.02.03-008, item

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					proximity of the BLN site are screened for potential of a release and the potential impact upon control room habitability. The chemicals screened out with the exception of those identified above. Thus, ethyl alcohol for barge traffic, nitrogen for the truck traffic, and the chlorine, anhydrous ammonia, propylene oxide, and hydrogen fluoride railroad tanker traffic release events are further evaluated in Section 6.4 to determine control room habitability.	7
3841	BLN	Pt 02	FSAR 02	02.02.T / T2.2-208	8. COLA Part 2, FSAR Chapter 2, Section 2.2.3 Table 2.2-208 will be revised in its entirety to read: Table 2.2-208: TOP 50 COMMODITIES SHIPPED VIA NSRC RAILROAD PAST HOLLYWOOD, AL, JUNE 2006 – JUNE 2007. Security-Related Information — Withheld Under 10 CFR 2.390(d) (See Part 9 of this COL Application)	RAI LTR 132 S1 response to RAI 02.02.03-008, item 8
3854	BLN	Pt 02	FSAR 02	02.02.T / T2.2-215	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-215 (both pages) will be revised to include the LMA of BLN COL 2.2-1	Editorial
3855	BLN	Pt 02	FSAR 02	02.02.T / T2.2-216	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-216 (all pages) will be revised to include the LMA of BLN COL 2.2-1	Editorial
3856	BLN	Pt 02	FSAR 02	02.02.T / T2.2-217	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-217 (all pages) will be revised to include the LMA of BLN COL 2.2-1	Editorial
3857	BLN	Pt 02	FSAR 02	02.02.T / T2.2-220	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-220 will be revised to include the LMA of BLN COL 2.2-1	Editorial
3858	BLN	Pt 02	FSAR 02	02.02.T / T2.2-221	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-221 will be revised to include the LMA of BLN COL 2.2-1	Editorial
3859	BLN	Pt 02	FSAR 02	02.02.T / T2.2-222	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-222 will be revised to include the LMA of BLN COL 2.2-1	Editorial
3860	BLN	Pt 02	FSAR 02	02.02.T / T2.2-223	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-223 will be revised to include the LMA of BLN COL 2.2-1	Editorial
5375	BLN	Pt 02	FSAR 02	02.03.01.02.01	COLA Part 2, FSAR Chapter 2, Subsection 2.3.1.2.1 will be revised To read: ...the probability of freezing rain (glaze ice) with a thickness of 15 mm (0.59 in) at the BLN site in any year is two percent."	Editorial
5376	BLN	Pt 02	FSAR 02	02.03.01.02.01.04	COLA Part 2, FSAR Chapter 2, Subsection 2.3.1.2.1.4 will be revised To read: Recent studies based on data from the NLDN (Reference 221) indicate that the above strike densities are upper bound for the BLN site.	Editorial
5659	BLN	Pt 02	FSAR 02	02.03.01.02.01.05	COLA Part 2, FSAR Chapter 2, Subsection 2.3.1.2.1.5 will be revised from: "Marshall county" and "Franklin county" To read: "Marshall County" and "Franklin County"	Editorial
5574	BLN	Pt 02	FSAR 02	02.03.01.02.01.06	6. COLA Part 2, FSAR, Chapter 2, Subsection 2.3.1.2.1.6, will be revised to read: The ventilation rates shown in Table 2.3-211 are discussed in Subsection 2.3.2.1.5.	RAI LTR 77 S3 response to RAI 02.03.02-003
2225	BLN	Pt 02	FSAR 02	02.03.01.03	3. COLA Part 2, FSAR Chapter 2, Subsection 2.3.1.3.1 will be revised.	SUPERSEDED by Qb 2604 - RAI LTR 133 response to RAI 02.03.01-012 item 3
2604	BLN	Pt 02	FSAR 02	02.03.01.03	3. COLA Part 2, FSAR Chapter 2, Subsection 2.3.1.3.1 will be revised	RAI LTR 133S response to RAI 02.03.01-012, item 3
2580	BLN	Pt 02	FSAR 02	02.03.01.03	COLA Part 2, FSAR Chapter 2, Subsection 2.3.1.3.1 will be revised to add the following as the final	RAI LTR 088S response to

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					two paragraphs of this subsection: Dry-bulb, coincident wet-bulb, and non-coincident wet-bulb temperatures represent significant site characteristics because this data is used in demonstrating that the BLN site characteristics are bounded by the AP1000 DCD site parameters. The BLN site characteristic temperatures were developed by considering both 100-year return temperatures and 0% exceedance temperatures. These values were calculated using a 35-year sequential hourly meteorological data set for Huntsville, AL, National Weather Service (NWS) station. The difference between the BLN site characteristics and the DCD site parameters, used for design, provides additional margin. General predictions on global or U.S. climatic changes expected during the period of reactor operation are uncertain and are only applicable on a macroclimatic scale. Since the maximum data span available (that is representative of the microclimate near the BLN site) was used in the severe weather analysis, accurate severe weather phenomena projections have been provided based on historic data. Projection of future climatological conditions at the BLN site are speculative at best, based on current understanding and modeling of global climate change.	RAI 02.03.01-009
4644	BLN	Pt 02	FSAR 02	02.03.02.01.03.01.01	COLA Part 2, FSAR Chapter 2, Subsection 2.3.2.1.3.1.1 will be revised to read: Average precipitation at the BLN site during 2006-2007 reached a maximum monthly mean in October (5.1 inches) and a minimum monthly mean in September (0.1 inches). The maximum monthly precipitation at the BLN site during 2006-2007 is 5.1 inches (Table 2.3-307).	TVA LTR dated 2/18/2009 in response to TVA/NRC telecom of 2/10/09.
5660	BLN	Pt 02	FSAR 02	02.03.02.01.03.03	COLA Part 2, FSAR Chapter 2, Subsection 2.3.2.1.3.3 will be revised from: "Winds speeds..." To read: "Wind speeds..."	Editorial
4643	BLN	Pt 02	FSAR 02	02.03.02.02.01	COLA Part 2, FSAR Chapter 2, Subsection 2.3.2.2.1 will be revised to include the following new final paragraph: The SACTI quantitative analysis determined that the towers do not deposit entrained moisture or salts within the first 6600 feet (~1.3 miles) due to high elevation of the discharge of the NDCTs. This distance is well beyond the plants electrical substation and onsite transmission path. Most deposition from the BLN towers occurs to the S to SSW or NNE due to channeling of winds by the river valley (away from electrical equipment). The transmission lines are located northwest of the towers and enter the plant from the west and southwest. The substation is located almost due north of the towers and is outside the zone of influence of the plume.	RAI LTR 077 S2 response to RAI 02.03.02-004
5047	BLN	Pt 02	FSAR 02	02.03.03.02.01	3. COLA Part 2, FSAR Chapter 2, Subsection 2.3.3.2.1, will be revised to add the following after the first paragraph: The vertical temperature difference reading is obtained from the ambient temperature sensors installed at 10 and 55 meters.	RAI LTR 089 S2 response to 07.05-01 item 3
2548	BLN	Pt 02	FSAR 02	02.03.03.02.02.01	2. COL Part 2, FSAR Subsection 2.3.3.2.2.1, Data Acquisition, will be revised in the paragraph "Meteorological sensor outputs are sampled... ...is considered invalid and treated as missing. To read: Meteorological sensor outputs are sampled at the following rates: horizontal wind direction and wind speed, every five seconds (720 per hour); temperature and dewpoint, every minute (60 per hour); rainfall, every 15 minutes (4 per hour). The temperature and dewpoint sampling interval will be changed to a five second sampling rate at least one year before fuel load. Each piece of data is checked to verify that it is between the minimum and maximum instrument limits. Data outside of specified limits is considered invalid and treated as missing.	RAI LTR 96S2 response to RAI 02.03.03-03 item 2
6001	BLN	Pt 02	FSAR 02	02.03.04	COLA Part 2, FSAR Chapter 2, Section 2.3.4, will be revised to add new Subsection 2.3.4.4 "Technical Support Center Atmospheric Dispersion Factors" following the existing Subsection 2.3.4.3.	COL-SER-OI-Ch02 response to OI 02.03.04-

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
						002
2226	BLN	Pt 02	FSAR 02	02.03.07 REF 238-239	4. COLA Part 2, FSAR Chapter 2, Subsection 2.3.7, "References," will be revised to include: 238. ASHRAE Handbook-Fundamentals, Atlanta: American Society of Heating, Refrigerating and Air-Conditioning Engineers, Inc. (ASHRAE), 2001. 239. ASHRAE Handbook-Fundamentals, Atlanta: American Society of Heating, Refrigerating and Air-Conditioning Engineers, Inc. (ASHRAE), 1997.	SUPERSEDED by Qb 2605 - RAI LTR 133 response to RAI 02.03.01-012 item 4
2605	BLN	Pt 02	FSAR 02	02.03.07 REF 238-239	4. COLA Part 2, FSAR Chapter 2, Subsection 2.3.7, "References," will be revised to include: 238. ASHRAE Handbook-Fundamentals, Atlanta: American Society of Heating, Refrigerating and Air-Conditioning Engineers, Inc. (ASHRAE), 2001. 239. ASHRAE Handbook-Fundamentals, Atlanta: American Society of Heating, Refrigerating and Air-Conditioning Engineers, Inc. (ASHRAE), 1997.	RAI LTR 133S response to RAI 02.03.01-012, item 4
2224	BLN	Pt 02	FSAR 02	02.03.T / T2.3-203 Sh02	2. COLA Part 2, FSAR Chapter 2, Table 2.3-203, Sheet 2 of 2, BLN Site Characteristics for temperatures is revised per RAI response.	SUPERSEDED by Qb 2603 - RAI LTR 133 response to RAI 02.03.01-012 item 2
2603	BLN	Pt 02	FSAR 02	02.03.T / T2.3-203 Sh02	2. COLA Part 2, FSAR Chapter 2, Table 2.3-203, Sheet 2 of 2, BLN Site Characteristics for temperatures is revised per RAI response.	RAI LTR 133S response to RAI 02.03.01-012, item 2
5661	BLN	Pt 02	FSAR 02	02.03.T / T2.3-209	COLA Part 2, FSAR Chapter 2, Table 2.3-209 will be revised to add an LMA of BLN COL 2.3-1 and to add a closing parenthesis on the Reference 234 at the bottom of the table.	Editorial
5046	BLN	Pt 02	FSAR 02	02.03.T / T2.3-317	2. COLA Part 2, FSAR Chapter 2, Table 2.3-317 will be revised to read: Wind Direction (WD) and Wind Speed (WS) 10, 55 Ambient Air Temperature 10, 55	RAI LTR 089 S2 response to 07.05-01 item 2
5045	BLN	Pt 02	FSAR 02	02.03.T / T2.3-317 Sh 2 of 2	1. COLA Part 2, FSAR Chapter 2, Table 2.3-317, Sheet 2 of 2, will be revised to insert the following after the "Ambient Air Temperature" entry: Differential Air Temperature (calculated or displayed) Based on vertical temperature difference from ambient air temperature sensors at 10 and 55 meters. Range -9º F to 18º F and accuracy of ±0.27º F.	RAI LTR 089 S2 response to 07.05-01 item 1
5325	BLN	Pt 02	FSAR 02	02.03.T / T2.3-321	3. COLA Part 2, FSAR Chapter 2, Section 2.3, Table 2.3-321 values for the X/Q's will be revised from the current Revision 1 to read as shown in Attachment 2. The column headings for the "Fuel Building Rail Bay Door" on sheets 1 and 2 are also revised to read "Radwaste Building Truck Staging Area Door." The Notes are not revised and remain as shown in Revision 1 of the FSAR	BLN-VOL-LTR 007 item 3
5380	BLN	Pt 02	FSAR 02	02.03.T / T2.3-324	COLA Part 2, FSAR. Chapter 2, will be revised to delete FSAR Revision 1 Table 2.3-324, "ANNUAL AVERAGE X/Q (SEC/M3) FOR NO DECAY, DEPLETED FOR EACH 22.5° SECTOR AT THE DISTANCES (MILES) SHOWN AT THE TOP."	RAI LTR 140 S1 response to RAI 02.03.05-006
2549	BLN	Pt 02	FSAR 02	02.03.T / T2.3-325	COLA Part 2, FSAR. Chapter 2, will be revised to delete FSAR Table 2.3-325, "ANNUAL AVERAGE [Chi]/Q VALUES FOR NO DECAY, DEPLETED."	SUPERSEDED by Qb 5380 - RAI LTR 140 response to RAI 02.03.05-006
5662	BLN	Pt 02	FSAR 02	02.03.T / T2.3-332 thru 338	COLA Part 2, FSAR Chapter 2, Tables 2.3-332 through 2.3-338 (all pages) will be revised to add an LMA of BLN COL 2.3-2.	Editorial
5927	BLN	Pt 02	FSAR 02	02.04.01.02.03.04	COLA Part 2, FSAR Chapter 2, Subsection 2.4.1.2.3.4 will be revised from: "Nickajack Dam controls a drainage area of 21,870 sq. mi. with a maximum dam discharge rate of 1500,000 cfs." To read:	Editorial

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					"Nickajack Dam controls a drainage area of 21,870 sq. mi. with a maximum dam discharge rate of 500,000 cfs."	
5663	BLN	Pt 02	FSAR 02	02.04.01.02.04	COLA Part 2, FSAR Chapter 2, Subsection 2.4.1.2.4 will be revised from: "The largest water user is TVA's Widows Creek Fossil Plant, which which withdraws and discharges approximately 1500 Mgd for thermoelectric power generation." To read: "The largest water user is TVA's Widows Creek Fossil Plant, which withdraws and discharges approximately 1500 Mgd for thermoelectric power generation."	Editorial
5664	BLN	Pt 02	FSAR 02	02.04.02.01	COLA Part 2, FSAR Chapter 2, Subsection 2.4.2.1 will be revised To read: "These mainstream reservoirs, however, play an essential part in reducing the flood crest..."	Editorial
2189	BLN	Pt 02	FSAR 02	02.04.02.03	Incorrect Figure called out. FSAR Subsection 2.4.2.3, paragraph 18 begins with 'The Plant Design…' Second Sentence change 'Table 2.4.2-201' To read: Table 2.4.2-207	Editorial
5665	BLN	Pt 02	FSAR 02	02.04.03.04	COLA Part 2, FSAR Chapter 2, Subsection 2.4.3.4 will be revised from: "Ød = Developed angle of friction of soil material. A conservative value of 13 degrees was adopted for materials in the dams investigated." To read: "Ød = Developed angle of friction of soil material. A conservative value of 13 degrees was adopted for materials in the dams investigated."	Editorial
5666	BLN	Pt 02	FSAR 02	02.04.04.01	COLA Part 2, FSAR Chapter 2, Subsection 2.4.4.1 will be revised from: "Failure scenarios for Fontana Dam includes assumed simultaneous failure..." To read: "Failure scenarios for Fontana Dam include assumed simultaneous failure..."	Editorial
5667	BLN	Pt 02	FSAR 02	02.04.04.01	COLA Part 2, FSAR Chapter 2, Subsection 2.4.4.1 will be revised from: "Fort Loudoun, Tellico, and Watts Bar have previously been are not expected to fail in the OBE, as previously discussed." To read: "Fort Loudoun, Tellico, and Watts Bar are not expected to fail in the OBE, as previously discussed."	Editorial
5668	BLN	Pt 02	FSAR 02	02.04.05	COLA Part 2, FSAR Chapter 2, Subsection 2.4.5 will be revised from: "The top of gates at Guntersville Dam, is 595.4 ft. (Reference 222)." To read: "The top of gates at Guntersville Dam is 595.4 ft. (Reference 222)."	Editorial
5669	BLN	Pt 02	FSAR 02	02.04.11.03	COLA Part 2, FSAR Chapter 2, Subsection 2.4.11.3 will be revised from: "Five major reservoirs provide almost 90 percent of the flood storage capacity of the watershed above Chattanooga Tennessee (Reference 211)." To read: "Five major reservoirs provide almost 90 percent of the flood storage capacity of the watershed above Chattanooga, Tennessee (Reference 211)."	Editorial
5670	BLN	Pt 02	FSAR 02	02.04.12.1.1	COLA Part 2, FSAR Chapter 2, Subsection 2.4.12.1.1 will be revised from: "...the Sequatchie Valley fault (Reference 242)." To read: "...the Sequatchie Valley fault (Reference 242)."	Editorial
5673	BLN	Pt 02	FSAR 02	02.04.12.2.4.2	COLA Part 2, FSAR Chapter 2, Subsection 2.4.12.2.4.2 will be revised to add a period after each use of "ft" to read "ft." and to remove the comma after "therefore" in the second sentence of the second paragraph.	Editorial

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
5674	BLN	Pt 02	FSAR 02	02.04.14	COLA Part 2, FSAR Chapter 2, Subsection 2.4.14 will be revised from: "The grade elevation of the BLN are above the probable maximum flood elevation;..." To read: "The grade elevation of BLN is above the probable maximum flood elevation;..."	Editorial
5962	BLN	Pt 02	FSAR 02	02.04.T / T2.4.12-204 Sh2	COLA Part 2, FSAR Chapter 2, Table 2.4.12-204 Sheet 2 of 3 will be revised to change the Notes column for Monitoring Point MW-1212b from "98145.45189" to "(m)"	Editorial
6294	BLN	Pt 02	FSAR 02	02.04.T. / T2.4.13-204	8. COLA Part 2, FSAR Chapter 2, Section 2.4, Table 2.4.13-204 values will be revised.	RAI LTR 063 S1 response to RAI 02.04.13-04a S1 item 8 (Changes 1 through 7 were included in the August 1, 1008 response)
2497	BLN	Pt 02	FSAR 02	02.05.01.01.03.02	2. COLA Part 2, FSAR. Chapter 2, subsection 2.5.1.1.3.2, last paragraph will be revised to include a discussion of features identified in Gadsden soils. [Note that because the superseded version of this letter was incorporated into Rev 1 via Qb 1612, this change actually replaces the last four (4) paragraphs of the Rev 1 section. erg]	RAI LTR 123S2 response to RAI 02.05.01-01, item 2 Note this SUPERSEDES previous Rev 1 change made via Qb 1612.
2498	BLN	Pt 02	FSAR 02	02.05.01.01.03.02	3. COLA Part 2, FSAR. Chapter 2, subsection 2.5.1.1.3.2, 3rd paragraph to update the Figure references to Figures 2.5-208a and -208b. [Note - this change previously incorporated into FSAR Rev 1.]	DUPLICATE of Qb 1613 previously incorporated into Rev 1 - RAI LTR 123S2 response to RAI 02.05.01-01, item 3
2581	BLN	Pt 02	FSAR 02	02.05.02.04.04.01.03	COLA Part 2, FSAR Chapter 2, Section 2.5.2.4.4.1.3, equation 2.5.2-11 for u2(t) will be corrected	DUPLICATE of Qb 2291 previously incorporated into Revision 1 - BLN VOL-02_05_02 Response to NRC Observations
2582	BLN	Pt 02	FSAR 02	02.05.02.04.04.01.03	COLA Part 2, FSAR Chapter 2, Subsection 2.5.2.4.4.1.3, will be revised in the paragraph "Recent discussions with Dr. Tuttle (Reference 386) indicate... ...all ruptures are similar in size to the 1811-1812 earthquakes. To read: Recent discussions with Dr. Tuttle (Reference 386) indicate that she considers that the difference between the size of the 1811-1812 earthquakes and those of the 900 and 1450 sequences is likely to be smaller than what was portrayed in Figure 6 of Tuttle et al. (Reference 374). As a result, Exelon provided a revised model (Reference 356, as shown in the final revision of the Exelon application, Reference 294) for New Madrid sequences to consist of two alternative models of rupture or earthquake sequences. In Model A, all ruptures are similar in size to the 1811-1812 earthquakes.	BLN VOL-02_05_02 Response to NRC Observations
5316	BLN	Pt 02	FSAR 02	02.05.02.04.04.02	1. COLA, part 2, Chapter 2, Subsection 2.5.2.4.4.2, will be revised to read: An updated source characterization logic tree for repeating large magnitude Charleston earthquakes based on these new data is presented in Figure 2.5-268A. The logic tree is simplified and does not include all possible branches. Missing are branches for maximum earthquake magnitudes, mean repeat times, and the two different durations of the paleoliquefaction record, or "completeness intervals. Data for the three branches are presented in Tables 2.5-215, 2.5-216, and 2.5-217, and the basis for the alternative characterizations is described as follows.	BLN RAI LTR 155 response to RAI 02.05.02-010 item 1
5317	BLN	Pt 02	FSAR 02	02.05.02.04.04.04	2. COLA, part 2, Chapter 2, Subsection 2.5.2.4.4.4, will be revised to read: The Charleston source was simplified to be represented by the Woodstock fault (see Figure 2.5-262), because this fault is near the center of the alternative geometries and this fault has the highest	BLN RAI LTR 155 response to RAI 02.05.02-010 item 2

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					weight among alternative geometries (see Figure 2.5-268A). Further, the Charleston source contributes only a small percentage of the hazard at BLN, as is discussed below. This simplification does not affect hazard results in a significant way. A simplified logic tree for the Woodstock source is presented in Figure 2.5-268B. The hazard analysis is based on Scenarios 1, 2, and 3 and does not account for events in the 6,000-yr interval of the paleoliquefaction record.	
2556	BLN	Pt 02	FSAR 02	02.05.04.01.03	1. COLA Part 2, FSAR Chapter 2, Subsection 2.5.4.1.3, will be revised in the paragraph "The relief and hydraulic gradient... ...Monteagle Limestones crop out beneath the Permian Sandstone cap. To read: The relief and hydraulic gradient at the BLN site are not favorable for the development of large cavern systems. In lowland areas like the BLN site, where limestone units have little relief, are relatively close to groundwater levels, and groundwater has relatively low hydraulic gradients, cave systems that can be entered and explored are not known. A map of the distribution of caves in northeastern Alabama shows many hundreds of caves, mostly in highland areas (Figure 2.5-303A: Reference 413). Caves within five miles of the BLN site (Figure 2.5-303B) are also associated with highlands and are formed in the Mississippian Bangor and Monteagle Limestones which crop out beneath the resistant cap of the Pottsville and Pennington Formations (Alabama Cave Survey, 2008). No enterable caves are known within the Ordovician limestones of the Sequatchie Valley in Alabama.	RAI LTR 101S response to RAI 02.05.04-001 item 1
5963	BLN	Pt 02	FSAR 02	02.05.04.02.03	COLA Part 2, FSAR Chapter 2, Subsection 2.5.4.2.3, 5th paragraph, will be revised from: "Unconfined compressive strength testing was performed on 61 samples, and stress strain measurements were included for 21 of the samples tested." To read: "Unconfined compressive strength testing was performed on 65 samples, and stress strain measurements were included for 21 of the samples tested."	Editorial
6003	BLN	Pt 02	FSAR 02	02.05.04.05.02.01	COLA Part 2, FSAR Chapter 2, Subsection 2.5.4.5.2.1, will be revised to read: Blasting is quicker than rock predrilling or line drilling, but would also result in more noise and vibrations, and a greater potential for damage to rock bearing surfaces. On the other hand, rock predrilling or line drilling would be slower than blasting, but reduces potential for overexcavation to remove damaged rock. The experience from construction of Bellefonte Units 1 and 2 shows that predrilling and line drilling were used for the near-vertical rock excavation sides with controlled blasting techniques for the foundation areas with normal and acceptable levels of damage that were readily addressed by inspection and repairs (Reference 201). Blasting is similarly controlled for Units 3 and 4 to minimize fracturing of the in-situ rock consistent with the needs for removal. The exposed foundation rock after removal of the blasted rock is assessed by the geologic mapping program described in Subsection 2.5.4.5.5, and areas judged to have damages from blasting are remediated.	COL-SER-OI-Ch02 response to OI 02.05.04-005
5676	BLN	Pt 02	FSAR 02	02.05.04.05.03.01	COLA Part 2, FSAR Chapter 2, Subsection 2.5.4.5.3.1 will be revised from: "...preparation of a 1987 soils Investigation at the site." To read: "...preparation of a 1987 soils investigation at the site."	Editorial
5677	BLN	Pt 02	FSAR 02	02.05.04.05.03.02	COLA Part 2, FSAR Chapter 2, Subsection 2.5.4.5.3.2 will be revised To read: "• Standard Proctor Maximum Dry Density (MDD) – 1.93 g/cm<superscript 3> (120.5 pcf)" and "• Cohesion, c (at OMC and 2% above OMC) – 66.0 kPa (0.69 tsf; 1,380 psf)"	Editorial
6002	BLN	Pt 02	FSAR 02	02.05.04.05.05	COLA Part 2, FSAR Chapter 2, Subsection 2.5.4.5.5, will be revised to read: Geologic mapping (based on guidance provided in Appendix A of NUREG/CR-5738 as referenced in Regulatory Guide 1.132) and geophysical exploration (as discussed in Subsection 2.5.4.12.6) of excavations for safety-related structures are conducted. Geologic maps of the excavation sides and the bearing surface are used to document the subgrade conditions and to identify features requiring additional exploration. Unforeseen geologic features that are encountered during mapping or geophysical exploration are evaluated. The geologic maps are also used (prior to placement of concrete or a mud mat for subgrade protection) to identify areas needing additional rock removal,	COL-SER-OI-Ch02 response to OI 02.05.01-001

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					placement of dental concrete or grout, or installation of rock bolts for slope integrity. Subsection 2.5.4.12 provides further discussion of the improvement techniques. The NRC will be notified no later than 30 days before any excavations for safety-related structures are scheduled to be open to allow for NRC examinations and evaluation.	
3994	BLN	Pt 02	FSAR 02	02.05.04.10.01	1. COLA Part 2, FSAR. Chapter 2, Subsection 2.5.4.10.1, third paragraph, will be revised to read: The value for ø was conservatively taken as 46º, the lower bound value for Unit A argillaceous limestone (the weaker of the two rock types) determined from Hoek-Brown analyses discussed in Subsection 2.5.4.2.3.4. The value of c was taken as zero.	RAI LTR 101 S2 response to RAI 02.05.04-018 item 1
3995	BLN	Pt 02	FSAR 02	02.05.04.10.01	2. COLA Part 2, FSAR. Chapter 2, Subsection 2.5.4.10.1 will be revised to add a new paragraph following the third paragraph to read: The Terzaghi equation used in Method 1 is based on length to width (L/B) ratios greater than 10. For L/B ratios less than 10, shape correction factors are applied to the corresponding bearing capacity factors. Correction factors are provided in Table 6-1 of EM 1110-1-2908 (Reference 456). Because the value of cohesion for the rock was taken as 0, only the correction factor for the Nγ term was used in the calculations. The equivalent area mat dimensions for static loading are approximately 127 feet by 256 feet, for an approximate L/B of 2, and a corresponding correction factor of 0.9. To consider eccentric loading that produces the maximum DCD design bearing pressure of 35,000 psf under dynamic loading, the shield building area was converted to an equivalent rectangle having approximate dimensions of 51.7 feet by 146.7 feet. These dimensions are an approximate L/B of 3 with a corresponding shape correction factor of 0.92.	RAI LTR 101 S2 response to RAI 02.05.04-018 item 2
3996	BLN	Pt 02	FSAR 02	02.05.04.10.01	3. COLA Part 2, FSAR. Chapter 2, Section 2.5.4.10.1; last paragraph, first and second bullets will be revised to read: Using the lower bound rock properties for argillaceous limestone as shown in Table 2.5-236, both methods show bearing capacities well above the requirements in DCD Table 2-1 (8900 pounds per square foot [psf] for static and 35,000 psf for dynamic). The calculated ultimate bearing capacities for Method 1 considering shape factor corrections and eccentric loading and allowable bearing capacity for Method 2 are: • Method 1: 692,000 psf static, and 368,000 psf dynamic, and • Method 2: 236,000 psf. This method provides an allowable bearing pressure based on rock properties only, not the process by which loading is applied. It is therefore applicable to both static and dynamic loading.	RAI LTR 101 S2 response to RAI 02.05.04-018 item 3
5678	BLN	Pt 02	FSAR 02	02.05.04.10.03	COLA Part 2, FSAR Chapter 2, Subsection 2.5.4.10.3 will be revised from: "...(217 kpa divided by 3316 MPa = 0.00006)." To read: "...(217 kPa divided by 3316 MPa = 0.00006)."	Editorial
5679	BLN	Pt 02	FSAR 02	02.05.04.10.04.01	COLA Part 2, FSAR Chapter 2, Subsection 2.5.4.10.4.1 will be revised from: "The maximum estimated settlement is 0.18 in. beneath Unit 3 and 0.20 in beneath Unit 4." To read: "The maximum estimated settlement is 0.18 in. beneath Unit 3 and 0.20 in. beneath Unit 4."	Editorial
2283	BLN	Pt 02	FSAR 02	02.05.05.01.01	COLA Part 2, FSAR Chapter 2, Section 2.5.5.1.1 will be revised in the paragraph "Based on the grades in the plant area... ...a potential safety hazard to the Unit 4 Category I Structures." To read: Based on the grades in the plant area as shown on Figure 2.4.2-202, no permanent cut slopes, or man-made fill slopes, exist that could compromise the operation of the safety-related plant facilities. The grading shown on Figure 2.5-362 of the BLN power block construction zone pad is generally level	Seismology/Geotech Trip Report VOL-GEO-20081217-OR

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					at about elevation 628.6 ft. for a minimum distance of over 500 ft. from the perimeter of the BLN nuclear islands. Fill slopes at the perimeter of the fill pad are limited in height to approximately 16 ft., and inclined at approximately 3:1 (horizontal to vertical). Existing graded or natural ground surface inclinations below or adjacent to the edge of the southwest, northwest, and northeast margins of the pad are relatively flat, and do not show evidence of past instability or potential unstable conditions as described in Subsection 2.5.4.1. The southeast margin of the pad extends to the toe of natural ridge slopes, a portion of which is steepened by excavation to extend the level pad southeastward. The steepest slope at the southeast pad margin is an inclination of approximately 3:1 (horizontal to vertical) on an 80 to 160-ft.high cutslope. The toe of this cutslope is at least 950 ft. from the Unit 4 turbine building, and 1000 ft. from the Unit 4 nuclear island. The minimum separation distance between the plant and cutslope toe is over 10 times the slope height, providing a substantial safety buffer zone against possible slope failure under dynamic or static loading conditions. Therefore, this cut slope does not pose a potential safety hazard to the Unit 4 Category I Structures.	
2499	BLN	Pt 02	FSAR 02	02.05.07 REF 478-480	4. COLA Part 2, FSAR Chapter 2, Section 2.5.7 is modified to add new references. 478. Soil Survey Staff, 1999, Soil Taxonomy: a basic system of soil classification for making and interpreting soil surveys: U.S Department of Agriculture, Agriculture Handbook No. 436, revised edition, Chapter 19, Ultisols, pp. 721-781. 479. Natural Resources Conservation Service Web Soil Survey 2.0, accessed 9/17/08. 480. Vepraskas, M. J., 1995, (revised) Redoximorphic features for identifying aquic conditions: Technical Bulletin 301, North Carolina State University, Raleigh, NC.	RAI LTR 123 S2 response to RAI 02.05.01-01, item 4 This change supplements the addition of Ref 477 in RAI LTR 123 original response.
2496	BLN	Pt 02	FSAR 02	02.05F / F2.5-209	1. COLA Part 2, FSAR Chapter 2 Figures are modified by replacing Figures 2.5-208 and 209 as follows: c. Add new Figure 2.5-209, Soil Weathering Features in Pleistocene Terrace Deposits, Gadsden, Alabama. (Attachment 2.5.1-01A)	RAI LTR 123 Supp 2 response to RAI 02.05.01-01, item 1 This change revises the Figure 2.5-209 incorporated in Rev 1 based on RAI LTR 123 original response.
5318	BLN	Pt 02	FSAR 02	02.05F / F2.5-268	3. COLA Part 2, Chapter 2, Figure 2.5-268 will be renumbered to Figure 2.5-268A.	BLN RAI LTR 155 response to RAI 02.05.02-010 item 3
5884	BLN	Pt 02	FSAR 02	02.05F / F2.5-268A/B	COLA Part 2, Chapter 2, Figures 2.5-268A and 268B will be renumbered to Figures 2.5-268a and 2.5-268b. The references in the text are also similarly revised.	Editorial revision to BLN RAI LTR 155 response to RAI 02.05.02-010 item 3
5319	BLN	Pt 02	FSAR 02	02.05F / F2.5-268B	4. COLA Part 2, Chapter 2, will be revised to add new Figure 2.5-268B.	BLN RAI LTR 155 response to RAI 02.05.02-010 item 4
2557	BLN	Pt 02	FSAR 02	02.05F / F2.5-303	2. COLA Part 2, FSAR Chapter 2, Section 2.5, Figure 2.5-303, will be revised as indicated in Attachment 02.05.04-01A.	RAI LTR 101S response to RAI 02.05.04-001 item 2
5687	BLN	Pt 02	FSAR 02	02DD	COLA Part 2, FSAR Chapter 2, Appendix 2DD, will be revised to add LMA of BLN SUP 2.3-2 at the beginning of the Appendix.	Editorial
5326	BLN	Pt 02	FSAR 02	02DD.T / T2DD-204	4. COLA Part 2, FSAR Chapter 2, Appendix 2DD, Table 2DD-204 will be revised from the current Revision 1 to omit the first set of comparisons on Sheet 2 of 6 since this set is a duplicate of the third set on Sheet 1 of 6.	BLN-VOL-LTR 007 item 4
5327	BLN	Pt 02	FSAR 02	02DD.T / T2DD-204	5. COLA Part 2, FSAR Chapter 2, Appendix 2DD, Table 2DD-204 will be revised from the current Revision 1 to replace the second set of comparisons on Sheet 2 of 6 with the set of comparisons shown in Attachment 3.	BLN-VOL-LTR 007 item 5

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
5328	BLN	Pt 02	FSAR 02	02DD.T / T2DD-204	6. COLA Part 2, FSAR Chapter 2, Appendix 2DD, Table 2DD-204 will be revised from the current Revision 1 to replace the first set of comparisons on Sheet 5 of 6 with the set of comparisons shown in Attachment 3.	BLN-VOL-LTR 007 item 6
5134	BLN	Pt 02	FSAR 03	03.00 TOC	COLA Part 2, FSAR Chapter 3, Table of Contents for 3.9.3.4.4 is revised to add a comma after "Repair" to read: Inspection, Testing, Repair, and/or Replacement"	Editorial
5694	BLN	Pt 02	FSAR 03	03.05.01.06	COLA Part 2, FSAR Chapter 3, Subsection 3.5.1.6 will be revised from: "The aircraft handling facilities and air routes are described in Subsection 2.2.2.6." To read: "The aircraft handling facilities and air routes are described in Subsection 2.2.2.7."	Editorial
5421	BLN	Pt 02	FSAR 03	03.07.04.02.01	4. COLA Part 2, FSAR Chapter 3, Subsection 3.7.4.2.1, will be revised to add the following sentence to the end of the existing FSAR added text: The trigger value is initially set at 0.01g.	COL-SER-OI-Ch01 response to OI 01.04-01 item 4
4799	BLN,STD	Pt 02	FSAR 03	03.09.03.04.04	Revise Section title to add comma after Repair, to read "Inspection, Testing, Repair, and/or Replacement of Snubbers"	Consistency with DCD
5591	BLN,STD	Pt 02	FSAR 03	03.09.03.04.04, item a.1	2. COLA, Part 2, Revision 1, FSAR Chapter 3, Subsection 3.9.3.4.4, item a.1, will be revised to read: A list of snubbers on systems which experience sufficient thermal movement to measure cold to hot position is included in Table 3.9-201.	RAI LTR 007 S2 response to RAI 03.09.06-003 item 2
5592	BLN,STD	Pt 02	FSAR 03	03.09.03.04.04, item a.3	3. COLA, Part 2, Revision 1, FSAR Chapter 3, Subsection 3.9.3.4.4, item a.3, will be revised to read: Safety-related snubbers are identified in Table 3.9-201, including the snubber identification and the associated system or component, e.g., line number. The snubbers on the list are hydraulic and constructed to ASME Section III, Subsection NF. The snubbers are used for shock loading only. None of the snubbers are dual purpose or vibration arrestor type snubbers.	RAI LTR 007 S2 response to RAI 03.09.06-003 item 3
2284	BLN,STD	Pt 02	FSAR 03	03.09.06.02.02	Add "(Reference 201)" {red, hyperlinked text} after "MPR-2524-A" on top of page 3.9-9 (new text to be inserted under the bulleted item titled, "Risk Ranking.")	Editorial
5593	BLN,STD	Pt 02	FSAR 03	03.09.T / T3.9-201	4. COLA, Part 2, FSAR Chapter 3, Table 3.9-201 will be added to read: TABLE 3.9-201 SAFETY RELATED SNUBBERS	RAI LTR 007 S2 response to RAI 03.09.06-003 item 4
5691	BLN,STD	Pt 02	FSAR 03	03.09.T / T3.9-201	COLA, Part 2, FSAR Chapter 3, Table 3.9-201 will be revised to add LMA of STD SUP 3.9-3.	Editorial
5108	BLN	Pt 02	FSAR 05	05.02.04.01	COLA Part 2, FSAR, Subsection 5.2.4.1, fifth paragraph, will be revised To read: The inservice inspection program is augmented for reactor vessel top head inspections by use of the ASME Code Case N-729-1, "Alternative Examination Requirements for Pressurized-Water Reactor (PWR) Vessel Upper Heads With Nozzles Having Pressure-Retaining Partial-Penetration Welds," as modified by the conditions specified in 10 CFR 50.55a(g)(6)(ii)(D).	SER with Open Items Confirmatory Item 5.2-1
4800	BLN,STD	Pt 02	FSAR 05	05.03.02.06	COLA Part 2, FSAR, Chapter 5, Subsection 5.3.2.6, will be revised from the commitment provided in July 10, 2008, supplemental response to provide extensive discussion of the capsule preparation process.	RAI LTR 002 Supp 2 response to RAI 05.03.01-01(a) SER with Open Items Confirmatory Item 5.3-1
5693	BLN	Pt 02	FSAR 05	05.03.02.06	COLA Part 2, FSAR, Chapter 5, Subsection 5.3.2.6 (as revised by the Supplement 2 response to BLN-RAI-LTR-002) will be revised from: The type and quantity of test specimens exceed the minimum requirements of EI85-82. To read:	COL-SER-OI-Ch05 response to OI 05.03-01

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					The type, quantity, and storage conditions (e.g., surveillance capsules backfilled with inert gas) of test specimens meet or exceed the minimum requirements of ASTM E-185.	
5077	BLN,STD	Pt 02	FSAR 06	06.01.02.01.06	3. COLA Part 2, FSAR Chapter 6, Subsection 6.1.2.1.6, Service Level I and III Coatings, 1st paragraph, will be revised to read: Regulatory Guide 1.54 and ASTM D5144 (Reference 201) form the basis for the coating program.	BLN-VOL-LTR-005 item 3
5078	BLN,STD	Pt 02	FSAR 06	06.01.02.01.06	4. COLA Part 2, FSAR Chapter 6, Subsection 6.1.2.1.6, Service Level I and III Coatings, 2nd paragraph, will be revised to read: Coating system monitoring requirements for the containment coating systems are based on ASTM D5163 (Reference 202), "Standard Guide for Establishing Procedures to Monitor the Performance of Coating Service Level I Coating Systems in an Operating Nuclear Power Plant," and ASTM D7167 (Reference 203), "Standard Guide for Establishing Procedures to Monitor the Performance of Safety-Related Coating Service Level III Lining Systems in an Operating Nuclear Power Plant."	BLN-VOL-LTR-005 item 4
5075	BLN,STD	Pt 02	FSAR 06	06.01.02.01.06	1. COLA Part 2, FSAR Chapter 6, Subsection 6.1.2.1.6 will be revised to add: Add the following after the third paragraph of the subsection titled "Service Level II Coatings" within DCD Subsection 6.1.2.1.6. Coating system inspection and monitoring requirements for the Service Level II coatings used inside containment will be performed in accordance with a program based on ASTM D5144 (Reference 201), "Standard Guide for Use of Protective Coating Standards in Nuclear Power Plants" and the guidance of ASTM D5163 (Reference 202), " Standard Guide for Establishing Procedures to Monitor the Performance of Coating Service Level I Coating Systems in an Operating Nuclear Power Plant." Any anomalies identified during coating monitoring are resolved in accordance with applicable quality requirements.	BLN-VOL-LTR-005 item 1
5076	BLN,STD	Pt 02	FSAR 06	06.01.03.02	2. COLA Part 2, FSAR Chapter 6, Add the following new subsection after subsection 6.1.3.2: The following information supplements the information provided in DCD subsection 6.1.4. 6.1.4 References 201. ASTM 5144-08, "Standard Guide for Use of Protective Coating Standards in Nuclear Power Plants" 202. ASTM D5163-05a, "Standard Guide for Establishing Procedures to Monitor the Performance of Coating Service Level I Coating Systems in an Operating Nuclear Power Plant" 203 ASTM D7167-05, "Standard Guide for Establishing Procedures to Monitor the Performance of Safety-Related Coating Service Level III Lining Systems in an Operating Nuclear Power Plant"	BLN-VOL-LTR-005 item 2
3483	BLN	Pt 02	FSAR 06	06.02.05.01.02	4. COLA Part 2, FSAR Chapter 6, will be revised to include the following new Subsection 6.2.5.1.2 with an LMA of BLN DEP 2.3-1. 6.2.5.1.2 Power Generation Design Basis Replace the second sentence of DCD Subsection 6.2.5.1.2 with the following sentence. The specified maximum allowable containment leak rate is 0.09 weight percent of the containment air mass per day at the calculated peak accident pressure, Pa, identified in subsection 6.2.1.	RAI LTR 129S response to RAI 15.00.03-001, item 4
5049	BLN,STD	Pt 02	FSAR 06	06.03.08.01	4. COLA Part 2, FSAR Chapter 6, Subsection 6.3.8.1 will be revised to read:	RAI LTR 030 S2 response to RAI 06.02.02-001 item

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					6.3.8.1 Containment Cleanliness Program Insert the following information at the end of DCD Subsection 6.3.8.1: This COL Item is addressed below. Administrative procedures implement the containment cleanliness program. Implementation of the program minimizes the amount of debris left in containment following personnel entry and exits. The program is consistent with the containment cleanliness program limits discussed in DCD Subsection 6.3.8.1. The program includes, as a minimum, the following: Responsibilities The program defines the organizational responsibilities for implementing the program; defines personnel and material controls; and defines the inspection and reporting requirements. Implementation Containment Entry/Exit • Controls to account for the quantities and types of materials introduced into the containment. • Limits on the types and quantities of materials, including scaffolding and tools, to ensure adequate accountability controls. This may be accomplished by the work management process. Storage of aluminum is prohibited without engineering authorization. Cardboard boxes or miscellaneous packing material is not brought into containment without approval. • If entries are made at power, prohibited materials and limits on quantities of materials that may generate hydrogen are established. • Controls for loose items, such as keys and pens, which could be inadvertently left in containment. • Methods and controls for securing any items and materials left unattended in containment. • Administrative controls for accounting for tools, equipment and other material are established. • Administrative controls for accounting of the permanent removal of materials previously introduced into the containment. • Limits on the types and quantities of materials, including scaffolding and tools, that may be left unattended in containment during outages and power operation. Types of materials considered are tape, labels, plastic film, and paper and cloth products. • Requirements and actions to be taken for unaccounted for material. • Requirements for final containment cleanliness inspections consistent with the design bases provided in DCD Subsection 6.3.8.1. • Record keeping requirements for entry/exit logs. Housekeeping Housekeeping procedures require that work areas be maintained in a clean and orderly fashion during work activities and returned to original conditions (or better) upon completion of work. Sampling Program A sampling program is implemented consistent with NEI Guidance Report 04-07, "Pressurized Water Reactor Sump Performance Evaluation Methodology" as supplemented by the NRC in the "Safety Evaluation by The Office of Nuclear Reactor Regulation Related to NRC Generic Letter 2004-02, Nuclear Energy Institute Guidance Report (Proposed Document Number NEI 04-07), 'Pressurized Water Reactor Sump Performance Evaluation Methodology."' Latent debris sampling is implemented before startup. The sampling is conducted after containment exit cleanliness inspections to provide reasonable assurance that the plant latent debris design bases are met. Sampling frequency and scope may be adjusted based on sampling results. Results are evaluated post-start up and any nonconforming results will be addressed in the Corrective Action Program.	4 SUPERSEDES RAI LTR 030 S1 response to RAI 06.02.02-001
3972	BLN,STD	Pt 02	FSAR 06	06.04.04	2. COLA Part 2, FSAR Chapter 6, Subsection 6.4.4, will be revised to add the following information	RAI LTR 137 response to

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					with LMAs STD SUP 6.4-2 and BLN COL 6.4-1: Insert the following information at the end of the eighth paragraph of DCD Subsection 6.4.4. Table 6.4-202 provides additional details regarding the evaluated onsite chemicals.	02.02.03-10, item 2
6320	BLN	Pt 02	FSAR 06	06.04.04	COLA Part 2, FSAR Chapter 6, Subsection 6.4.4, will be revised to add an LMA of STD COL 6.4-1.	Editorial revision to RAI LTR 137 response to 02.02.03-10, item 2
3862	BLN	Pt 02	FSAR 06	06.04.04.02	10. COLA Part 2, FSAR Chapter 6, Subsection 6.4.4.2 will be revised.	RAI LTR 132 S1 response to RAI 02.02.03-008, item 10
5320	BLN	Pt 02	FSAR 06	06.04.04.02	1. COLA Part 2, FSAR Chapter 6, Subsection 6.4.4.2, tenth paragraph (as revised in response to BLN-RAI-LTR-132, Supplement 1) will be revised from: The analysis shows that for the case resulting in the most rapid rise in the chlorine concentration inside the control room, it takes approximately ten minutes after the event initiation before the chlorine concentration reaches the human detection threshold of 0.31 ppm. The chlorine concentration inside the control room would reach the IDLH value of 10 ppm 16 minutes after the event initiation, or 6 minutes after human detection. To read: The analysis shows that for the case resulting in the most rapid rise in the chlorine concentration inside the control room, it takes approximately ten minutes after the event initiation before the chlorine concentration reaches the human detection threshold of 0.31 ppm. The chlorine concentration inside the control room would reach the IDLH value of 10 ppm at approximately 16 minutes after the event initiation, or approximately 6 minutes after human detection.	BLN RAI LTR 159 response to RAI 06.04-006 item 1
5321	BLN	Pt 02	FSAR 06	06.04.04.02	2. COLA Part 2, FSAR Chapter 6, Subsection 6.4.4.2, 13th paragraph (as revised in response to BLNRAI-LTR-132, Supplement 1) will be revised from: The sensitivity study shows that for the most rapid hydrogen fluoride concentration build up inside the control room it takes approximately five to six minutes after the event takes place before hydrogen fluoride concentration at the control room HVAC intake reaches elevated levels. Approximately one additional minute or less passes before the hydrogen fluoride concentration inside the control room reaches the human detection threshold of 0.04 ppm. Hydrogen fluoride concentration inside the control room would reach the IDLH value of 30 PPM at about 27 minutes, or 15 minutes after human detection. To read: The sensitivity study shows that for the most rapid hydrogen fluoride concentration build up inside the control room it takes approximately five to six minutes after the event takes place before hydrogen fluoride concentration at the control room HVAC intake reaches elevated levels. Approximately one additional minute or less passes before the hydrogen fluoride concentration inside the control room reaches the human detection threshold of 0.04 ppm. Hydrogen fluoride concentration inside the control room would reach the IDLH value of 30 PPM at approximately 27 minutes, or approximately 15.75 minutes after human detection.	BLN RAI LTR 159 response to RAI 06.04-006 item 2
5322	BLN	Pt 02	FSAR 06	06.04.04.02	3. COLA Part 2, FSAR Chapter 6, Subsection 6.4.4.2, (as revised in response to BLN-RAI-LTR-132, Supplement 1) will be revised to include a new paragraph (following the 16th paragraph which begins "A combined operator manual action…") to read: With VES in operation the only potential toxic inflow to the MCR envelope is the inleakage total of 15 cfm. This inleakage results in a slight increase in toxic gas concentration in the interior of the MCR, but remains below the chemical IDLH peak concentration criteria. For chlorine, the allowed operator	BLN RAI LTR 159 response to RAI 06.04-006 item 3

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					action time decreases from approximately 6 minutes to approximately 5.75 minutes with odor detection of 0.31 ppm and for hydrogen fluoride, the allowed operator action time decreases from approximately 15.75 minutes to approximately 15.5 minutes. Allowed operator action time is greater than 2 minutes and therefore satisfies Regulatory Guide (RG) 1.78 guidance for protecting the control room operator from toxic gas releases.	
5479	BLN	Pt 02	FSAR 06	06.04.04.02	COLA Part 2, FSAR Chapter 6, Subsection 6.4.4.2 will be revised from: The VES can provide sufficient air for numerous personal for the duration of this short term event. To read: The VES can provide sufficient air for numerous personnel for the duration of this short term event.	Editorial revision to RAI LTR 132 S1 response to RAI 02.02.03-008, item 10
3863	BLN	Pt 02	FSAR 06	06.04.T / T6.4-201	11. COLA Part 2, FSAR Chapter 6, Table 6.4-201 will be revised. TABLE 6.4-201 INPUT VALUES USED IN CHEM ANALYSIS OF CHLORINE	RAI LTR 132 S1 response to RAI 02.02.03-008, item 11
3973	BLN,STD	Pt 02	FSAR 06	06.04.T / T6.4-202	3. COLA Part 2, FSAR Chapter 6, new Table 6.4-202 - Onsite Chemicals will be added (with LMAs as shown) to read {Reviewer's Note: The DCD evaluated hazards are identified in FSAR Table 6.4-202 as standard supplemental (STD SUP) material. Revisions to the amounts and distances evaluated by WEC since the time of the DCD material approval are identified as standard COL information item (STD COL) material. Any additional site specific chemicals used, along with quantities and locations stored onsite are also identified in the new FSAR Table 6.4-202 as site specific COL information item (BLN COL) material. This note for reviewer information only and is not a part of the COLA change.}	RAI LTR 137 response to 02.02.03-10, item 3
5013	BLN,STD	Pt 02	FSAR 06	06.04.T / T6.4-202	COLA Part 2, FSAR Chapter 6, Table 6.4-202 - Onsite Chemicals, footnote 1, will be revised to read (adds new last sentence): 1) This table supplements DCD Table 6.4-1. Quantities are by largest container content for the specified location per unit. Quantities and distances are bounding calculation values and not actual amounts and distances.	Clarification of table added by RAI LTR 137 response to 02.02.03-10, item 3
5014	BLN,STD	Pt 02	FSAR 06	06.04.T / T6.4-202	COLA Part 2, FSAR Chapter 6, Table 6.4-202 - Onsite Chemicals, will be revised to remove superscript (b) from Corrosion Inhibiter and Scale Inhibiter Quantity values.	Correction of table added by RAI LTR 137 response to 02.02.03-10, item 3
5725	BLN,STD	Pt 02	FSAR 06	06.04.T / T6.4-202	COLA Part 2, FSAR Chapter 6, Table 6.4-202 - Onsite Chemicals, will be revised to add superscript note (3) to Corrosion Inhibiter and Scale Inhibiter Quantity values; and to add new Note 3) at the bottom of the table that reads: "3) Corrosion inhibitor and scale inhibitor are mixed together in a 10,000 gal tank."	Addition to table added by RAI LTR 137 response to 02.02.03-10, item 3
6319	BLN	Pt 02	FSAR 06	06.04.T / T6.4-202	COLA Part 2, FSAR Chapter 6, Table 6.4-202, is revised to replace each LMA of STD SUP 6.4-1 with an LMA of STD SUP 6.4-2.	Editorial revision to RAI LTR 137 response to 02.02.03-10, item 3
3864	BLN	Pt 02	FSAR 06	06.04F / F6.4-201	12. COLA Part 2, FSAR Chapter 6, Figure 6.4-201 will be revised.	RAI LTR 132 S1 response to RAI 02.02.03-008, item 12
3865	BLN	Pt 02	FSAR 06	06.04F / F6.4-202	13. COLA Part 2, FSAR Chapter 6, add Figure 6.4-202 to include hydrogen fluoride.	RAI LTR 132 S1 response to RAI 02.02.03-008, item 13
3484	BLN	Pt 02	FSAR 06	06.05	5. COLA Part 2, FSAR Chapter 6, Section 6.5 will be revised To read: This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.	RAI LTR 129S response to RAI 15.00.03-001, item 5
3485	BLN	Pt 02	FSAR 06	06.05.T / T6.5-201	6. COLA Part 2, FSAR Chapter 6, Section 6.5, will be revised to add the following new table with an LMA of BLN DEP 2.3-1.	RAI LTR 129S response to RAI 15.00.03-001, item 6

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					TABLE 6.5-201 BLN PRIMARY CONTAINMENT OPERATION FOLLOWING A DESIGN BASIS ACCIDENT BLN Design basis containment leak rate 0.09% containment air weight per day Note: This table supplements DCD Table 6.5.3-1.	
5726	BLN	Pt 02	FSAR 07	07.05	COLA Part 2, FSAR Chapter 7, Revision 1 will be revised to add separator bars and introductory statements for the table references (with a single LMA of BLN SUP 7.5-1) to read: ---------- Add the following paragraph at the end of Subsection 7.5.2. BLN SUP 7.5-1 FSAR Table 7.5-201 supplements DCD Table 7.5-1 and provides variable data shown in the DCD Table as "site specific." Add the following paragraph at the end of Subsection 7.5.3.5. BLN SUP 7.5-1 FSAR Table 7.5-202 supplements DCD Table 7.5-8 and provides variable data shown in the DCD Table as "site specific." ----------	Editorial (includes Qb 4803)
4803	BLN	Pt 02	FSAR 07	07.05	1- COLA Part 2, FSAR Chapter 7, Revision 1 will be revised to add the following paragraph at the end of section 7.5: BLN SUP 7.5-1 FSAR Table 7.5-202 supplements DCD Table 7.5-8 and provides variable data shown in the DCD Table as "site specific."	RAI LTR 089 S1 response to RAI 07.05.01, item 1
4804	BLN	Pt 02	FSAR 07	07.05.T / T7.5-201	2- COLA Part 2, FSAR Chapter 7, Revision 1 Table 7.5-201, will be revised to include the site specific ranges for the boundary environs radiation.	RAI LTR 089 S1 response to RAI 07.05.01, item 2
5895	BLN	Pt 02	FSAR 07	07.05.T / T7.5-201	COLA Part 2, FSAR Chapter 7, Revision 1 Table 7.5-201, will be revised to include the following statement in the Remarks column: Conforms to Regulatory Guide 1.97	Clarification of RAI LTR 089 S1 response to RAI 07.05.01, item 2
4805	BLN	Pt 02	FSAR 07	07.05.T / T7.5-202	3- COLA Part 2, FSAR Chapter 7, Revision 1 Table 7.5-202 - Summary of Type E Variables will be added as a supplement to DCD Table 7.5-8	RAI LTR 089 S1 response to RAI 07.05.01, item 3
5093	BLN,STD	Pt 02	FSAR 08	08.02.01.02	1. COLA Part 2, Subsection 8.2.1.2 (with LMA of BLN COL 8.2-1), is revised to read: Add the following paragraph at the end of the first paragraph of DCD Subsection 8.2.1.2. The transformer area for each unit contains the main stepup transformer (the GSU), (3 single phase transformers plus one spare), three unit auxiliary transformers (the UATs), and two reserve auxiliary transformers (the RATs). The two RATs are connected to the 500 kV to 230 kV transformer located in the 500 kV switchyard. The high side (500 kV) winding of the GSUs is connected in wye configuration to the 500 kV switchyard. Add the following paragraph and subsections at the end of the DCD Subsection 8.2.1.2. Each transformer is connected to the switchyard by an offsite circuit beginning at the switchyard side of the breaker(s) within the switchyard and ending at the high voltage terminals of the GSU and RATs.	RAI LTR 027 S1 response to RAI 14.03-001 item 1 SER with Open Items Confirmatory Item 8.2A-1 SER with Open Items Confirmatory Item 14.3-1
4885	BLN	Pt 02	FSAR 08	08.02.01.02.02	COLA Part 2, FSAR Chapter 8, will be revised to add the following paragraph at the end of Subsection 8.2.1.2.2:	RAI LTR 149 response to Verbal Request

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					The protective devices controlling the switchyard breakers are set with consideration given to preserving the plant grid connection following a turbine trip.	
5143	BLN	Pt 02	FSAR 08	08.02.01.04	COLA Part 2, Subsection 8.2.1.4, correct spelling of maintenance in second paragraph (currently maintainance)	Editorial
6052	BLN	Pt 02	FSAR 08	08.02.02	COLA Part 2, FSAR Chapter 8, Section 8.2.2, Grid Stability, revise the first paragraph from: "In order to maintain reactor coolant pump (RCP) operation for three seconds following a turbine trip as specified in DCD Subsection 8.2.2, the grid voltage at the high-side of the GSU, and RATs cannot drop more than 15 percent from the pre-trip steady-state voltage." To read: "In order to maintain reactor coolant pump (RCP) operation for three seconds following a turbine trip as specified in DCD Subsection 8.2.2, the grid voltage at the high-side of the GSU, and RATs cannot dip more than 0.15 pu from the pre-trip steady-state voltage."	Constency with Westinghouse interface criteria
6053	BLN	Pt 02	FSAR 08	08.02.02	COLA Part 2, FSAR Chapter 8, Section 8.2.2, Grid Stability, revise the next to last paragraph from: "The 15 percent maximum voltage drop requirement is also met when there is another transmission element out of service, including the largest generator or most critical transmission line." To read: "The 0.15 pu maximum voltage dip requirement is also met when there is another transmission element out of service, including the largest generator or most critical transmission line."	Constency with Westinghouse interface criteria
4882	BLN	Pt 02	FSAR 08	08.02.02	1. COLA Part 2, FSAR Chapter 8, Subsection 8.2.2 will be revised to read: Table 8.2-201 confirms that the interface requirements for steady state load, inrush kVA for motors, nominal voltage, allowable voltage regulation, nominal frequency, allowable frequency fluctuation, maximum frequency decay rate, and limiting under frequency value for RCP have been met.	RAI LTR 026 S1 response to RAI 08.02-007 item 1 SER with Open Items Confirmatory Item 8.2-2
4883	BLN	Pt 02	FSAR 08	08.02.T / T8.2-201	2. COLA Part 2, FSAR Chapter 8, will be revised to include new Table 8.2-201 "Grid Stability Interface Evaluation" with an LMA of BLN COL 8.2-2	RAI LTR 026 S1 response to RAI 08.02-007 item 2 SER with Open Items Confirmatory Item 8.2-2
6054	BLN	Pt 02	FSAR 08	08.02.T / T8.2-201	COLA Part 2, FSAR Chapter 8, Table 8.2-201, line item entries for Allowable voltage regulation will be revised from "+/- 20% total for transient" to read "0.15 pu transient dip**" with the addition of Note ** to read: ** Applicable to Turbine Trip Only. The maximum allowable voltage dip from the pre-event steady state voltage value during the 3 second turbine trip event transient as measured at the point of connection to the high side of the generator step-up transformer and the reserve auxiliary transformer.	Editorial
4884	BLN,STD	Pt 02	FSAR 08	08.03.01.01.02.04	COLA Part 2, FSAR Chapter 8, Section 8.3.1.1.2.4 will be revised from: Operation, inspection and maintenance procedures consider both the diesel generator manufacturer's recommendations and industry diesel working group recommendations. To read: Operation, inspection and maintenance (including preventive, corrective, and predictive maintenance) procedures consider both the diesel generator manufacturer's recommendations and industry diesel working group recommendations.	RAI LTR 149 response to RAI 08.03.01-002 SER with Open Items Confirmatory Item 8.3-1
2493	BLN,STD	Pt 02	FSAR 08	08.03.01.01.06	COLA Part 2, FSAR Chapter 8, Subsection 8.3.1.1.6 will be revised To read: Procedures implement periodic testing of protective devices that provide penetration overcurrent protection. A sample of each different type of overcurrent device is selected for periodic testing during refueling outages. Testing includes:	RAI LTR 138 in response to RAI 08.03.01-001 SER with Open Items Confirmatory Item 8.3.2-1

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					• Verification of thermal and instantaneous trip characteristics of molded case circuit breakers. • Verification of long time, short time, and instantaneous trips of medium voltage vacuum circuit breakers. • Verification of long time, short time, and instantaneous trips of low voltage air circuit breakers. • Verification of Class 1E and non-Class 1E dc protective device characteristics (except fuses) per manufacturer recommendations, including testing for overcurrent interruption and/or fault current limiting. Penetration protective devices are maintained and controlled under the plant configuration control program. A fuse control program, including a master fuse list, is established based on industry operating experience.	
4943	BLN,STD	Pt 02	FSAR 08	08.03.01.04	COLA Part 2, FSAR Chapter 8, will be revised to add the following paragraph at the end of Subsection 8.3.1.4. 8.3.1.4 Inspection and Testing Add the following text at the end of DCD Subsection 8.3.1.4 Procedures are established for periodic verification of proper operation of the Onsite AC Power System capability for automatic and manual transfer from the preferred power supply to the maintenance power supply and return from the maintenance power supply to the preferred power supply.	RAI LTR 151 response to RAI 08.02-010(b) SER with Open Items Confirmatory Item 8.2-1
5959	BLN	Pt 02	FSAR 08	08.03.01.04	COLA Part 2, FSAR Chapter 8, Subsection 8.3.1.4, will be revised to add an LMA of STD SUP 8.3-4. Also revise LMA on 8.3.2.1.1.1 from STD SUP 8.3-1 to STD SUP 8.3-3. There is a previous use of SUP 8.3-1.	Revises BLN RAI LTR 151 response to RAI 08.02-010 (b) SER with Open Items Confirmatory Item 8.2-1
5144	BLN	Pt 02	FSAR 09	09.01.05	COLA Part 2, Subsection 9.1.5, remove semicolon near bottom of page 9.1-2 following "corrective action"	Editorial for consistency
5145	BLN	Pt 02	FSAR 09	09.01.05.04	COLA Part 2, Subsection 9.1.5, remove capitalization of "Intervals" in third bullet - change to lower case	Editorial for consistency
5862	BLN,STD	Pt 02	FSAR 09	09.01.06	1- COLA Part 2, FSAR Chapter 9, Subsection 9.1.6 will be revised to read: STD COL 9.1-7 A spent fuel rack Metamic coupon monitoring program is to be implemented when the plant is placed into commercial operation. This program includes tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and / or visual examination. The program will also include tests to monitor changes in physical properties of the absorber material, including neutron attenuation and thickness measurements.	RAI LTR 165 in response to RAI 09.01.02-001 item 1
6372	BLN	Pt 02	FSAR 09	09.01.06	COLA Part 2, FSAR Chapter 9, Subsection 9.1.6 will be revised To read: STD COL 9.1-7 A spent fuel rack Metamic coupon monitoring program will be implemented when the plant is placed into commercial operation. This program will include tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and / or visual examination. The program will also include testing to monitor changes in physical properties of the absorber material, including neutron attenuation and thickness measurements.	Editorial revision of RAI LTR 165 in response to RAI 09.01.02-001 item 1
5198	BLN	Pt 02	FSAR 09	09.02.05.02.01	COLA Part 2, FSAR Chapter 9, Subsection 9.2.5.2.1 will be revised from: This water supply meets or exceeds the pressure, capacity, and quality requirements in DCD subsection 9.2.5.	BLN-VOL-LTR-006 response to NRC Request

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					To read (retaining LMA of BLN COL 9.2-1): This water supply meets or exceeds the quality requirements in DCD Subsection 9.2.5.1 and the capacity and pressure requirements in DCD Subsection 9.2.5.1.2.	
5199	BLN	Pt 02	FSAR 09	09.02.06	COLA Part 2, FSAR Chapter 9, Subsection 9.2.6 will be revised to read (each item with LMA BLN SUP 9.2-1): 9.2.6.2.1 General Description Replace the final paragraph at the end of DCD Subsection 9.2.6.2.1. There is no on-site waste treatment. The waste treatment plant is the local municipal waste treatment plant. Waste is transferred there through the municipal sewer system. 9.2.6.2.2 Component Description Replace the text under Trunk Line in DCD Subsection 9.2.6.2.2 (to remove the reference to the "site" treatment plant) with: The trunk line is the primary line that the sanitary drainage system piping connects into for transport of the sanitary drainage to the treatment plant. Replace the last sentence under Manholes in DCD Subsection 9.2.6.2.2 (to remove the reference to the "site specific") with: Quantity and locations of the manholes are determined by these criteria. Replace the last sentence under Lift Stations in DCD Subsection 9.2.6.2.2 (to remove the reference to the "site specific") with: Quantity and locations of the lift stations are determined by these criteria. 9.2.6.4 Test and Inspection Replace the paragraph in DCD Subsection 9.2.6.4 (to remove the reference to the "site" specific governing codes) with: The sanitary drainage system is tested by water or air and established to be watertight in accordance with the 2006 International Plumbing Code. System inspection is performed in compliance with the 2006 International Plumbing Code. 9.2.6.5 Instrument Application Replace the text under DCD Subsection 9.2.6.5 (to remove the reference to the "site" treatment plant) with: Sufficient instrumentation for operation is provided to monitor and control the transfer to the treatment plant.	BLN-VOL-LTR-006 response to NRC Request
2221	BLN	Pt 02	FSAR 09	09.02.08.02.03	COLA Part 2, FSAR, Chapter 9, Section 9.2.8.2.3, will be revised To read: The turbine building closed cooling water system is placed in operation during the plant startup sequence after cooling water flow from the CWS, or RWS when applicable, is established but prior to the operation of systems that require turbine building closed cooling water flow. The system is filled by the demineralized water transfer and storage system through a fill line to the surge tank. The system is placed in operation by starting one of the pumps.	RAI LTR 134 response to RAI 09.02.02-01

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4836	BLN	Pt 02	FSAR 09	09.02.11	1. COLA Part 2, FSAR Chapter 9, Subsection 9.2.11 will be revised as shown in response to RAI 09.02.01-06. {General rewrite}	RAI LTR 144 response to RAI 09.02.01-006 item 1
5285	BLN	Pt 02	FSAR 09	09.02.11	3. COLA Part 2, FSAR Chapter 9, Section 9.2.11, will be revised from (as previously revised in response to BLN-RAI-LTR-144): The RWS provides raw strained river water from the Guntersville Reservoir for makeup to the circulating water system (CWS) natural draft cooling tower basins and reservoir filtered water to the Standby Service Water mechanical draft cooling tower basins and to the demineralized water treatment system (DTS). The RWS also provides an alternate supply of filtered reservoir… To read: The RWS provides raw strained river water from the Guntersville Reservoir for makeup to the circulating water system (CWS) natural draft cooling tower basins and reservoir treated water to the Standby Service Water mechanical draft cooling tower basins and to the demineralized water treatment system (DTS). The RWS also provides an alternate supply of treated reservoir...	RAI LTR 156 response to RAI 01-014 item 3
2228	BLN	Pt 02	FSAR 09	09.02.11.01	COLA Part 2, FSAR. Chapter 9, Section 9.2.11.1, second paragraph will be revised from: Failure of the RWS or its components does not affect the ability of safety-related systems to perform their intended function. To read: Failure of the RWS or its components does not affect the ability of safety-related systems to perform their intended function. Potential flooding due to failure of the RWS is bounded by the failure of the interfacing systems analyzed in the DCD and the FSAR and does not result in detrimental effects on SSCs important to safety.	RAI LTR 103 S1 response to RAI 01-09
5287	BLN	Pt 02	FSAR 09	09.02.11.01.02.01	5. COLA Part 2, FSAR Chapter 9, Section 9.2.11.1.2.1 will be revised from (as previously revised in response to BLN-RAI-LTR-144): * Piping to provide an alternate makeup supply of filtered river water to the FPS primary and secondary fire water storage tanks. To read: * Piping to provide an alternate makeup supply of treated river water to the FPS primary and secondary fire water storage tanks.	RAI LTR 156 response to RAI 01-014 item 5
5286	BLN	Pt 02	FSAR 09	09.02.11.01.02.01	4. COLA Part 2, FSAR Chapter 9, Section 9.2.11.1.2.1 will be revised from (as previously revised in response to BLN-RAI-LTR-144): The ancillary RWS pumps provide a continuous supply of filtered river water… To read: The ancillary RWS pumps provide a continuous supply of treated river water…	RAI LTR 156 response to RAI 01-014 item 4
5839	BLN	Pt 02	FSAR 09	09.02.11.01.02.02	COLA Part 2, FSAR Chapter 9, Subsection 9.2.11.1.2.2 will be revised to remove the "in" from the final phrase of the subsection to read "...when the CWS is not available."	Editorial
5288	BLN	Pt 02	FSAR 09	09.02.11.02	5. COLA Part 2, FSAR Chapter 9, Section 9.2.11.2 will be revised from (as previously revised in response to BLN-RAI-LTR-144): The flow path for the functions described in the power generation design basis is from the Guntersville Reservoir, through trash rakes, intake screens and into the basins where the water is available for distribution. The RWS pumps discharge through strainers into a common distribution header for each unit. The ancillary RWS pumps discharge into a common header to a multi-unit	RAI LTR 156 response to RAI 01-014 item 5

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					media filter to a distribution header for each unit. A bypass is provided for the multi-media filter. The RWS provides a piping connection to the municipal water supply for filling and makeup to the primary and secondary fire water storage tanks. A normally closed connection provides a backup supply for filling the fire water storage tanks with raw filtered river water by the ancillary RWS pumps. To read: The flow path for the functions described in the power generation design basis is from the Guntersville Reservoir, through trash rakes, intake screens and into the basins where the water is available for distribution. The RWS pumps discharge through strainers into a common distribution header for each unit. The ancillary RWS pumps discharge into a common header to a multi-unit media filter through an ultraviolet (UV)/oxidation subsystem and activated charcoal bed to a distribution header for each unit. A bypass is provided for the multi-media filter, UV/oxidation subsystem, and activated charcoal bed. The RWS water is treated, as necessary, to provide source water of suitable quality to the Demineralized Water Treatment System and the Standby Service Water System. This water has suspended solids less than 1000 ppb and a pH between 5.8 to 7.5. Additionally, the RWS provides strained water for makeup to the Circulating Water System. The RWS provides a piping connection to the municipal water supply for filling and makeup to the primary and secondary fire water storage tanks. A normally closed connection provides a backup supply for filling the fire water storage tanks with raw filtered river water by the ancillary RWS pumps.	
5289	BLN	Pt 02	FSAR 09	09.02.11.02.01	6. COLA Part 2, FSAR Chapter 9, Section 9.2.11.2.1 for Granular Media Filters will be revised from (as previously revised in response to BLN-RAI-LTR-144): A Multi-unit media filter is located upstream…. To read: Multiple granular media filter units are located upstream…	RAI LTR 156 response to RAI 01-014 item 6
5290	BLN	Pt 02	FSAR 09	09.02.11.02.01	7. COLA Part 2, FSAR Chapter 9, Section 9.2.11.2.1 after Granular Media Filters add the following new component descriptions: Ultraviolet (UV)/Oxidation Subsystem In-line ultraviolet light sources are located downstream of the granular media filters where low turbidity conditions exist to achieve highly effective UV irradiation of bacteria. This UV light treatment is augmented with the use of hydrogen peroxide to further assist in elimination of bacteriological material, and also to eradicate any larval stage Zebra mussel clams and other biota which may be present in the raw water. Activated Charcoal Beds Activated charcoal filters are located downstream of the UV/oxidation subsystem in order to remove any organic compounds from the raw water. In addition, activated charcoal reduces the levels of residual peroxide. The charcoal filters are periodically backwashed and the wash water discharged to the reservoir.	RAI LTR 156 response to RAI 01-014 item 7
5949	BLN	Pt 02	FSAR 09	09.02.11.03	COLA Part 2, FSAR Chapter 9, Section 9.2.11.3, will be revised from: 9.2.11.3 System Operation The RWS pumps are used to fill the CWS cooling tower basin and the ancillary RWS pumps fill the SWS cooling tower basins following an outage, if required. To read: The RWS pumps are used to fill the CWS cooling tower basin, and the ancillary RWS pumps fill the	Editorial

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					SWS cooling tower basins following an outage, if required.	
5930	BLN	Pt 02	FSAR 09	09.02.11.05	COLA Part 2, FSAR Chapter 9, Section 9.2.11.5, will be revised from: "Preventive maintenance requirements are established. Vendor information, industry and system operating experience considered in determining testing requirements." To read: "Vendor information, with consideration of industry and actual system operating experience, is used to determine preventive maintenance testing requirements."	Editorial
5291	BLN	Pt 02	FSAR 09	09.02F / F9.2-201	8. COLA Part 2, FSAR Chapter 9, Section 9.2, Figure 9.2-201 will be revised to show the UV/Oxidation subsystem and the activated charcoal filters. (Actual figure will be provided with future COLA amendment.)	RAI LTR 156 response to RAI 01-014 item 8
4837	BLN	Pt 02	FSAR 09	09.02F / F9.2.1-201 (Sh1 of 2)	2. COLA Part 2, FSAR Chapter 9, Figure 9.2.1-201 (Sheet 1 of 2) will be replaced with the figure provided in Attachment 09.02.01-06B	SUPERSEDED by Qb 5291 - RAI LTR 144 response to RAI 09.02.01-006 item 2
5146	BLN	Pt 02	FSAR 09	09.05.01.08.02.02.01	COLA Part 2, Subsection 9.5.1.8.2.2.1, item d, last sub-bullet, add a period at the end of "record files"	Editorial for consistency
2145	BLN,STD	Pt 02	FSAR 09	09.05.01.08.06	1- COLA Part 2, FSAR Chapter 9 Subsection 9.5.1.8.6 will be revised To read: 9.5.1.8.6 Testing and Inspection Testing and inspection requirements are imposed through administrative procedures. Maintenance or modifications to the fire protection system are subject to inspection for conformation to design requirements. Procedures governing the inspection, testing, and maintenance of fire protection alarm and detection systems, and water-based suppression and supply systems, utilize the guidance of NFPA 72 (DCD Reference 9.5.5.2) and NFPA 25 (Reference 212). Installation of portions of the system where performance cannot be verified through pre-operational tests, such as penetration seals, fire retardant coatings, cable routing, and fire barriers are inspected. Inspections are performed by individuals knowledgeable of fire protection design and installation requirements. Open flame or combustiongenerated smoke is not used for leak testing or similar procedures such as air flow determination. Inspection and testing procedures address the identification of items to be tested or inspected, responsible organizations for the activity, acceptance criteria, documentation requirements and sign-off requirements.	RAI LTR 128 response to RAI 09.05.01-16 item 1
2146	BLN,STD	Pt 02	FSAR 09	09.05.05 / REF 212	2- COLA Part 2, FSAR Chapter 9, Subsection 9.5.5 will be revised to add a new Reference 212 as follows: 212. National Fire Protection Association, "Standard for the Inspection, Testing, and Maintenance of Water-Based Fire Protection Systems," NFPA 25, 2008.	RAI LTR 128 response to RAI 09.05.01-16 item 2
4928	BLN	Pt 02	FSAR 09	09A.02.01	COLA Part 2, FSAR Chapter 9, Appendix 9A, Section 9A.2.1 will be added with an LMA of BLN DEP 18.8-1, to read: 9A.2.1 Fire Area Description ---------- Add the following information at the end of the first paragraph of DCD Subsection 9A.2.1. Figure 9A-201 replaces DCD Figure 9A-3 (Sheet 1) to reflect the relocation of the Operations Support Center. ----------	Consistency - Revised figures were not identified in the FSAR text
5080	BLN,STD	Pt 02	FSAR 10	10.01.03.01	1. In Revision 1, the COLA Part 2, FSAR Subsection 10.1.3.1, last sentence of the paragraph was revised from: In addition, the FAC monitoring program considers the information of Generic Letter 89-08 and industry guidelines.	SUPERSEDED by Qb 5701 - RAI LTR 018 S1 response to RAI 10.03.06-002 item 1 SER with Open Items

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					To read: In addition, the FAC monitoring program considers the information of Generic Letter 89-08, EPRI NSAC-202L-R3, and industry operating experience. The program requires a grid layout for obtaining consistent pipe thickness measurements when using Ultrasonic Test Techniques. The FAC program obtains actual thickness measurements for highly susceptible FAC locations for new lines as defined in EPRI NSAC-202L-R3. At a minimum, a Pass 1 Analysis is used for low susceptible FAC locations and a Pass 2 Analysis for highly susceptible FAC locations will be considered. To determine wear of piping and components where operating conditions are inconsistent or unknown the guidance provided in EPRI NSAC-202L is used to determine wear rates. In a future revision, the above revised material will be further revised from: At a minimum, a Pass 1 Analysis is used for low susceptible FAC locations and a Pass 2 Analysis for highly susceptible FAC locations will be considered. To read: At a minimum, a CHECWORKS type Pass 1 Analysis is used for low susceptible FAC locations and a CHECWORKS type Pass 2 Analysis for highly susceptible FAC locations will be considered.	Confirmatory Item 10.1-1
5701	BLN,STD	Pt 02	FSAR 10	10.01.03.01	1. In Revision 1, the COLA Part 2, FSAR Subsection 10.1.3.1, last sentence of the paragraph was revised from: In addition, the FAC monitoring program considers the information of Generic Letter 89-08 and industry guidelines. To read: In addition, the FAC monitoring program considers the information of Generic Letter 89-08, EPRI NSAC-202L-R3, and industry operating experience. The program requires a grid layout for obtaining consistent pipe thickness measurements when using Ultrasonic Test Techniques. The FAC program obtains actual thickness measurements for highly susceptible FAC locations for new lines as defined in EPRI NSAC-202L-R3. At a minimum, a Pass 1 analysis is used for low and highly susceptible FAC locations and a Pass 2 analysis is used for highly susceptible FAC locations when the Pass 1 analysis results warrant. To determine wear of piping and components where operating conditions are inconsistent or unknown, the guidance provided in EPRI NSAC-202L is used to determine wear rates. In a future revision, the above revised material will be further revised from: The FAC program obtains actual thickness measurements for highly susceptible FAC locations for new lines as defined in EPRI NSAC-202L-R3. At a minimum, a Pass 1 analysis is used for low and highly susceptible FAC locations and a Pass 2 analysis is used for highly susceptible FAC locations when the Pass 1 analysis results warrant. To read: The FAC program obtains actual thickness measurements for highly susceptible FAC locations for new lines as defined in EPRI NSAC-202L-R3 (Reference 201). At a minimum, a CHECWORKS type Pass 1 analysis is used for low and highly susceptible FAC locations and a CHECWORKS type Pass 2 analysis is used for highly susceptible FAC locations when Pass 1 analysis results warrant.	RAI LTR 18 S2 response to RAI 10.03.06-002 item 1 SER with Open Items Confirmatory Item 10.1-1
5081	BLN,STD	Pt 02	FSAR 10	10.01.04	2. COLA Part 2, FSAR Section 10.1, will be further revised to include a new Subsection 10.1.4, References, following Subsection 10.1.3: Add the following after DCD Subsection 10.1.3:	SUPERSEDED by Qb 5702 - RAI LTR 018 S1 response to RAI 10.03.06-002 item 2

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					10.1.4 References 201. EPRI NSAC-202L-R3, Recommendations for an Effective Flow-Accelerated Corrosion Program (NSAC-202L-R3), Electric Power Research Institute (EPRI) Technical Report 1011838, Palo Alto, CA, 2006.	SER with Open Items Confirmatory Item 10.1-1
5702	BLN,STD	Pt 02	FSAR 10	10.01.04	2. COLA Part 2, FSAR Section 10.1, will be further revised to include a new Subsection 10.1.4, References, following Subsection 10.1.3: Add the following after DCD Subsection 10.1.3: 10.1.4 References 201. EPRI NSAC-202L-R3, Recommendations for an Effective Flow-Accelerated Corrosion Program (NSAC-202L-R3), Electric Power Research Institute (EPRI) Technical Report 1011838, Palo Alto, CA, 2006.	RAI LTR 18 S2 response to RAI 10.03.06-002 item 2 SER with Open Items Confirmatory Item 10.1-1
4898	BLN,STD	Pt 02	FSAR 10	10.02.02	COLA Part 2, FSAR Chapter 10, Subsection 10.2.2, will be revised to add a separator bar between STD SUP 10.2-1 text and the STD SUP 10.2-4 text.	Editiorial
4899	BLN,STD	Pt 02	FSAR 10	10.03.02.02.01	COLA Part 2, FSAR Chapter 10, Subsection 10.3.2.2.1, will be revised to remove the "will" from the sentence beginning "Operations and maintenance procedures will include precautions,..."	Editorial
3974	BLN,STD	Pt 02	FSAR 10	10.04.05.02.02	4. COLA Part 2, FSAR Chapter 10, Subsection 10.4.5.2.2, will be revised under "Circulating Water Chemical Injection" from: • Silt Dispersant – Polyacrylate To read: • Silt Dispersant – Polymeric silt dispersant	RAI LTR 137 response to 02.02.03-10, item 4
5042	BLN	Pt 02	FSAR 10	10.04.07.02.01	Revise COLA Part 2, FSAR Chapter 10, Subsections 10.4.7.2.1 reference to carbohydride to reference carbohydrazide.	Editorial, Based on NRC comment 20090529 SER with Open Items agreement per SER page 10-26
4900	BLN,STD	Pt 02	FSAR 10	10.04.07.02.01	COLA Part 2, FSAR Chapter 10, Subsection 10.4.7.2.1, will be revised to remove the "will" from the sentence beginning "Operations and maintenance procedures will include precautions,..."	Editorial
2281	BLN,STD	Pt 02	FSAR 11	11.02.01.02.04	3. COLA Part 2, FSAR Chapter 11 will be revised to change last paragraph of FSAR (i.e., DCD) Subsection 11.2.1.2.4 To read: The monitored radwaste discharge pipeline is engineered to preclude leakage to the environment. This pipe is routed from the auxiliary building to the radwaste building (the short section of pipe between the two buildings is fully available for visual inspection as noted above) and then out of the radwaste building to the licensed release point for dilution and discharge. The discharge radiation monitor and isolation valve are located inside the auxiliary building. The exterior piping is designed to preclude inadvertent or unidentified releases to the environment. No valves, vacuum breakers, or other fittings are incorporated outside of buildings. This greatly reduces the potential for undetected leakage from this discharge to the environment at a non-licensed release point, and supports compliance with 10 CFR 20.1406 (Reference 5). BLN SUP 11.2-1 The exterior radwaste discharge piping is enclosed within a guard pipe and monitored for leakage. Liquid radwaste effluent will be discharged to the Tennessee River (Guntersville Reservoir) with cooling tower blowdown. The cooling tower blowdown line is sealed and monitored for leakage.	RAI LTR 109 S1 response to RAI 12.03-12.04-01, item 3 SER with Open Items Confirmatory Item 12.3-1

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5633	BLN,STD	Pt 02	FSAR 11	11.02.01.02.04	COLA Part 2, FSAR Chapter 11will be revised to include the following lead-in sentence prior to the change for the last paragraph of FSAR (i.e., DCD) Subsection 11.2.1.2.4 identified in Qb 2281. Replace the last paragraph in DCD Subsection 11.2.1.2.4 with the following information:	Editorial supplement for consistent incorporaton of RAI LTR 109 S1 response to RAI 12.03-12.04-01, item 3 SER with Open Items Confirmatory Item 12.3-1
5292	BLN	Pt 02	FSAR 11	11.02.03	9. COLA Part 2, FSAR Chapter 11, Section 11.2, will be revised to add the following subsection with LMA of BLN SUP 11.2-2: 11.2.3 Radioactive Releases Add the following new paragraph at the end of DCD Subsection 11.2.3: The only liquid effluent site interface parameter outside of the Westinghouse scope is the release point to the Guntersville Reservoir.	RAI LTR 156 response to RAI 01-014 item 9
5964	BLN	Pt 02	FSAR 11	11.02.03.05.02	COLA Part 2, FSAR Chapter 11, Subsection 11.2.3.5.2, will be revised from: "The annual usage for each of these activities is assumed to be 2.9E+08 person-hours." To read: "The annual usage for each of these activities is assumed to be 2.3E+07 person-hours."	Editorial
5293	BLN,STD	Pt 02	FSAR 11	11.03	10. COLA Part 2, FSAR Chapter 11, Section 11.3, will be revised to add the following subsection with LMA of STD SUP 11.3-2: 11.3.3 Radioactive Releases Add the following new paragraph at the end of DCD Subsection 11.3.3: There are no gaseous effluent site interface parameters outside of the Westinghouse scope.	RAI LTR 156 response to RAI 01-014 item 10
5381	BLN	Pt 02	FSAR 11	11.04.02.04.03	COLA Part 2, FSAR Chapter 11, Subsection 11.4.2.4 will be revised to add a new subsection with the LMA of STD COL 11.4-2 to read: Add the following after DCD Subsection 11.4.2.4.2: 11.4.2.4.3 Alternatives for B and C Wastes It is expected that Class B and C wastes will constitute approximately 5 percent by volume of the low level radioactive waste (LLRW) that will be generated by the plant with the balance being Class A waste. The volume of wet Class B and C waste is approximately 100 percent of the total Class B and C Waste. As of July 1, 2008, the LLRW disposal facility in Barnwell, South Carolina is no longer accepting Class B and C waste from sources in states that are outside of the Atlantic Compact. However, the disposal facility in Clive, Utah is still accepting Class A waste from out of state. Should there be no disposal facilities that will accept the Class B and C wastes after the plant begins operation, there are several options available for storage of such waste: • As provided in referenced DCD Subsection 11.4.2., the Auxiliary Building is designed to have more than a year of spent resin storage capacity at the expected rate and the spent resin tanks may be mixed to limit the radioactivity concentrations thereby limiting the volume of Class B and C wet waste requiring storage. • Vendor services are available to process Class A, B, and C waste and transfer for storage that material until a disposal site is available. Currently, Waste Control Specialists (WCS) of Texas is available to store Class A, B, and C material pending the availability of a licensed disposal site.	COL-SER-OI-Ch11 response to OI 11.04-01 and RAI LTR 163 response to RAI 11.04-01

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					• If additional storage capacity were eventually needed, the plant could construct or expand storage facilities onsite or gain access to a storage facility at another licensed nuclear plant.	
5594	BLN	Pt 02	FSAR 11	11.04.02.04.03	COLA Part 2, FSAR Chapter 11, Subsection 11.4.2.4.3 (as added by SER Chapter 11 Open Item response) will be revised to remove the initialism "(LLRW)" and to replace the use of LLRW with "low level radioactive waste" and to remove the initialism "(WCS)" since these are not used in any other locations.	Editorial
5606	BLN	Pt 02	FSAR 11	11.04.02.04.03	COLA Part 2, FSAR Chapter 11, Subsection 11.4.2.4.3 (as added by SER Chapter 11 Open Item response) will be revised to correct the LMA from COL 11.4-2 to COL 11.4-1.	LMA identification correction Revision to COL-SER-OI-Ch11 response to OI 11.04-01 and RAI LTR 163 response to RAI 11.04-01
5737	BLN	Pt 02	FSAR 11	11.04.02.04.03	COLA Part 2, FSAR Chapter 11, Subsection 11.4.2.4 will be revised from new LMA of STD COL to BLN COL.	This response is PLANT-SPECIFIC. Revision to COL-SER-OI-Ch11 response to OI 11.04-01 and RAI LTR 163 response to RAI 11.04-01
4946	BLN,STD	Pt 02	FSAR 11	11.04.06	1. COLA Part 2, FSAR Chapter 11, Subsection 11.4.6, will be revised to read: A Process Control Program (PCP) is developed and implemented in accordance with the recommendations and guidance of NEI 07-10A (Reference 201). The PCP describes the administrative and operational controls used for the solidification of liquid or wet solid waste and the dewatering of wet solid waste.	BLN-VOL-LTR-003 response to NEI 07-10 item 1 SER with Open Items Confirmatory Item 11.4-1
4947	BLN,STD	Pt 02	FSAR 11	11.04.07	2. COLA Part 2, FSAR Chapter 11, Subsection 11.4.7, will be revised to read: 201. NEI 07-10A, "Generic FSAR Template Guidance for Process Control Program (PCP)," Revision 0, March 2009.	BLN-VOL-LTR-003 response to NEI 07-10 item 2 SER with Open Items Confirmatory Item 11.4-1
4944	BLN,STD	Pt 02	FSAR 11	11.05.07	1. COLA Part 2, FSAR Chapter 11, Subsection 11.5.7, will be revised to read: An Offsite Dose Calculation Manual (ODCM) is developed and implemented in accordance with the recommendations and guidance of NEI 07-09A (Reference 202). The ODCM contains the methodology and parameters used for calculating doses resulting from liquid and gaseous effluents. The ODCM addresses operational setpoints, including planned discharge rates, for radiation monitors and monitoring programs (process and effluent monitoring and environmental monitoring) for the control and assessment of the release of radioactive material to the environment. The ODCM provides the limitations on operation of the radwaste systems, including functional capability of monitoring instruments, concentrations of effluents, sampling, analysis, 10 CFR Part 50, Appendix I dose and dose commitments, and reporting. The ODCM will be finalized prior to fuel load with site-specific information.	BLN-VOL-LTR-003 response to NEI 07-09 item 1 SER with Open Items Confirmatory Item 11.5-1
4945	BLN,STD	Pt 02	FSAR 11	11.05.08	2. COLA Part 2, FSAR Chapter 11, Subsection 11.5.8, will be revised to read: 202. NEI 07-09A, "Generic FSAR Template Guidance for Offsite Dose Calculation Manual (ODCM) Program Description," Revision 0, March 2009.	BLN-VOL-LTR-003 response to NEI 07-09 item 2 SER with Open Items Confirmatory Item 11.5-1
6074	BLN	Pt 02	FSAR 12	12.01	5. COLA Part 2, FSAR Chapter 12, Section 12.1, will be revised to read:	COL-SER-OI-Ch12 S1 response to OI 12.01-001

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					This section incorporates by reference NEI 07-08A, Generic FSAR Template Guidance for Ensuring That Occupational Radiation Exposures Are As Low As Is Reasonably Achievable (ALARA), Revision 0. See Table 1.6-201. ALARA practices are developed in a phased milestone approach as part of the procedures necessary to support the Radiation Protection Program. Table 13.4-201 describes the major milestones for ALARA procedures development and implementation.	item 5 SNC Letter ND-09-1770
6075	BLN	Pt 02	FSAR 12	12.01	6. COL Part 2 FSAR Chapter 12, Section 12.1, will be revised to add new text (with an LMA of STD COL 12.1-1) that reads: Revise the last sentence of NEI 07-08A Subsection 12.1.2 to read: ALARA procedures are established, implemented, maintained and reviewed consistent with 10 CFR 20.1101 and the quality assurance criteria described in Part III of the Quality Assurance Program Description, which is discussed in Section 17.5.	COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 6 SNC Letter ND-09-1770
6076	BLN	Pt 02	FSAR 12	12.01.03	7. COL Part 2 FSAR Chapter 12, Section 12.1.3, will be revised to read: This COL item is addressed in NEI 07-08A and Appendix 12AA.	COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 7 SNC Letter ND-09-1770
4930	BLN,STD	Pt 02	FSAR 12	12.03.01.02	COLA Part 2, FSAR Chapter 12, Subsection 12.3.1.2 will be added with an LMA of BLN DEP 18.8-1, to read: 12.3.1.2 Radiation Zoning and Access Control ---------- Add the following information at the end of the second paragraph of DCD Subsection 12.3.1.2. Figure 12.3-201, Figure 12.3-202, and Figure 12.3-203 replace DCD Figure 12.3-1 (Sheet 11), DCD Figure 12.3-2 (Sheet 11), and DCD Figure 12.3-3 (Sheet 11), respectively, to reflect the relocation of the Operations Support Center. ----------	Editorial
5938	BLN	Pt 02	FSAR 12	12.03.05.01	7. COLA Part 2, FSAR Chapter 12, Section 12.3.5.1, will be revised to read: This COL item is addressed in Subsection 12.5.4 and Appendix 12AA.	COL-SER-CI-Ch12 response to CI 12.01.01 item 7 SNC Letter #ND-09-1529
5309	BLN	Pt 02	FSAR 12	12.04.01.09	COLA Part 2, FSAR Chapter 12, Subsection 12.4.1.9, will be revised to include the following new subsection (12.4.1.9.4.5 for VEGP) at the end of the section with a Left Margin Annotation (LMA) of STD SUP 12.4-1: STD SUP 12.4-1 12.4.1.9.x Operating Unit Radiological Surveys The operating unit conducts radiological surveys in the unrestricted and controlled area and radiological surveys for radioactive materials in effluents discharged to unrestricted and controlled areas in implementing 10 CFR 20.1302. These surveys demonstrate compliance with the dose limits of 10 CFR 20.1301 for construction workers.	COL-SER-OI-Ch12 response to OI 12.04-01
5939	BLN	Pt 02	FSAR 12	12.05	8. COLA Part 2, FSAR Chapter 12, Section 12.5, will be revised to add new text after Section 12.5.2.2 (with an LMA of STD COL 12.3-1) that reads: 12.5.4 Controlling Access and Stay Time Add the following text to the end of DCD Subsection 12.5.4. STD COL 12.3-1 A closed circuit television system may be installed in high radiation areas to allow remote monitoring of individuals	COL-SER-CI-Ch12 response to CI 12.01.01 item 8 SNC Letter #ND-09-1529

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					entering high radiation areas by personnel qualified in radiation protection procedures.	
5941	BLN	Pt 02	FSAR 12	12AA	9. COLA Part 2, FSAR Chapter 12, Appendix 12AA will be revised in its entirety as shown in Attachment 12.01-01A of COL-SER-CI-CH12 (SNC Letter #ND-09-1529). The changes include those provided in the response to BLN-RAI-LTR-142 (NRC RAI Number 01-11, ADAMS ML083510576).	COL-SER-CI-Ch12 response to CI 12.01.01 item 9 SNC Letter #ND-09-1529
6077	BLN	Pt 02	FSAR 12	12AA	1. COLA Part 2, FSAR Chapter 12, Appendix 12AA, text after the last bullet of NEI 07-03 Subsection 12.5.4.8 will be revised to read: This subsection adopts NEI 08-08A (Reference 201), for a description of the operational and programmatic elements and controls that minimize contamination of the facility, site, and the environment, to meet the requirements of 10 CFR 20.1406.	COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 1 SNC Letter #ND-09-1770
6078	BLN	Pt 02	FSAR 12	12AA.05.04.14	2. COL Part 2 FSAR Chapter 12, Appendix 12AA 5.4.14 last paragraph, will be revised to read: This subsection adopts NEI 08-08A (Reference 201) for the Groundwater Monitoring Program description.	COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 2 SNC Letter #ND-09-1770
6079	BLN	Pt 02	FSAR 12	12AA.05.04.15	3. COL Part 2 FSAR Chapter 12, Appendix 12AA.5.4.15, will be revised to read: This subsection adopts NEI 08-08A (Reference 201) for discussion of record keeping practices important to decommissioning.	COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 3 SNC Letter #ND-09-1770
3478	BLN,STD	Pt 02	FSAR 12	12AA.Ref	5. COLA Part 2, FSAR Chapter 12, Appendix 12AA, Revision 1, will be revised to include the following additional statements: Revise the References section, Reference 8, to read as follows: 8. Regulatory Guide 1.97, Revision 3, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident."	RAI LTR 142 response to RAI 01-11, item 5
6080	BLN	Pt 02	FSAR 12	12AA.Ref	4. COL Part 2 FSAR Chapter 12, Appendix 12AA reference to NEI 07-03 References will be revised to read: 201. NEI 08-08A, Generic FSAR Template Guidance for Life Cycle Minimization of Contamination, Revision 0, October 2009.	COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 4 SNC Letter #ND-09-1770
5942	BLN	Pt 02	FSAR 12	12AA.T / T12AA-201	10. COLA Part 2, FSAR Chapter 12, Appendix 12AA, add new Table 12AA-201 (with an LMA of STD COL 12.3-1) as shown in Attachment 12.01-01B of COL-SER-CI-CH12 (SNC Letter #ND-09-1529).	COL-SER-CI-Ch12 response to CI 12.01.01 item 10 SNC Letter #ND-09-1529
5073	BLN,STD	Pt 02	FSAR 13	13.02	4. COLA Part 2, FSAR Chapter 13, Section 13.2, will be revised to read: This section incorporates by reference NEI 06-13A, Template for an Industry Training Program Description.	BLN-VOL-LTR-004 response to NEI 06-13 item 4
5349	BLN	Pt 02	FSAR 13	13.04.T / T13.4-201 08	1. COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, item 8, Fire Protection Program, will be revised (to add the following new milestone): (portions applicable to SNM) 10 CFR 30.32 10 CFR 40.31 Prior to initial receipt of byproduct source, or special nuclear materials (excluding Exempt Quantities as 10 CFR 30.32(a) 10 CFR 40.31(a)	COL-SER-OI-Ch01 response to OI 01.05-01 item 1

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					described in 10 CFR 30.18)	
2282	BLN,STD	Pt 02	FSAR 13	13.04.T / T13.4-201 10	1. COLA Part 2, FSAR. Chapter 13, Table 13.4-201, Item 10, will be revised	RAI LTR 109 S1 response to RAI 12.05-02 SER with Open Items Confirmatory Item 12.5-1
6081	BLN	Pt 02	FSAR 13	13.04.T / T13.4-201 10	5. COL Part 2 FSAR Chapter 13, Table 13.4-201 (Sheet 3 of 7) Item 10 will be revised to add a reference to 10 CFR 20.1406 to the Program Source (Required by) column.	COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 5 SNC Letter #ND-09-1770
6082	BLN	Pt 02	FSAR 13	13.04.T / T13.4-201 10	6. COL Part 2 FSAR Chapter 13, Table 13.4-201 (Sheet 3 of 7) Item 10 will be revised to add a new sub-bullet "· Minimization of Contamination" to the Program Title column.	COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 6 SNC Letter #ND-09-1770
5350	BLN	Pt 02	FSAR 13	13.04.T / T13.4-201 11	2. COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, item 11, Non Licensed Plant Staff Training Program, will be revised (to add the following new milestone): (portions applicable to SNM) 10 CFR 30.32 10 CFR 40.31 Prior to initial receipt of byproduct source, or special nuclear materials (excluding Exempt Quantities as described in 10 CFR 30.18) 10 CFR 30.32(a) 10 CFR 40.31(a)	COL-SER-OI-Ch01 response to OI 01.05-01 item 2
5352	BLN	Pt 02	FSAR 13	13.04.T / T13.4-201 14	3. COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, item 14, Emergency Planning, will be revised (to add the following new milestone): (portions applicable to SNM) 10 CFR 30.32 10 CFR 40.31 Prior to initial receipt of byproduct source, or special nuclear materials (excluding Exempt Quantities as described in 10 CFR 30.18) 10 CFR 30.32(a) 10 CFR 40.31(a)	COL-SER-OI-Ch01 response to OI 01.05-01 item 3
5880	BLN	Pt 02	FSAR 13	13.04.T / T13.4-201 14	COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, items 8, 11, 14 and 15, will be revised to add a comma after "byproduct" in the milestone "Prior to initial receipt of byproduct..."	Editorial revision to COL-SER-OI-Ch01 response to OI 01.05-01 item 1
4991	BLN,STD	Pt 02	FSAR 13	13.04.T / T13.4-201 15	COLA Part 2, FSAR. Chapter 13, Table 13.4-201, Item 15, will be revised to move FFD to new line item along with new Cyber Security Program, and modify implementation milestones for Security Program items and FFD items. Revised text: 15. Security Program: 10 CFR 50.34(c); Physical Security Program 10 CFR 73.55; 10 CFR 73.56; 10 CFR 73.57; 13.6 Prior to receipt of fuel onsite (protected area) License Condition Safeguards Contingency Program 10 CFR 50.34(d); 10 CFR Part 73, Appendix C 13.6 Prior to	SUPERSEDED by and Incorporated into Qb 5682 - Changes to address Security Regulation revisions

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					receipt of fuel onsite (protected area) License Condition Training and Qualification Program 10 CFR Part 73, Appendix B 13.6 Prior to receipt of fuel onsite (protected area) License Condition 20. Fitness For Duty Programs: Fitness for Duty Program (Construction - Mgt. & Oversight Personnel) 10 CFR Part 26, Subparts A–H, N, and O 13.7 Prior to initiating onsite construction License Condition Fitness for Duty Program (Construction – Workers & First Line Supv.) 10 CFR Part 26, Subpart K 13.7 Prior to initiating onsite construction License Condition Fitness for Duty Program (Operation) 10 CFR Part 26 13.7 Prior to initial fuel load License Condition 21. Cyber Security Program 10 CFR 73.54 13.6 Prior to initial fuel load License Condition	
5353	BLN	Pt 02	FSAR 13	13.04.T / T13.4-201 15	4. COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, item 15, Security Program, will be revised (to add the following new milestone): (portions applicable to SNM) 10 CFR 30.34 10 CFR 40.41 Prior to initial receipt of byproduct source, or special nuclear materials (excluding Exempt Quantities as described in 10 CFR 30.18) 10 CFR 30.32(a) 10 CFR 40.31(a)	COL-SER-OI-Ch01 response to OI 01.05-01 item 4
5682	BLN	Pt 02	FSAR 13	13.04.T / T13.4-201 15	3. Change COLA Part 2, FSAR, Table 13.4-201, by separating the Fitness for Duty (FFD) Program (Line item 20) from the Security Program (Line item 15), and adding a new Line item 21 for the Cyber Security Program. Table 13.4-201 is also changed to clarify that the Security Program implementation milestone is "Prior to receipt of fuel onsite (protected area)," and the Fitness for Duty Program construction milestone is "Prior to initiating onsite construction," in conformance with the Final Rule related to Power Reactor Security Requirements.	BLN VOL-SEC-CYBER-20090811 item 3
4955	BLN,STD	Pt 02	FSAR 13	13.06	1. Change COLA Part 2, FSAR, Section 13.6, by deleting the second paragraph of STD COL 13.6-1 related to physical security during construction. The paragraph to be deleted currently reads as follows: The Physical Security Plan during construction, including control of access to the new plant construction site, is consistent with NEI 03-12, Appendix F (Reference 201), which is currently under NRC review.	BLN-P02-VOL-SEC-FFD-20090323-OR
4992	BLN,STD	Pt 02	FSAR 13	13.06	COLA Part 2, FSAR. Chapter 13, Section 13.6, will be revised To read (with an added LMA of STD COL 13.6-5 along with the existing STD COL 13.6-1): The Security Plan consists of the Physical Security Plan, the Training and Qualification Plan, and the Safeguards Contingency Plan. The Security Plan is submitted to the Nuclear Regulatory Commission as a separate licensing document in order to fulfill the requirements of 10 CFR 52.79(a)(35) and 52.79(a)(36). The Security Plan meets the requirements contained in 10 CFR Part 73 and will be maintained in accordance with the requirements of 10 CFR 52.98. The Plan is categorized as Security Safeguards Information and is withheld from public disclosure pursuant to 10 CFR 73.21. The Cyber Security Plan is submitted to the Nuclear Regulatory Commission as a separate licensing document in order to fulfill the requirements of 10 CFR 52.79(a)(36). The Cyber Security Plan meets	Changes to address Security Regulation revisions and remove incorrect reference to Part 26 for Security Program. Partially SUPERSEDED by Qb 5680.

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					the requirements contained in 10 CFR 73.54 and will be maintained in accordance with the requirements of 10 CFR 52.98. The Plan is categorized as Security Related Information and is withheld from public disclosure pursuant to 10 CFR 2.390. Table 13.4-201 provides milestones for security program and cyber security program implementation.	
5680	BLN,STD	Pt 02	FSAR 13	13.06	1. Change COLA Part 2, FSAR, Section 13.6, Security, third and fourth paragraphs to read: The Cyber Security Plan is submitted to the Nuclear Regulatory Commission as a separate licensing document to fulfill the requirements contained in 10 CFR 52.79(a)(36) and 10 CFR 73.54. The Cyber Security Plan will be maintained in accordance with the requirements of 10 CFR 52.98. The Plan is withheld from public disclosure pursuant to 10 CFR 2.390. Table 13.4-201 provides milestones for security program and cyber security program implementation.	VOL-SEC-CYBER-20090811 item 1 - SUPERSEDES a portion of Qb 4992
4956	BLN,STD	Pt 02	FSAR 13	13.06.02	2. Change COLA Part 2, FSAR, Section 13.6.2, References, by replacing Reference 201, NEI 03-12, Appendix F, with "Not used."	SUPERSEDED by Qb 5681 - BLN-P02-VOL-SEC-FFD-20090323-OR
4993	BLN,STD	Pt 02	FSAR 13	13.06.02	COLA Part 2, FSAR. Chapter 13, Subsection 13.6.2, will be revised to remove Reference 202.	Changes to address Security Regulation revisions (complements Qb 4992)
5681	BLN	Pt 02	FSAR 13	13.06.02	2. Delete COLA Part 2, FSAR, Subsection 13.6.2, REFERENCES, in its entirety. A change in TVA's letter dated March 23, 2009, deleted Reference 201, NEI 03-12, Appendix F. The change addressed by this letter supplements the change to delete Reference 201 by also deleting Reference 202, NEI 04-04, and consequently deletes the entire subsection.	SUPERSEDED by Qb 5745 - VOL-SEC-CYBER-20090811 item 2
5745	BLN,STD	Pt 02	FSAR 13	13.06.02	Notwithstanding Qb 5681, restore COLA Part 2, FSAR, Section 13.6.2, References, with Reference 201 of "Not used."	References section needed to support DC 13.7 redistribution as identified at top of FSAR Section 13.7.
4948	BLN,STD	Pt 02	FSAR 13	13.07	1. Change COLA Part 2, FSAR, Section 13.7 by revising the first paragraph to reflect the current status of the referenced regulation, 10 CFR Part 26, which was revised subsequent to submittal of the BLN COLA. The discussion of the applicable guidance, NEI 06-06, is also changed by deleting the phrase indicating that the guidance was under NRC review when the BLN COLA was submitted. In addition, the second paragraph is deleted, because the exemption that was required at the time of the BLN COLA submittal is no longer needed. The text in Section 13.7 is changed to read: STD SUP 13.7-1 The Fitness for Duty (FFD) Program is implemented and maintained in two phases; the construction phase program and the operating phase program. The construction and operations phase programs are implemented as identified in Table 13.4-201. The construction phase program is consistent with NEI 06-06 (Reference 201). The workforce population subject to random testing during construction is determined on a weekly basis by averaging the total number of active construction badges over each preceding seven-day period. The random selection from each week's workforce population is identified by a standard computer-generated random number generator using this number of active badges as the range of numbers considered in the weekly random testing selection.	BLN-P02-VOL-SEC-FFD-20090323-OR item 1

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					The operations phase program is consistent with 10 CFR Part 26.	
4949	BLN,STD	Pt 02	FSAR 13	13.07.01	2. Change COLA Part 2, FSAR, Subsection 13.7.1, by changing Reference 201 to read: Nuclear Energy Institute "Fitness for Duty Program Guidance for New Nuclear Power Plant Construction Sites", NEI 06-06, Revision 4, February 2009.	BLN-P02-VOL-SEC-FFD-20090323-OR item 2
2597	BLN,STD	Pt 02	FSAR 14	14.02 - 14.04	COLA Part 2, FSAR. Chapter 14, will be revised as shown in Attachment 14.02-12A of response to RAI 14.02-012. {General rewrite}	RAI LTR 139 response to RAI 14.02-012, item 1 SER with Open Items Confirmatory Item 14.2-1 thru 10 & 12
5774	BLN	Pt 02	FSAR 14	14.02.01	Revise reference to Regulatory Guide 1.206, Part I, Section C.1.14.2 as identified in Qb 2597 to Section C.I.14.2.	Editorial revision to Qb 2597 as identified in RAI LTR 139 S1 response to RAI 14.02-012, item 2
5135	BLN	Pt 02	FSAR 14	14.02.01.04	Add LMA of STD COL 14.4-3 to the additions of 14.2.1.4 and 14.2.1.5 that follow 14.2.1.3 as identified in Qb 2597.	Editorial revision to Qb 2597 as identified in RAI LTR 139 S1 response to RAI 14.02-012, item 2
5721	BLN	Pt 02	FSAR 14	14.02.02.04 - .06	Revise Subsections 14.2.2.4 to remove untitled subsection numbers 14.2.2.4.1 and 14.2.2.4.2. The text remains, only the subsection numbers are removed. Revise Subsections 14.2.2.5 to remove untitled subsection numbers 14.2.2.5.1 and 14.2.2.5.2. The text remains, only the subsection numbers are removed. Revise Subsections 14.2.2.6 to remove untitled subsection numbers 14.2.2.6.1 and 14.2.2.6.2. The text remains, only the subsection numbers are removed.	Editorial revision to Qb 2597 - RAI LTR 139 response to RAI 14.02-012, item 1
4851	BLN,STD	Pt 02	FSAR 14	14.02.03	2. COLA Part 2, FSAR Chapter 14, Subsections 14.2.12 through 14.2.15, as shown in letter 139, will be renumbered and added to Subsection 14.2.3.1, , Conduct of Test Program, as shown below: Add the following Subsections after DCD Subsection 14.2.3.1: 14.2.12 change to 14.2.3.1.1 14.2.13 change to 14.2.3.1.2 14.2.14 change to 14.2.3.1.3 14.2.15 change to 14.2.3.1.4	SUPERSEDED by Qb 4990 - RAI LTR 139 S1 response to RAI 14.02-012, item 2
4990	BLN,STD	Pt 02	FSAR 14	14.02.03	2. COLA Part 2, FSAR Chapter 14, Subsections 14.2.12 through 14.2.15, as shown in letter 139, will be renumbered and added to Subsection 14.2.3.1, , Conduct of Test Program, as shown below: Add the following Subsections after DCD Subsection 14.2.3.1: 14.2.12 changed to 14.2.3.1.1 now change to 14.2.3.1.2 14.2.13 changed to 14.2.3.1.2 now change to 14.2.3.1.3 14.2.14 changed to 14.2.3.1.3 now change to 14.2.3.1.4 14.2.15 changed to 14.2.3.1.4 now change to 14.2.3.1.5 Also correct format for titles of each of the above sections to Initial Cap Only.	RAI LTR 139 S1 response to RAI 14.02-012, item 2 SER with Open Items Confirmatory Item 14.2-4
5749	BLN	Pt 02	FSAR 14	14.02.03	Add LMA of STD COL 14.4-3 to the additions at the end of 14.2.3 as identified in Qb 2597.	Editorial revision to Qb 2597 as identified in RAI LTR 139 S1 response to RAI 14.02-012, item 2
4852	BLN,STD	Pt 02	FSAR 14	14.02.03.02.01	3. COLA Part 2, FSAR Subsection 14.2.3.2.1, fourth paragraph will be changed to read:	RAI LTR 139 S1 response

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					Each area of startup testing is reviewed and evaluated by the PT&O organization and the JTWG. The test results at each power ascension testing power plateau are reviewed and evaluated by the PT&O organization and the JTWG and approved by the plant manager before proceeding to the next plateau. Startup test reports are prepared in accordance with the guidance in position C.1.a of Regulatory Guide 1.16, "Reporting of Operating Information -- Appendix A Technical Specifications" and position C.9 of Regulatory Guide 1.68, "Initial test Programs for Water-Cooled Nuclear Power Plants.	to RAI 14.02-012, item 3 SER with Open Items Confirmatory Item 14.2-5
5643	BLN	Pt 02	FSAR 14	14.02.03.02.01	COLA Part 2, FSAR Subsection 14.2.3.2.1, fourth paragraph will be changed from: Startup test reports are prepared in accordance with the guidance in position C.1.a of Regulatory Guide 1.16, "Reporting of Operating Information -- Appendix A Technical Specifications" and position C.9 of Regulatory Guide 1.68, "Initial test Programs for Water-Cooled Nuclear Power Plants." To read: Startup test reports are prepared in accordance with the guidance in position C.9 of Regulatory Guide 1.68, "Initial test Programs for Water-Cooled Nuclear Power Plants."	Regulatory Guide 1.16 withdrawn by NRC 8-11-2009 via 74 FR 40244. This modifies the change in RAI LTR 139 S1 response to RAI 14.02-012, item 3 SER with Open Items Confirmatory Item 14.2-5
5751	BLN	Pt 02	FSAR 14	14.02.03.03.01	Add LMA of STD COL 14.4-4 to text beginning at Subsection 14.2.3.3.1 as identified in Qb 2597.	Editorial revision to Qb 2597 as identified in RAI LTR 139 S1 response to RAI 14.02-012, item 2
5879	BLN	Pt 02	FSAR 14	14.02.03.03.01	COLA Part 2, FSAR Subsection 14.2.3.3.1, first paragraph will be changed from: A startup report is submitted per Regulatory Guide 1.16 at the earliest of: To read: A startup report is submitted at the earliest of:	Regulatory Guide 1.16 withdrawn by NRC 8-11-2009 via 74 FR 40244. Revision to Qb 2597 - RAI LTR 139 response to RAI 14.02-012, item 1
4850	BLN,STD	Pt 02	FSAR 14	14.02.05.01	1. COLA Part 2, FSAR Chapter 14, Subsection 14.2.11, as shown in letter 139, will be renumbered and added to Subsection 14.2.5, Utilization of Reactor Operating and Testing Experience in the Development of Test Program, as shown below: Add the following Subsections after DCD Subsection 14.2.10.5: 14.2.11 change to new last paragraph of 14.2.5 14.2.11.1 change to 14.2.5.1 14.2.11.2 change to 14.2.5.2 14.2.11.3 change to 14.2.5.3 14.2.11.4 change to 14.2.5.4 14.2.11.5 change to 14.2.5.5	RAI LTR 139 S1 response to RAI 14.02-012, item 1 SER with Open Items Confirmatory Item 14.2-7
6518	BLN,STD	Pt 02	FSAR 14	14.02.05.01	COLA Part 2, FSAR Chapter 14, Subsection 14.2.5, as shown in Revision 1, will be revised to omit the subsection number of 14.2.5 and align the subtitle of "Utilization of Operating Experience," with the left margin.	Editorial
4853	BLN,STD	Pt 02	FSAR 14	14.02.08	4. COLA Part 2, FSAR Chapter 14, Subsection 14.2.8, last paragraph, as shown in letter 139, will be revised to read: The milestone schedule for developing plant operating procedures is presented in Table 13.4-201. The operating and emergency procedures are available prior to start of licensed operator training and, therefore, are available for use during the ITP. Required or desired procedure changes may be identified during their use. Administrative procedures describe the process for revising plant operating procedures.	RAI LTR 139 S1 response to RAI 14.02-012, item 4 SER with Open Items Confirmatory Item 14.2-9
5789	BLN	Pt 02	FSAR 14	14.03.02.03	Under Selection Criteria, fourth sub-bullet under the first main bullet, revise the reference to "DCD Section 16.3" to red text and add hyperlink.	Editorial

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
5094	BLN,STD	Pt 02	FSAR 14	14.03.T / T14.3-201	2. COLA Part 2, Section 14.3, Table 14.3-201 entry for offsite power (with BLN SUP 14.3-2), is revised from: ZBS Transmission Switchyard and Offsite Power System XX To read: ZBS Transmission Switchyard and Offsite Power System XX	See Qb 5755 - RAI LTR 027 S1 response to RAI 14.03-001 item 2 SER with Open Items Confirmatory Item 8.2A-1 SER with Open Items Confirmatory Item 14.3-1
5755	BLN,STD	Pt 02	FSAR 14	14.03.T / T14.3-201	This change should have read: 2. COLA Part 2, Section 14.3, Table 14.3-201 entry for offsite power (with BLN SUP 14.3-2), is revised from: ZBS Transmission Switchyard and Offsite Power System XX(underlined) To read: ZBS Transmission Switchyard and Offsite Power System XX	RAI LTR 027 S1 response to RAI 14.03-001 item 2 SER with Open Items Confirmatory Item 8.2A-1 SER with Open Items Confirmatory Item 14.3-1
5095	BLN,STD	Pt 02	FSAR 14	14.03.T / T14.3-201	3. COLA Part 2, Section 14.3, Table 14.3-201 legend, is revised to add: XX = Selected for ITAAC	RAI LTR 027 S1 response to RAI 14.03-001 item 3 SER with Open Items Confirmatory Item 8.2A-1 SER with Open Items Confirmatory Item 14.3-1
3486	BLN	Pt 02	FSAR 14	14.03.T / T14.3-202	7. COLA Part 2, FSAR Chapter 14, Section 14.3, will be revised to add the following new table with an LMA of BLN DEP 2.3-1. TABLE 14.3-202 BLN RADIOLOGICAL ANALYSIS Reference Design Feature Value Section 2.3.4 BLN Atmospheric dispersion factors – X/Q (sec/m3) - Site Boundary X/Q 0 - 2 hour time interval ≤5.85 x 10-4 Note: This table supplements DCD Table 14.3-7.	RAI LTR 129S response to RAI 15.00.03-001, item 7
4854	BLN,STD	Pt 02	FSAR 14	14.04.02	5. COLA Part 2, FSAR Chapter 14, Subsection 14.4.2, as shown in letter 139, will be revised to read: Preoperational and startup test specifications and procedures are provided to the NRC in accordance with the requirements of DCD Subsection 14.2.3. The controls for development of test specifications and procedures are also described in Subsection 14.2.3. A cross reference list is provided between ITAACs and test procedures and/or sections of test procedures.	RAI LTR 139 S1 response to RAI 14.02-012, item 5 SER with Open Items Confirmatory Item 14.2-3
3487	BLN	Pt 02	FSAR 15	15.00	8. COLA Part 2, FSAR Chapter 15, Section 15.0 will be revised from: This section of the referenced DCD is incorporated by reference with no departures or supplements. To read: This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.	RAI LTR 129S response to RAI 15.00.03-001, item 8
3488	BLN	Pt 02	FSAR 15	15.00.03.02	9. COLA Part 2, FSAR Chapter 15, Section 15.0 will be revised to add the following	RAI LTR 129S response to

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					information with an LMA of BLN DEP 2.3-1. 15.0.3.2 Initial Conditions Replace the third paragraph of DCD Subsection 15.0.3.2 with the following: Core power ±2 percent allowance for calorimetric error. The main feed water flow measurement supports a 1-percent power uncertainty; use of a 2-percent power uncertainty is conservative. Accidents use 2% core power uncertainty unless identified in Table 15.0-2.	RAI 15.00.03-001, item 9
3489	BLN	Pt 02	FSAR 15	15.00.T / T15.0-201	10. COLA Part 2, FSAR Chapter 15, Section 15.0, will be revised to add the following new table with an LMA of BLN DEP 2.3-1. TABLE 15.0-201 BLN SUMMARY OF INITIAL CONDITIONS AND COMPUTER CODES USED Note: This table supplements DCD Table 15.0-2.	RAI LTR 129S response to RAI 15.00.03-001, item 10
3490	BLN	Pt 02	FSAR 15	15.06	11. COLA Part 2, FSAR Chapter 15, Subsection 15.6 will be revised to add the following information with an LMA of BLN DEP 2.3-1. 15.6.5.3.1.2 Core Release Replace the first two sentences of the second paragraph of DCD Subsection 15.6.5.3.1.2 with the following sentence: The core fission product inventory at the time of the most accidents is based on operation near the end of a fuel cycle is provided in Table 15A-3 of DCD Appendix 15A and in Table 15A-201 of FSAR Appendix 15A.	RAI LTR 129S response to RAI 15.00.03-001, item 11
3491	BLN	Pt 02	FSAR 15	15.06.05.03.07.03	12. COLA Part 2, FSAR Chapter 15, Subsection 15.6.5.3.7.3, will be revised from: [Site-specific X/Q values provided in Subsection 2.3.4 are bounded by the values given in DCD Tables 15A-5 and 15A-6. (This text to be revised in a future amendment.)] To read: Site-specific X/Q values provided in Subsection 2.3.4 are not bounded by the values given in DCD Tables 15A-5 and 15A-6. Therefore, a site-specific dose consequence analysis was performed as discussed in Subsection 15.6.5.	RAI LTR 129S response to RAI 15.00.03-001, item 12
3492	BLN	Pt 02	FSAR 15	15.06.T / T15.6-201	13. COLA Part 2, FSAR Chapter 15, Section 15.6 will be revised to add the following information with an LMA of BLN DEP 2.3-1. TABLE 15.6-201 BLN ASSUMPTIONS AND PARAMETERS USED IN CALCULATING RADIOLOGICAL CONSEQUENCES OF A LOSS-OF-COOLANT ACCIDENT BLN Containment leakage release data - Containment leak rate, 0-24 hr (% per day) 0.09 (for EAB) 0.10 (for LPZ and Control Room) Note: This table supplements DCD Table 15.6.5-2.	RAI LTR 129S response to RAI 15.00.03-001, item 13
3493	BLN	Pt 02	FSAR 15	15.06.T / T15.6-202	14. COLA Part 2, FSAR Chapter 15, Section 15.6 to add the following information with an LMA of BLN DEP 2.3-1.	RAI LTR 129S response to RAI 15.00.03-001, item 14

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					TABLE 15.6-202 BLN RADIOLOGICAL CONSEQUENCES OF A LOSS-OF-COOLANT ACCIDENT WITH CORE MELT BLN Exclusion zone boundary dose (1.4 - 3.4 hr)(1) 23.8 Notes: 1. The effective unfiltered inleakage is based on a total inleakage of 5 cfm with credit taken for purging of the vestibule volume and the incomplete mixing of the vestibule and control room volumes with outside air following ingress/egress. 2. This table supplements DCD Table 15.6.5-2.	
5634	BLN	Pt 02	FSAR 15	15.06.T / T15.6-202	COLA Part 2, FSAR Chapter 15, Section 15.6, as modified by Qb 3493, is corrected to provide appropriate Table footnotes and column headers. Remove Note 1 as it is not applicable to this table. Change Note 2 from "2. This table supplements DCD Table 15.6.5-2." to read: "1. This table supplements DCD Table 15.6.5-3."	Editorial correction to RAI LTR 129S response to RAI 15.00.03-001, item 14
3494	BLN	Pt 02	FSAR 15	15A.03.01.03	15. COLA Part 2, FSAR Chapter 15, Appendix 15A will be revised to add the following information with an LMA of BLN DEP 2.3-1. 15A.3.1.3 Core Source Term Replace the first sentence of DCD Subsection 15A.3.1.3 with the following sentence: Table 15A-3 and FSAR Table 15A-201 list the core source terms at shutdown.	RAI LTR 129S response to RAI 15.00.03-001, item 15
3495	BLN	Pt 02	FSAR 15	15A.T / T15A-201	16. COLA Part 2, FSAR Chapter 15, Appendix 15A will be revised to add the following new Table 15A-201 with an LMA of BLN DEP 2.3-1. TABLE 15A-201 BLN REACTOR CORE SOURCE TERM(1) Note: 1. The following assumptions apply: • Core thermal power of 3434 MWt (1 percent above the design core power of 3400 MWt). • Three-region equilibrium cycle core at end of life. • These source terms applied only for the EAB doses.	RAI LTR 129S response to RAI 15.00.03-001, item 16
3496	BLN	Pt 02	FSAR 15	15A.T / T15A-202	17. COLA Part 2, FSAR Chapter 15, Appendix 15A, will be revised to add the following information with an LMA of BLN DEP 2.3-1. TABLE 15A-202 BLN OFFSITE ATMOSPHERIC DISPERSION FACTORS (χ/Q) FOR ACCIDENT DOSE ANALYSIS(1) BLN Site Boundary X/Q (s/m3) 0 - 2 hours(2) 5.85 x 10-4 Notes: 1. The LOCA dose analysis models the bounding atmospheric dispersion factors listed above. Other analyses model more conservative values.	RAI LTR 129S response to RAI 15.00.03-001, item 17

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					2. Nominally defined as the 0- to 2-hour interval but is applied to the 2-hour interval having the highest activity releases in order to address 10 CFR Part 50.34 requirements. 3. This table supplements DCD Table 15A-5.	
3497	BLN	Pt 02	FSAR 16	16.01	18. COLA Part 2, FSAR Chapter 16, Section 16.1, first sentence, will be revised from: Subsections 16.1.1 and 16.1.2 of the DCD are incorporated by reference with no departures or supplements. To read: Subsections 16.1.1 and 16.1.2 of the DCD are incorporated by reference with the following departures and/or supplements.	RAI LTR 129S response to RAI 15.00.03-001, item 18
4901	BLN,STD	Pt 02	FSAR 16	16.01	COLA Part 2, FSAR Chapter 16, Section 16.1, last two sentences, will be revised from: However, the generic technical specifications and bases provided with Chapter 16 of the DCD are incorporated by reference into the plant-specific technical specifications provided in Part 4 of this COL application. In addition, a full information set of the plant-specific technical specifications and bases are provided in Part 4 of this COL application. To read: However, the generic technical specifications and bases provided with Chapter 16 of the DCD are incorporated directly into the plant-specific technical specifications and bases provided in Part 4 of this COL application.	Editorial
3498	BLN	Pt 02	FSAR 16	16.01	19. COLA Part 2, FSAR Chapter 16, Section 16.1 will be revised to add the following at the end of the current text with an LMA of BLN DEP 2.3-1: The plant-specific Technical Specifications include an allowable primary containment leakage rate of 0.09% of primary containment air weight per day. This departure from the DCD Generic Technical Specifications and Bases is described and justified in Parts 4 and 7 of the COL application.	RAI LTR 129S response to RAI 15.00.03-001, item 19
4889	BLN,STD	Pt 02	FSAR 17	17.04	COLA Part 2, FSAR, Chapter 17, Section 17.4 "Design Reliability Assurance Program" will be revised to read: This section of the referenced DCD is incorporated by reference with the following departures and/or supplements. STD SUP 17.4-1 The quality assurance requirements for non-safety related SSCs within the scope of D-RAP is in accordance with the Quality Assurance Program Description (QAPD), Part III.	RAI LTR 150 response to RAI 17.04-002 SER with Open Items Confirmatory Item 17.4-1
5831	BLN	Pt 02	FSAR 17	17.05	6. Revise FSAR Subsection 17.5 to include the following new paragraph following the existing first paragraph (with the same LMAs as the existing first paragraph): Conformance statements for QA-related Regulatory Guides (including Regulatory Guides 1.28, 1.30, 1.33, 1.38, 1.39, 1.94, and 1.116) are provided in Appendix 1AA. While many Regulatory Guide positions can be identified as applicable to the scope of work identified and addressed by the DCD and others can be identified as applicable to the scope of work identified and addressed by the COLA, some QA guidance related positions could be accomplished by either scope of work and thus be addressed in either the DCD or the COLA. These positions are primarily dependent on who performs the work. The DCD conformance statement indicates an exception to apply NQA-1. The COLA identifies an exception to apply NQA-1. Per DCD Section 17.3, WEC work performed up to March 15, 2007 applied a 1991 version of the standard. A 1994 version of the standard is applied for work performed after that date by WEC. If the work is performed under the applicant's COL program, the 1994 version of NQA-1 identified in the COLA QAPD is applied. Thus, DCD scope (identified in DCD Appendix 1A) and "remaining scope" differentiate the application of the guidance identified in these	COL-SER-OI-Ch01 S1 response to OI 01.04-02 item 6

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					Regulatory Guides.	
4842	BLN,STD	Pt 02	FSAR 17	17.06	Add "(Reference 205)" {red, hyperlinked text} after "...10 CFR Part 52," as "...NEI 07-02A, Generic FSAR Template Guidance for Maintenance Rule Program Description for Plants Licensed Under 10 CFR Part 52," (Reference 205), with the following supplemental information.	Editorial
4903	BLN,STD	Pt 02	FSAR 17	17.08	Revise Reference 203 initial quote marks to be "beginning" quote instead of "ending" quote. Revise Reference 205 quotation mark font (beginning and ending) to match the font used for other quotation marks.	Editorial
4920	BLN,STD	Pt 02	FSAR 19	19.58	1. COLA Part 2, FSAR Chapter 19, Subsection 19.58 will be revised to read: This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.	RAI LTR 152 response to RAI 19-09 item 1
4921	BLN	Pt 02	FSAR 19	19.58	2. COLA Part 2, FSAR Chapter 19, Subsection 19.58 will be revised to add new Subsection 19.58.3, that reads: 19.58.3 Conclusion Add the following information at the end of DCD Subsection 19.58.3: BLN SUP 19.58-1 Table 19.58-201 documents the site-specific external events evaluation that has been performed for BLN Units 3 and 4. This table provides a general explanation of the evaluation and resultant conclusions and provides a reference to applicable sections of the COL where more detailed supporting information (including data used, methods and key assumptions) regarding the specific event is located. Based upon this evaluation, it is concluded that the BLN Units 3 and 4 site is bounded by the High Winds, Floods and Other External Events analysis documented in DCD Section 19.58 and APP-GW-GLR -101 (Reference 201) and no further evaluations are required at the COL application stage.	RAI LTR 152 response to RAI 19-09 item 2
4922	BLN	Pt 02	FSAR 19	19.58	3. COLA Part 2, FSAR Chapter 19, Subsection 19.58 will be revised to add new Subsection 19.58.4, that reads: 19.58.4 References 201. Westinghouse Electric Company LLC, "AP1000 Probabilistic Risk Assessment Site-Specific Considerations," Document Number APP-GW-GLR-101, Revision 1, October 2007. 202. NUREG/CR-4461, "Tornado Climatology of the Contiguous United States," Revision 2, February 2007. 203. Texas Tech University, Wind Science and Engineering Center, "A Recommendation for an Enhanced Fujita Scale (EF-Scale)," June 2004. 204. ASCE Standard ASCE/SEI 7-05, "Minimum Design Loads for Buildings and Other Structures," 2006.	RAI LTR 152 response to RAI 19-09 item 3
4924	BLN	Pt 02	FSAR 19	19.58.T / T19.58-201	4. COLA Part 2, FSAR Chapter 19, Section 19.58, add new Table 19.58-201 "External Event Frequencies for BLN" as shown in Attachment 19-09A.	RAI LTR 152 response to RAI 19-09 item 4 SER with Open Items Confirmatory Item 19.58-1
5961	BLN	Pt 02	FSAR 19	19.58.T / T19.58-201	COLA Part 2, FSAR Chapter 19, Section 19.58, new Table 19.58-201 is revised to add LMA of BLN SUP 19.58-1	Revises RAI LTR 152 response to RAI 19-09 item 4 SER with Open Items Confirmatory Item 19.58-1

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
4895	BLN	Pt 02	FSAR 19	19.58.T / T19.58-201 Note 1	1. COLA Part 2, FSAR Chapter 19, Subsection 19.58 will be revised to include clarification of "Applicable" in FSAR Table 19.58-201. Refer to response to RAI 19-09, this letter, for the details of COLA changes.	INCLUDED in Qb 4924 - RAI LTR 152 response to RAI 19-09
4935	BLN,STD	Pt 02	FSAR 19	19.59.10.05	1. COLA Part 2, FSAR Chapter 19, subsection 19.59.10.5, STD COL 19.59.10-1, first three sentences will be changed to read: A review of the differences between the as-built plant and the design used as the basis for the AP1000 seismic margins analysis will be completed prior to fuel load. A verification walkdown will be performed with the purpose of identifying differences between the as-built plant and the design. Any differences will be evaluated and the seismic margins analysis modified as necessary to account for the plant-specific design, and any design changes or departures from the certified design.	RAI LTR 152 response to RAI 19-20 item 1
4923	BLN,STD	Pt 02	FSAR 19	19.59.10.05	5. COLA Part 2, FSAR Chapter 19, Subsection 19.59.10.5, fourth paragraph will be revised to read: As discussed in Section 19.58.3, it has been confirmed that the Winds, Floods, and Other External Events analysis documented in DCD Section 19.58 is applicable to the site. The site-specific design has been evaluated and is consistent with the AP1000 PRA assumptions. Therefore, Section 19.58 of the AP1000 DCD is applicable to this design.	RAI LTR 152 response to RAI 19-09 item 5
4936	BLN,STD	Pt 02	FSAR 19	19.59.10.05	2. COLA Part 2, FSAR Chapter 19, Subsection 19.59.10.5, STD COL 19.59.10-3 will be revised to read: A review of the differences between the as-built plant and the design used as the basis for the AP1000 internal fire and internal flood analyses will be completed prior to fuel load. Plant specific internal fire and internal flood analyses will be evaluated and the analyses modified as necessary to account for the plant-specific design, and any design changes or departures from the certified design.	RAI LTR 152 response to RAI 19-20 item 2
4905	BLN,STD	Pt 02	FSAR 19	19.59.10.06	Add "(Reference 201)" {red, hyperlinked text} under heading "PRA Input to the Reactor Oversight Process" at the end of the first paragraph to read - "The mitigating systems performance indicators (MSPI) are evaluated based on the indicators and methodologies defined in NEI 99-02 (Reference 201)."	Editorial
4906	BLN,STD	Pt 02	FSAR 19	19.59.11	Add the following to include new Reference 201: 19.59.11 References [separator bar] Add the following text to the end of DCD Subsection 19.59.11: 201. NEI 99-02, Nuclear Energy Institute, "Regulatory Assessment Performance Indicator Guideline," Technical Report NEI 99-02, Revision 5, July 2007.	Editorial
Pt 04						**27 COLA Changes**
5790	BLN,STD	Pt 04		A, A.2-4.1.2	Change COLA Part 4, Section A.2, GTS 4.1.2, Justification reference from "FSAR Section 2.1.3.5" to read "FSAR Subsection 2.1.3.4."	Editorial
4950	BLN,STD	Pt 04		A, A.2-5.2.2	3. Change COLA Part 4, Section A.2, second item GTS 5.2.2, to read: GTS 5.2.2 The bracketed information in the GTS reads: [The unit staff organization shall include the following: a. A non-licensed operator shall be assigned to each reactor containing fuel and an … b., c., d., e. …Policy Statement on Engineering Expertise on Shift.] Remove the brackets and adopt the bracketed information in the GTS except that 5.2.2.d is omitted.	BLN-P02-VOL-SEC-FFD-20090323-OR item 3

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					Justification: Generic TS bracketed information is applicable and adopted except for GTS 5.2.2.d which is no longer necessary due to revisions to Part 26 since the approval of the GTS. The removal of GTS 5.2.2.d is consistent with TSTF-511 identified by NRC as an appropriate change to implement the revisions to Part 26 (See 73 FR 79923, Notice of Availability of Model Safety Evaluation, Model No Significant Hazards Determination, and Model Application for Licensees That Wish To Adopt TSTF–511, Revision 0, "Eliminate Working Hour Restrictions From TS 5.2.2 To Support Compliance With 10 CFR Part 26").	
3499	BLN	Pt 04		A, A.3	20. COLA Part 4, Section A, will be revised to include new Section A.3 to read: A.3 The following items are departures from the AP1000 DCD Generic Technical Specifications and Bases. GTS 5.5.8 The maximum allowable primary containment leakage rate, La, at Pa, is reduced from the GTS value of 0.10% to 0.09% of primary containment air weight per day. This departure is further discussed and justified in Part 7 of the COL application.	RAI LTR 129S response to RAI 15.00.03-001, item 20
5712	BLN	Pt 04		A, A.3	COLA Part 4, Section A, added Section A.3 will be revised from A.3 to 3 for formatting consistency.	Editorial revision to RAI LTR 129S response to RAI 15.00.03-001, item 20
6360	BLN	Pt 04		B, 00 TOC/Rev Summary	Technical Specifications Table of Contents/Revision Summary page, under Revision Column, Replace FSAR 1 with FSAR 2.	Conform to revision status of COLA
4908	BLN,STD	Pt 04		B, 01.03	COLA Part 4, Section B, Technical Specification 1.3 will be revised on page 1.3-6 under EXAMPLES in the next to last sentence on the page to move the period inside the quotation marks around "clock."	Consistency with WEC AP1000 GTS
4909	BLN,STD	Pt 04		B, 03.01.04	COLA Part 4, Section B, Technical Specification LCO 3.1.4 will be revised to include line spaces before and after the underlined ALL CAPS "AND" in the LCO.	Consistency with WEC AP1000 GTS
4910	BLN,STD	Pt 04		B, 03.03.01	COLA Part 4, Section B, Technical Specification 3.3.1 Action K will be revised to move the underlined ALL CAPS "OR" between Required Actions K.1.2 and K.2 to the left to vertically align with the K.1.2 and K.2. COLA Part 4, Section B, Technical Specification 3.3.1 Action L will be revised to move the underlined ALL CAPS "OR" between Required Actions L.1 and L.2 to the left to vertically align with the L.1 and L.2. COLA Part 4, Section B, Technical Specification 3.3.1 Action M will be revised to move the underlined ALL CAPS "OR" between Required Actions M.1 and M.2.1 to the left to vertically align with the M.1 and M.2.1.	Consistency with WEC AP1000 GTS
5791	BLN,STD	Pt 04		B, 03.03.01	COLA Part 4, Section B, Technical Specification 3.3.1 SR 3.3.1.3, Note 3, remove second "the" from the phrase "3. If the calorimetric heat balance is < 70% RTP, and if the qΔT is:" to read "3. If the calorimetric heat balance is < 70% RTP, and if qΔT is:"	Consistency with WEC AP1000 GTS
4912	BLN,STD	Pt 04		B, 03.07.09	COLA Part 4, Section B, Technical Specification SR 3.7.9.3 will be revised to add a comma after SFS-PL-V066.	Consistency with WEC AP1000 GTS
4952	BLN,STD	Pt 04		B, 05.02.02	5. Change COLA Part 4, Section B, Complete Copy of PSTS and Bases, to revise current PSTS 5.2.2.b (which refers to TS 5.2.2.f) per above revision to read: b. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) and	BLN-P02-VOL-SEC-FFD-20090323-OR item 5

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					5.2.2.a and 5.2.2.e for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.	
4951	BLN,STD	Pt 04		B, 05.02.02	4. Change COLA Part 4, Section B, Complete Copy of PSTS and Bases, to omit current PSTS 5.2.2.d, and renumber current 5.2.2.e and 5.2.2.f as 5.2.2.d and 5.2.2.e to read (Note that Item 4 of the letter refers to changing 5.3.3.d - this has been corrected to 5.2.2.e): d. The operations manager or assistant operations manager shall hold an SRO license. e. An individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift.	BLN-P02-VOL-SEC-FFD-20090323-OR item 4
3500	BLN	Pt 04		B, 05.05.08	21. COLA Part 4, Section B, Technical Specification 5.5.8 will be revised from: 5.5.8 Containment Leakage Rate Testing Program c. The maximum allowable primary containment leakage rate, La, at Pa, shall be 0.10% of primary containment air weight per day. To read: 5.5.8 Containment Leakage Rate Testing Program c. The maximum allowable primary containment leakage rate, La, at Pa, shall be 0.09% of primary containment air weight per day.	RAI LTR 129S response to RAI 15.00.03-001, item 21
6361	BLN	Pt 04		B, B00 TOC/ Rev Summary	Technical Specifications Table of Contents/Revision Summary page, under Revision Column, Replace FSAR 1 with FSAR 2.	Conform to revision status of COLA
5138	BLN,STD	Pt 04		B, B00 TOC/ Bases	COLA Part 4, Section B, Technical Specification Bases Table of Contents page ii - Add line space following the top line that reads: B 3.4 REACTOR COOLANT SYSTEM (RCS) (continued) and deleted the duplicate of the top line that is the third line.	Editorial
6362	BLN	Pt 04		B, B02.01.02 / 2.1.2-1	Revise header from "Reactor Core SLs" to read "RCS Pressure SL".	Consistency with DCD
5139	BLN,STD	Pt 04		B, B03.01.06	COLA Part 4, Section B, Technical Specification footer placement is not consistent wtih the rest of the Bases from B 3.1.6-1 through B 3.3.5-5 and at B 3.4.9-6. Correct the footer placement so that they are consistent throughout the document.	Editorial
6500	BLN	Pt 04		B, B03.03.01	COLA Part 4, Section B, Bases 3.3.1, APPLICABLE SAFETY ANALYSES, LCOs, and APPLICABILITY, item 12 remove extra line space in the middle of the paragraph. COLA Part 4, Section B, Bases 3.3.1, APPLICABLE SAFETY ANALYSES, LCOs, and APPLICABILITY, item 14 remove extra line space in the middle of the last paragraph.	Editorial
6501	BLN	Pt 04		B, B03.03.02	COLA Part 4, Section B, Bases 3.3.2, APPLICABLE SAFETY ANALYSES, LCOs, and APPLICABILITY, item 4.b remove extra line space in the middle of the last paragraph. COLA Part 4, Section B, Bases 3.3.2, APPLICABLE SAFETY ANALYSES, LCOs, and APPLICABILITY, item 4.c(2) remove extra line space in the middle of the first paragraph. COLA Part 4, Section B, Bases 3.3.2, APPLICABLE SAFETY ANALYSES, LCOs, and APPLICABILITY, item 16.d remove extra line space in the middle of the paragraph.	Editorial

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
4913	BLN,STD	Pt 04		B, B03.03.02	COLA Part 4, Section B, Bases 3.3.2, APPLICABLE SAFTEY ANALYSES, LCOs, and APPLICABILITY item 18.a, second paragraph will be revised to capitalize the first letter of "functions" in the sentence that begins "The functions of the P-4 interlock..." to read "The Functions of the P-4 interlock..." COLA Part 4, Section B, Bases 3.3.2, APPLICABLE SAFTEY ANALYSES, LCOs, and APPLICABILITY item 18.a, third paragraph, first sentence will be revised to read "The reactor trip breaker position switches that provide input to the P-4 interlock only function to energize or de-energize or open or close contacts."	Consistency with WEC AP1000 GTS
5765	BLN,STD	Pt 04		B, B03.03.02	COLA Part 4, Section B, Bases 3.3.2, APPLICABLE SAFETY ANALYSES, LCOs, and APPLICABILITY, item 18.a remove extra line space in the middle of the second paragraph. COLA Part 4, Section B, Bases 3.3.2, APPLICABLE SAFETY ANALYSES, LCOs, and APPLICABILITY, item 18.d remove extra line space in the middle of the paragraph.	Editorial
4914	BLN,STD	Pt 04		B, B03.03.02	COLA Part 4, Section B, Bases 3.3.2, APPLICABLE SAFTEY ANALYSES, LCOs, and APPLICABILITY item 18.e, will be revised to add a comma following "...on High 2 pressurizer water level" in the second sentence.	Consistency with WEC AP1000 GTS
4915	BLN,STD	Pt 04		B, B03.03.02	COLA Part 4, Section B, Bases 3.3.2, ACTIONS item E.1, will be revised to remove the inadvertent line space in the middle of the last paragraph. COLA Part 4, Section B, Bases 3.3.2, ACTIONS item H.1, will be revised to remove the inadvertent line space in the middle of the second paragraph. COLA Part 4, Section B, Bases 3.3.2, ACTIONS item I.1 and I.2, will be revised to add the missing line space following the paragraph and before the subheader J.1 and J.2. COLA Part 4, Section B, Bases 3.3.2, ACTIONS item J.1 and J.2, will be revised to remove the inadvertent line space in the middle of the third paragraph. COLA Part 4, Section B, Bases 3.3.2, ACTIONS item P.1, P.2.1 and P.2.2, will be revised to remove the inadvertent line space in the middle of the last paragraph. COLA Part 4, Section B, Bases 3.3.2, ACTIONS item R.1, R.2.1.1, R.2.2, will be revised to remove the inadvertent line space in the middle of the last paragraph. COLA Part 4, Section B, Bases 3.3.2, ACTIONS item V.1, V.2.1, and V.2.2, will be revised to remove the inadvertent line space in the middle of the second paragraph. COLA Part 4, Section B, Bases 3.3.2, ACTIONS item Y.1, Y.2, Y.3 and Y.4, will be revised to remove the inadvertent line space in the middle of the last paragraph.	Consistency with WEC AP1000 GTS/Editorial
4916	BLN,STD	Pt 04		B, B03.04.03	COLA Part 4, Section B, Bases 3.4.3, SR 3.4.3.1, will be revised to make the current referenct to "a Note" to ALL CAPS to read "a NOTE" within the first sentence of the third paragraph.	Consistency with WEC AP1000 GTS
4917	BLN,STD	Pt 04		B, B03.04.04	COLA Part 4, Section B, Bases 3.4.4, LCO, will be revised to make the current reference to "the Note" in the sixth and tenth paragraphs to ALL CAPS to read "the NOTE" in both places.	Consistency with WEC AP1000 GTS
3501	BLN	Pt 04		B, B03.06.01	22. COLA Part 4, Section B, Bases 3.6.1, Applicable Safety Analyses, second paragraph will be revised from: The DBAs that result in a challenge to containment OPERABILITY from high pressures and temperatures are a loss of coolant accident (LOCA), a steam line break, and a rod ejection accident (REA) (Ref. 2). In addition, release of significant fission product radioactivity within containment can occur from a LOCA or REA. The DBA analyses assume that the containment is OPERABLE such that, for the DBAs involving release of fission product radioactivity, release to the environment is	RAI LTR 129S response to RAI 15.00.03-001, item 22

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					controlled by the rate of containment leakage. The containment is designed with an allowable leakage rate of 0.10% of containment air weight of the original content of containment air after a DBA per day (Ref. 3). This leakage rate, used in the evaluation of offsite doses resulting from accidents, is defined in 10 CFR 50, Appendix J (Ref. 1), as La: the maximum allowable containment leakage rate at the calculated peak containment internal pressure (Pa) resulting from the limiting DBA. The allowable leakage rate represented by La forms the basis for the acceptance criteria imposed on containment leakage rate testing. La is assumed to be 0.10% per day in the safety analysis. To read: The DBAs that result in a challenge to containment OPERABILITY from high pressures and temperatures are a loss of coolant accident (LOCA), a steam line break, and a rod ejection accident (REA) (Ref. 2). In addition, release of significant fission product radioactivity within containment can occur from a LOCA or REA. The DBA analyses assume that the containment is OPERABLE such that, for the DBAs involving release of fission product radioactivity, release to the environment is controlled by the rate of containment leakage. The containment is designed with an allowable leakage rate of 0.09% of containment air weight of the original content of containment air after a DBA per day (Ref. 3). This leakage rate, used in the evaluation of offsite doses resulting from accidents, is defined in 10 CFR 50, Appendix J (Ref. 1), as La: the maximum allowable containment leakage rate at the calculated peak containment internal pressure (Pa) resulting from the limiting DBA. The allowable leakage rate represented by La forms the basis for the acceptance criteria imposed on containment leakage rate testing. La is assumed to be 0.09% per day in the safety analysis.	
3502	BLN	Pt 04		B, B03.06.02	23. COLA Part 4, Section B, Bases 3.6.2, Applicable Safety Analyses, first paragraph will be revised from: The DBA that results in the largest release of radioactive material within containment is a loss of coolant accident (LOCA) (Ref. 3). In the analyses of DBAs, it is assumed that containment is OPERABLE, such that release of fission products to the environment is controlled by the rate of containment leakage. The containment is designed with an allowable leakage rate of 0.10% of containment air weight of the original content of containment air per day after a DBA (Ref. 2). This leakage rate is defined in 10 CFR 50, Appendix J (Ref. 1), as La, the maximum allowable containment leakage rate at the calculated peak containment internal pressure Pa following a DBA. This allowable leakage rate forms the basis for the acceptance criteria imposed on the SRs associated with the air locks. To read: The DBA that results in the largest release of radioactive material within containment is a loss of coolant accident (LOCA) (Ref. 3). In the analyses of DBAs, it is assumed that containment is OPERABLE, such that release of fission products to the environmen is controlled by the rate of containment leakage. The containment is designed with an allowable leakage rate of 0.09% of containment air weight of the original content of containment air per day after a DBA (Ref. 2). This leakage rate is defined in 10 CFR 50, Appendix J (Ref. 1), as La, the maximum allowable containment leakage rate at the calculated peak containment internal pressure Pa following a DBA. This allowable leakage rate forms the basis for the acceptance criteria imposed on the SRs associated with the air locks.	RAI LTR 129S response to RAI 15.00.03-001, item 23
Pt 05						**73 COLA Changes**
4806	BLN	Pt 05		- Definitions	1. COLA Part 5, Emergency Plan, Definitions, will be revised by changing the last sentence of the definition for Hostile Action to read: Non-terrorism based EALs are used to address such activities (e.g., violent acts between individuals	SUPERSEDED BY Qb 5329 - RAI LTR 146 response to RAI 13.03-033, item 1

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					in the owner controlled area).	
4807	BLN	Pt 05		- Definitions	2. COLA Part 5, Emergency Plan, Definitions, will be revised by adding a new defined term, Imminent, as follows: Imminent – Mitigation actions have been ineffective, additional actions are not expected to be successful, and trended information indicates that the event or condition will occur.	SUPERSEDED BY Qb 5330 - RAI LTR 146 response to RAI 13.03-033, item 2
5329	BLN	Pt 05		- Definitions	1. COLA Part 5, Emergency Plan, Definitions, will be revised by changing the last sentence of the definition for Hostile Action, from: Non-terrorism based EALs should be used to address such activities (e.g., violent acts between individuals in the owner controlled area). To read: Non-terrorism based EALs are used to address such activities (e.g., violent acts between individuals in the owner controlled area).	BLN RAI LTR 146S response to RAI 13.03-040 item 1 - SUPERSEDES Qb 4806
5330	BLN	Pt 05		- Definitions	2. COLA Part 5, Emergency Plan, Definitions, will be revised by adding a new defined term, Imminent, as follows: Imminent – Mitigation actions have been ineffective, additional actions are not expected to be successful, and trended information indicates that the event or condition will occur.	BLN RAI LTR 146S response to RAI 13.03-040 item 2 - SUPERSEDES Qb 4807
5331	BLN	Pt 05		I.B	3. COLA Part 5, Emergency Plan, Section I.B, first paragraph, will be revised from: Appendix 1 identifies radiological emergency recognition categories, their initiating conditions, and Emergency Action Levels (EALs). To read: BLN Emergency Plan Implementing Procedure, "Emergency Classification," identifies radiological emergency recognition categories, their initiating conditions, and Emergency Action Levels (EALs).	BLN RAI LTR 146S response to RAI 13.03-040 item 3
3684	BLN	Pt 05		II.A.1.a / II-1	5. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.A.1.a will be revised to update the principal organizations participating in emergency response activities at BLN.	RAI LTR 122 S4 response to RAI 13.03-18A, B, D & E, item 5. Application revisions 1-4 were previously identifed and incorporated into EP Revision 1.
5332	BLN	Pt 05		II.A.1.b	4. COLA Part 5, Emergency Plan, Section II.A.1.b, second paragraph, third sentence, will be revised from: Using approved emergency response procedures, including the Emergency Action Levels (EALs) provided in Appendix 1 of this plan, the Shift Manager determines if an emergency condition exists and, if so, the proper emergency classification. To read: Using approved emergency response procedures, including the Emergency Action Levels (EALs) provided in EPIP, "Emergency Classification," the Shift Manager determines if an emergency condition exists and, if so, the proper emergency classification.	BLN RAI LTR 146S response to RAI 13.03-040 item 4
3685	BLN	Pt 05		II.A.1.b / II-4	6. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.A.1.b, under the sub-headings "The State of Alabama" and "County Governments," will be revised.	RAI LTR 122 S4 response to RAI 13.03-18A, B, D & E, item 6. Application revisions 1-4 were previously identifed and incorporated into EP

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						Revision 1.
3686	BLN	Pt 05		II.A.F/F II-1	7. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Figure II-1 will be revised to include a breakdown of State/Local EOCs.	RAI LTR 122 S4 response to RAI 13.03-18A, B, D & E, item 7. Application revisions 1-4 were previously identifed and incorporated into EP Revision 1.
5140	BLN	Pt 05		II.B.F/F II-2	COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Figure II-2 will be revised to provide the text of footnote c associated with the RP Techs, the Chem Lab Techs, and the Mech. Craftsmen. Footnote c should read: (c) Augment within approximately 90 minutes of declaration	Editorial
6008	BLN	Pt 05		II.B.T/T II-2	COLA Part 5, Emergency Plan, Table II-2 will be replaced with Table II-2 provided in Attachment OI 13.03-04 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-004 (eRAI 13.03-44)
6009	BLN	Pt 05		II.B.T/T II-2	COLA Part 5, Emergency Plan, Table II-2 will be replaced with Table II-2 provided in Attachment OI 13.03-04 of this letter.	COL-SER-OI-Ch13 response to OI-13.03-005 (eRAI 13.03-45) - This COLA Change is a duplicate of Qb ID #6008
3690	BLN	Pt 05		II.C.1 / II-31	6. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.C.1 "Emergency Response Support and Resources" will be revised.	RAI LTR 122 S4 response to 13.03-20A & C, item 6 SER with Open Items Confirmatory Item 13.3-1
3691	BLN	Pt 05		II.C.3 / II-31	7. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.C.3 "Radiological Laboratories" will be revised.	RAI LTR 122 S4 response to 13.03-20A & C, item 7 SER with Open Items Confirmatory Item 13.3-2
4808	BLN	Pt 05		II.D	3. COLA Part 5, Emergency Plan, Subsection II.D, first paragraph will be revised to read: TVA uses a standard emergency classification scheme based on system and effluent parameters, which allows affected State and local response organizations to determine initial off-site response measures. For BLN, the initiating conditions are those provided in NEI 07-01, "Methodology for Development of Emergency Action Levels, Advanced Passive Light Water Reactors" (Reference 6) as it applies to AP1000 facilities and postulated accidents identified in the FSAR.	SUPERSEDED by Qb 5333 - RAI LTR 146 response to RAI 13.03-033, item 3
5333	BLN	Pt 05		II.D	5. COLA Part 5, Emergency Plan, Subsection II.D, first paragraph will be revised from: TVA has developed and implemented a standard emergency classification scheme, based on system and effluent parameters, on which affected State and local response organizations may rely for determining initial off-site response measures. For BLN, the initiating conditions include the conditions provided in NEI 07-01, Rev. 0, "Methodology for Development of Emergency Action Levels, Advanced Passive Light Water Reactors" (Reference 6) as it applies to AP1000 facilities and postulated accidents identified in the FSAR. To read: TVA uses a standard emergency classification scheme based on system and effluent parameters, which allows affected State and local response organizations to determine initial off-site response measures.	BLN RAI LTR 146S response to RAI 13.03-040 item 5 - SUPERSEDES Qb 4808
5334	BLN	Pt 05		II.D	6. COLA Part 5, Emergency Plan, Subsection II.D, third paragraph will be revised from:	BLN RAI LTR 146S

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					The range of conditions in NEI 07-01 and the applicable FSAR have been considered in the classification system of this plan. To read: The expected range of conditions from the applicable FSAR have been considered in the classification system of this plan.	response to RAI 13.03-040 item 6
4809	BLN	Pt 05		II.D.1	4. COLA Part 5, Emergency Plan, Subsection II.D.1, will be revised to read: Appendix E of 10 CFR Part 50 identifies four distinct classes of emergencies. The definitions of these emergency classes are more fully discussed in NEI 07-01, as follows: • Notification of Unusual Event - Events are in progress or have occurred which indicate a potential degradation of the level of safety of the plant or indicate that a security threat to facility protection has been initiated. No releases of radioactive material requiring off-site response or monitoring are expected unless further degradation of safety systems occurs. TVA actions undertaken at the Notification of Unusual Event include promptly informing State and local authorities of the event, augmenting on-shift resources as needed, assessment and response, and escalation to a more severe class, if appropriate. If the emergency class is not escalated to a more severe class, then State and local authorities will be notified of event termination in accordance with implementing procedures. • Alert - Events are in progress or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable lifethreatening risk to site personnel or damage to site equipment because of hostile action. Any releases are expected to be limited to small fractions of the EPA PAG exposure levels. TVA actions undertaken at the Alert emergency class include those described for the Notification of Unusual Event and activation of the TSC and OSC. In addition, CECC and other key emergency response personnel are alerted, on-site monitoring teams are dispatched, periodic plant status updates and meteorological assessments are provided to offsite authorities, as are dose estimates, if any event-related releases are occurring. • Site Area Emergency - Events are in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public or hostile action that results in intentional damage or malicious acts; 1) toward site personnel or equipment that could lead to the likely failure of or; 2) that prevent effective access to, equipment needed for the protection of the public. Any releases are not expected to result in exposure levels which exceed EPA PAG exposure levels beyond the site boundary. TVA actions undertaken at the Site Area Emergency class include those described for the Alert emergency class and activation of the CECC. In addition, an individual is dedicated to provide plant status updates to offsite authorities and periodic media briefings (jointly with offsite authorities when practicable), senior technical and management staff are made available for consultation with NRC and the State on a periodic basis, and release and dose projections based on available plant condition information and foreseeable contingencies are provided. • General Emergency - Events are in progress or have occurred which involve actual or imminent substantial core degradation or melting with potential for loss of containment integrity or hostile action that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA PAG exposure levels off-site for more than the immediate site area. TVA actions undertaken at the General Emergency class are identical to those described for the Site Area Emergency class except there is no more severe emergency class.	SUPERSEDED by Qb 5335 - RAI LTR 146 response to RAI 13.03-033, item 4

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					Appendix 1 of this plan provides recognition categories, the associated initiating condition matrices, and the emergency action levels based on the NEI 07-01 [Appendix 1 is "reserved" until after the NRC-endorsed version of NEI 07-01, Rev. 0 is available].	
5335	BLN	Pt 05		II.D.1	7. COLA Part 5, Emergency Plan, Subsection II.D.1, will be revised to read: Appendix E of 10 CFR Part 50 identifies four distinct classes of emergencies. The definitions of these emergency classes are as follows: • Notification of Unusual Event - Events are in progress or have occurred which indicate a potential degradation of the level of safety of the plant or indicate a security threat to facility protection has been initiated. No releases of radioactive material requiring off-site response or monitoring are expected unless further degradation of safety systems occurs. TVA actions undertaken at the Notification of Unusual Event include promptly informing State and local authorities of the event, augmenting on-shift resources as needed, assessment and response, and escalation to a more severe class, if appropriate. If the emergency class is not escalated to a more severe class, then State and local authorities will be notified of event termination in accordance with implementing procedures. • Alert - Events are in progress or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable lifethreatening risk to site personnel or damage to site equipment because of hostile action. Any releases are expected to be limited to small fractions of the EPA PAG exposure levels. TVA actions undertaken at the Alert emergency class include those described for the Notification of Unusual Event and activation of the TSC and OSC. In addition, CECC and other key emergency response personnel are alerted, on-site monitoring teams are dispatched, periodic plant status updates and meteorological assessments are provided to offsite authorities, as are dose estimates, if any event-related releases are occurring. • Site Area Emergency - Events are in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public or hostile action that results in intentional damage or malicious acts; 1) toward site personnel or equipment that could lead to the likely failure of or; 2) that prevent effective access to, equipment needed for the protection of the public. Any releases are not expected to result in exposure levels which exceed EPA PAG exposure levels beyond the site boundary. TVA actions undertaken at the Site Area Emergency class include those described for the Alert emergency class and activation of the CECC. In addition, an individual is dedicated to provide plant status updates to offsite authorities and periodic media briefings (jointly with offsite authorities when practicable), senior technical and management staff are made available for consultation with NRC and the State on a periodic basis, and release and dose projections based on available plant condition information and foreseeable contingencies are provided. • General Emergency - Events are in progress or have occurred which involve actual or imminent substantial core degradation or melting with potential for loss of containment integrity or hostile action that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA PAG exposure levels off-site for more than the immediate site area. TVA actions undertaken at the General Emergency class are identical to those described for the Site Area Emergency class except there is no more severe emergency class. BLN EPIP, "Emergency Classification," provides recognition categories, the associated initiating condition matrices, and the emergency action levels.	BLN RAI LTR 146S response to RAI 13.03-040 item 7 - SUPERSEDES Qb 4809

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4810	BLN	Pt 05		II.D.2	5. COLA Part 5, Emergency Plan, Subsection II.D.2, will be revised to read: TVA adopts the methodology provided in NEI 07-01. Appendix 1 provides the parameter values and equipment status that are indicative of each emergency class. [Appendix 1 is "Reserved" until after the NRC-endorsed version of NEI 07-01, Rev. 0 is available.]	SUPERSEDED by Qb 5336 - RAI LTR 146 response to RAI 13.03-033, item 5
5336	BLN	Pt 05		II.D.2	8. COLA Part 5, Emergency Plan, Subsection II.D.2, first paragraph will be revised from: This section incorporates by reference NEI 07-01, "Methodology for Development of Emergency Action Levels, Advanced Passive Light Water Reactors", Rev.0, dated [to be provided], ADAMS No. [to be provided]. Appendix 1 provides the parameter values and equipment status that are indicative of each emergency class. To read: BLN EPIP, "Emergency Classification," provides the parameter values and equipment status that are indicative of each emergency class. Changes to BLN EPIP, "Emergency Classification," are developed and approved consistent with the requirements of 10 CFR 50.54(q) and the guidance provided in USNRC Regulatory Issue Summary (RIS) 2005-02.	BLN RAI LTR 146S response to RAI 13.03-040 item 8 - SUPERSEDES Qb 4810
3692	BLN	Pt 05		II.E.1	1. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.E.1, "Notification Methods and Procedures: will be revised.	RAI LTR 122 S4 response to 13.03-22A, B, C & E, item 1 SER with Open Items Confirmatory Item 13.3-3
5337	BLN	Pt 05		II.E.1	9. COLA Part 5, Emergency Plan, Subsection II.E.1, first sentence, will be revised from: TVA establishes systems and procedures needed to provide prompt notification of affected State, local, and Federal authorities following the declaration of any emergency condition, consistent with the emergency classification and action level scheme described in Appendix 1. To read: TVA establishes systems and procedures needed to provide prompt notification of affected State, local, and Federal authorities following the declaration of any emergency condition, consistent with the emergency classification and action level scheme described in BLN EPIP, "Emergency Classification."	BLN RAI LTR 146S response to RAI 13.03-040 item 9
3693	BLN	Pt 05		II.E.3	2. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.E.3, will be revised.	RAI LTR 122 S4 response to 13.03-22A, B, C & E, item 2 SER with Open Items Confirmatory Item 13.3-4
3694	BLN	Pt 05		II.E.7	3. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.E.7, "Written Messages to the Public" will be revised.	RAI LTR 122 S4 response to 13.03-22A, B, C & E, item 3 SER with Open Items Confirmatory Item 13.3-5
3695	BLN	Pt 05		II.G.2	1. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.G.2, "Distribution and Maintenance of Public Information" will be revised	RAI LTR 122 S4 response to RAI 13.03-24A, B & C, item 1 SER with Open Items Confirmatory Item 13.3-6
3696	BLN	Pt 05		II.G.4	2. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.G.4, will be revised.	RAI LTR 122 S4 response to RAI 13.03-24A, B & C, item 2

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						SER with Open Items Confirmatory Item 13.3-7
6019	BLN	Pt 05		II.H.1	1. COLA Part 5, Emergency Plan, Section II.H.1, will be revised (in the discussion of the Operations Support Centers) to read: Designated plant support personnel, as indicated in Section II.B of this plan, assemble in the designated OSC to provide support to both the Control Room and TSC. The primary function of the OSC staff is to dispatch assessment, corrective action, and rescue personnel to locations in the plant as directed by the TSC and Control Room. TVA provides an OSC assembly area separate from the control room and the TSC. Personnel reporting to the OSC can be assigned duties in support of emergency operations. Personnel directed to perform emergency operational activities from the OSC are provided with adequate inventories of equipment and supplies, including protective clothing, respiratory protection and hand-held radios to ensure adequate communications, consistent with Appendix 6 of this Plan. The location and quantity of specific equipment and supplies is included in EPIP, "Emergency Equipment and Inventory," listed in Appendix 5.	COL-SER-OI-Ch13 response to OI 13.03-015 (eRAI 13.03-55) item 1
6005	BLN	Pt 05		II.H.4	COLA Part 5, Emergency Plan, Section II.H.4 will be revised from: Following declaration of an emergency condition, the ERFs are staffed and activated in accordance with EPIPs. To read: Following declaration of an emergency condition, the ERFs are staffed and activated in accordance with EPIP, "Activation of the Emergency Response Organization,' listed in Appendix 5, which provides specific timeliness goals for staffing each of the ERFs.	COL-SER-OI-Ch13 response to 13.03-001 (eRAI 13.03-41)
4811	BLN	Pt 05		II.H.5	6. COLA Part 5, Emergency Plan, Subsection II.H.5 will be revised to read: The bases for the Emergency Action Levels, as discussed in Appendix 1, describe the bases for the selection of the designated instruments as indicators of emergency conditions. [Appendix 1 is "Reserved" until after the NRC-endorsed version of NEI 07-01, Rev. 0 is available.]	SUPERSEDED by Qb 5338 - RAI LTR 146 response to RAI 13.03-033, item 6
5338	BLN	Pt 05		II.H.5	10. COLA Part 5, Emergency Plan, Subsection II.H.5 will be revised from: The bases for the Emergency Action Levels, as discussed in NEI 07-01, describe the bases for the selection of the designated instruments as indicators of emergency conditions. To read: BLN EPIP, "Emergency Classification," describes the bases for the selection of the designated instruments as indicators of emergency conditions.	BLN RAI LTR 146S response to RAI 13.03-040 item 10 - SUPERSEDES Qb 4811
4812	BLN	Pt 05		II.I.1	7. COLA Part 5, Emergency Plan, Subsection II.I.1, second sentence will be revised to read: Appendix 1 of this plan includes the various indications that correspond to the emergency initiating conditions based on the methodology provided in NEI 07-01. [Appendix 1 is "Reserved" until after the NRC-endorsed version of NEI 07-01, Rev. 0 is available.]	SUPERSEDED by Qb 5339 - RAI LTR 146 response to RAI 13.03-033, item 7
5339	BLN	Pt 05		II.I.1	11. COLA Part 5, Emergency Plan, Subsection II.I.1, will be revised from: Appendix 1 of this plan describes the plant system and effluent parameter values that are indicative of off-normal conditions. Appendix 1 of this plan includes the various indications that correspond to the emergency initiating conditions based on the methodology provided in NEI 07-01, Rev. 0. Plant procedures specify the types and capabilities of the instruments used to indicate emergency	BLN RAI LTR 146S response to RAI 13.03-040 item 11 - SUPERSEDES Qb 4812

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					conditions. To read: BLN EPIP, "Emergency Classification," describes the plant system and effluent parameter values that are indicative of off-normal conditions and includes the various indications that correspond to the emergency initiating conditions. Plant procedures specify the types and capabilities of the instruments used to indicate emergency conditions.	
3697	BLN	Pt 05		II.J.1	COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.J.1, last paragraph, will be revised from: TVA maintains the ability to notify individuals within the Protected Area within about 15 minutes of the declaration of any emergency requiring individual response actions, such as accountability or evacuation. To read: TVA maintains the ability to notify individuals within the Protected Area within about 15 minutes of the declaration of any emergency requiring individual response actions, such as accountability or evacuation. TVA expects notification of personnel located outside of the Protected Area to be completed within approximately one hour of the declaration of any emergency requiring individual response actions in those areas.	RAI LTR 122 S4 response to RAI 13.03-27A SER with Open Items Confirmatory Item 13.3-11
6046	BLN	Pt 05		II.J.2	COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.J.2 will be revised to read: The Shift Manager/Site Emergency Director or designee uses station and local area maps, information available from meteorological tower instrument readouts and current radiological data for determining the evacuation route. There are two access/egress roads at the Bellefonte Nuclear Plant site. Each of these roads intersects U.S. 72. The south exit route intersects with U.S. 72 approximately 1.5 miles east of the Bellefonte Nuclear Plant. The north exit route intersects with U.S. 72 approximately 1.5 miles north of the site. Provisions for evacuation of on-site individuals include evacuation by private automobile. The designated relocation site has decontamination and contamination control capability and equipment in the event it is needed. High traffic density is not considered in estimating evacuation times due to the sparsely populated area selected for the site. Should site evacuation via either designated evacuation route be determined to be inadvisable due to adverse conditions (e.g., weather-related, radiological, or traffic density conditions), affected individuals would be directed to a safe on-site area (as determined by the Site Emergency Director or his designee) for accountability and, if necessary, contamination monitoring and decontamination. Affected individuals evacuate the site via personal vehicles. If any individual on site does not have access to a personal vehicle, the Security Force makes arrangements for transportation with another evacuating individual. TVA directs evacuees to the designated assembly area. TVA informs individuals of the evacuation routes and appropriate instructions via plant training programs, visitor orientation, escort instructions, posted instructions, or within the content of audible messages. Appendix 8 of this plan provides a cross-reference to these provisions in State and local plans, as applicable.	COL-SER-OI-13.03-012 (OI 13.03-019) (eRAI 13.03-88)
6045	BLN	Pt 05		II.J.2	COLA Part 5, Emergency Plan, Section II.J.2 of the Emergency Plan will be revised as provided in Open Item 13.03-19 of this letter.	COL-SER-CI-13.03-012 (eRAI 13.03-008) - This COLA change is a duplicate

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						of Qb ID #6046
5340	BLN	Pt 05		II.J.7	12. COLA Part 5, Emergency Plan, Subsection II.J.7, third paragraph will be revised from: The Emergency Action Levels correspond to the projected dose to the population at risk and are determined consistent with the methodology discussed in NEI 07-01. To read: The Emergency Action Levels correspond to the projected dose to the population at risk and are determined consistent with the methodology discussed in BLN EPIP, "Emergency Classification."	BLN RAI LTR 146S response to RAI 13.03-040 item 12
6027	BLN	Pt 05		II.K.5	COLA Part 5, Emergency Plan, Section II.K.5 will be revised to read: TVA procedures establish requirements for decontamination of personnel, equipment, and areas when removable contamination levels exceed 1,000 disintegrations per minute per 100 square centimeters (dpm/100 cm2) beta-gamma or 20 dpm/100 cm2 alpha and release of the affected personnel, equipment, and areas from radiological controls is desirable. Items and areas may be returned to unrestricted use when removable contamination levels have been reduced below the stated guidelines. Some exceptions may be implemented for contaminated personnel under the direction of a Radiation Protection Supervisor. TVA implements requirements for personnel and area decontamination, including decontamination action levels and criteria for returning areas and items to normal use, in procedures supporting the radiation protection program. Decontamination methods are established in Radiation Protection procedures and are implemented under the direction of trained Radiation Protection personnel.	COL-SER-OI-Ch13 response to OI 13.03-022 (eRAI 13.03-61)
6028	BLN	Pt 05		II.K.6.a	COLA Part 5, Emergency Plan, Section II.K.6.a will be revised to read: The FSAR and Security Plan establish requirements for site access control. Following a site evacuation, law enforcement agencies control access to the owner-controlled area consistent with the requirements of the supporting State and local radiological emergency plans. Control of access to radiologically controlled areas, including contaminated areas, is provided by the Radiation Protection Program and its supporting procedures.	COL-SER-OI-Ch13 response to OI 13.03-023 (eRAI 13.03-62)
3698	BLN	Pt 05		II.K.6.b	COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.K.6.b, will be revised from: Should the potential exist for contamination of on-site food or drinking water supplies that renders these supplies non-consumable, the CECC staff make arrangements for transport of non-contaminated off-site supplies to the site. To read: Should the potential exist for contamination of on-site food or drinking water supplies that renders these supplies non-consumable, the Resource Support Coordinator in the CECC is responsible for making arrangements for transport of non-contaminated off-site supplies to the site. Food and water is made available on-site through acquisition of supplies under TVA commercial arrangements and subsequent transportation of supplies to the site, using either vendor or TVA-supplied transport. The Resources Support Coordinator is also responsible for making arrangements for distribution of food and water under emergency conditions.	RAI LTR 122 S4 response to RAI 13.03-28F SER with Open Items Confirmatory Item 13.3-13
3699	BLN	Pt 05		II.L.2	2. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.L.2, will be revised.	Partially SUPERSEDED by Qb 6021 - RAI LTR 122 S4 response to RAI 13.03-29B, item 2 SER with Open Items Confirmatory Item 13.3-14
6021	BLN	Pt 05		II.L.2	1. COLA Part 5, Emergency Plan, Section II.L.2 will be revised from (based on RAI response 13.03-	COL-SER-OI-Ch13

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					29(B) provided in February 6, 2009 letter): TVA maintains a trained Medical Emergency Response Team (MERT) at the site to provide 24 hour per day first aid support. TVA provides for MERT readiness through training consistent with Section II.O of this plan and drills and exercises consistent with Section II.N of this plan. First Aid stations are located throughout BLN providing the normal complement of first aid supplies and equipment necessary to treat those injuries not involving hospitalization or professional medical services. To read: TVA maintains a trained Medical Emergency Response Team (MERT) at the site to provide 24 hour per day first aid support. TVA provides for MERT readiness through training consistent with Section II.O of this plan and drills and exercises consistent with Section II.N of this plan. First Aid stations are located in the following locations and provide the normal complement of first aid supplies and equipment necessary to treat those injuries not involving hospitalization or professional medical services: • Auxiliary Building • Annex Building • Turbine Building • Maintenance Building • Administration Building • Training Building • Adjacent to the Spent Fuel Pool • Adjacent to the Reactor Cavity • Main Control Rooms • Technical Support Center • Operations Support Center • Central Emergency Control Center	response to OI 13.03-016 (eRAI 13.03-56) item 1
3700	BLN	Pt 05		II.M.2	1. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan Section II.M.2, third paragraph will be revised from: The decision to terminate and/or enter recovery from an incident for which onsite and offsite emergency response facilities have been activated is made by the SED after consultation with the plant technical and operations staffs and will be coordinated with the CECC Director. This decision is based upon a comprehensive review of plant status and system parameters. The State has the authority and responsibility for offsite recovery efforts. TVA provides assistance, as requested, through the recovery organization. Procedures and plans are then developed to implement the most expeditious recovery sequence to return the plant to normal operation. To read: The decision to terminate, or to terminate and enter recovery, from an incident for which onsite and offsite emergency response facilities have been activated is made by the SED with concurrence from the CECC Director and the Chief Nuclear Officer and Executive Vice President. The decision-making process includes consultation with the plant technical and operations staffs. This decision is based upon a comprehensive review of plant status and system parameters. The State has the authority and responsibility for offsite recovery efforts. TVA provides assistance, as requested, through the recovery organization. Procedures and plans are then developed to implement the most expeditious recovery sequence to return the plant to normal operation.	RAI LTR 122 S4 response to RAI 13.03-30A, B & D, item 1 SER with Open Items Confirmatory Item 13.3-15
3701	BLN	Pt 05		II.M.2	2. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.M.2, fifth paragraph will be revised.	RAI LTR 122 S4 response to RAI 13.03-30A, B & D, item 2

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
						SER with Open Items Confirmatory Item 13.3-16
3702	BLN	Pt 05		II.M.4	3. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.M.4, will be revised.	RAI LTR 122 S4 response to RAI 13.03-30A, B & D, item 3 SER with Open Items Confirmatory Item 13.3-17
6016	BLN	Pt 05		II.N.2.a	COLA Part 5, Emergency Plan, Section II.N.2.a will be revised to read: TVA tests communications with NRC Headquarters and the NRC Regional Operations Center from the Control Room, TSC, and CECC monthly. TVA tests communications with State and local governments within the Plume Exposure Pathway EPZ, as identified in Section II.A of this plan on a monthly basis.	COL-SER-OI-Ch13 response to OI 13.03.010
6017	BLN	Pt 05		II.N.2.a	COLA Part 5, Emergency Plan, Section II.N.2.a will be revised as provided in the response to SER Open Item 13.03-10 (this letter).	COL-SER-OI-Ch13 response to OI 13.03.011 (eRAI 13.03-51) - This COLA change is a duplicate of Qb ID #6016
3703	BLN	Pt 05		II.N.4	COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan, Section II.N.4, second paragraph, will be revised from: TVA makes arrangements for exercises to be critiqued by Federal, State, and local observers/evaluators. Specific areas to be evaluated by the facilitators are defined in the form of pre-printed critique sheets. To read: TVA designates individuals, called facilitators, who are responsible for guiding and evaluating drill and exercise performance. Specific areas to be evaluated by the facilitators are defined in the form of pre-printed critique sheets. TVA makes arrangements for exercises to be critiqued by Federal, State, and local observers/evaluators. Federal, State, and local observers/evaluators are encouraged to provide the results of their observations and evaluations directly to the TVA facilitators or formally during exercise critiques.	RAI LTR 122 S4 response to RAI 13.03-031A SER with Open Items Confirmatory Item 13.3-18
3704	BLN	Pt 05		II.O.2	1. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan Section II.O.2, On-site Emergency Response Training, will be revised.	RAI LTR 122 S4 response to RAI 13.03-032A & B, item 1 SER with Open Items Confirmatory Item 13.3-19
3705	BLN	Pt 05		II.O.3	2. COLA Part 5, Bellefonte Nuclear Plant Units 3 & 4 COL Application Emergency Plan Section II.O.3, First Aid Team Training, will be revised from: MERT members assigned to render treatment during a medical emergency receive training from the Safety and Emergency Response Training Academy, consistent with the projected hazards and events. To read: Members of TVA's Medical Emergency Response Team (MERT) complete a 120-hour program and are certified by the National Registry of Emergency Medical Technicians to meet the educational and examination requirements set forth by the U.S. Department of Transportation guidelines and the	RAI LTR 122 S4 response to RAI 13.03-032A & B, item 2 SER with Open Items Confirmatory Item 13.3-20

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					curriculum implemented by the National Highway Traffic Safety Administration under the National Emergency Medical Services Scope of Practice Model.	
4813	BLN	Pt 05		III.A	8. COLA Part 5, Emergency Plan, Subsection III.A, Reference 6 will be revised from: Nuclear Energy Institute, "Methodology for Development of Emergency Action Levels, Advanced Passive Light Water Reactors," NEI 07-01, Rev. 0, September 2007. To read: [Reserved]	SUPERSEDED by Qb 5341 - RAI LTR 146 response to RAI 13.03-033, item 8
5341	BLN	Pt 05		III.A	13. COLA Part 5, Emergency Plan, Subsection III.A, Reference 6 will be revised from: Nuclear Energy Institute, "Methodology for Development of Emergency Action Levels, Advanced Passive Light Water Reactors," NEI 07-01, Rev. 0, September 2007. To read: [Not used]	BLN RAI LTR 146S response to RAI 13.03-040 item 13 - SUPERSEDES Qb 4813
5342	BLN	Pt 05		III.B	14. COLA Part 5, Emergency Plan, Subsection III.B, Appendices will be revised from: Appendix 1 – Emergency Action Levels To read: Appendix 1 – Not Used	BLN RAI LTR 146S response to RAI 13.03-040 item 14
5343	BLN	Pt 05		III.B	15. COLA Part 5, Emergency Plan, Subsection III.B, Appendices will be revised from: Appendix 5 – Emergency Plan Implementing Procedures – Topical List To read: Appendix 5 – Emergency Plan Implementing Procedures	BLN RAI LTR 146S response to RAI 13.03-040 item 15
4814	BLN	Pt 05		x-App01	9. COLA Part 5, Emergency Plan, Appendix 1 – Emergency Action Levels will be revised by changing the cover page from: Appendix 1 – Emergency Action Levels To read: Appendix 1 – Emergency Action Levels [Reserved]	SUPERSEDED by Qb 5344 - RAI LTR 146 response to RAI 13.03-033, item 9
5344	BLN	Pt 05		x-APP01	16. COLA Part 5, Emergency Plan, Appendix 1 – Emergency Action Levels will be revised by changing the cover page from: Appendix 1 – Emergency Action Levels To read: Appendix 1 [Not Used]	BLN RAI LTR 146S response to RAI 13.03-040 item 16 - SUPERSEDES Qb 4814
4815	BLN	Pt 05		x-App01	10. COLA Part 5, Emergency Plan, Appendix 1 – Emergency Action Levels will be revised by deleting Pages A1-2 through A1-97.	SUPERSEDED by Qb 5345 - RAI LTR 146 response to RAI 13.03-033, item 10
5345	BLN	Pt 05		x-APP01	17. COLA Part 5, Emergency Plan, Appendix 1 – Emergency Action Levels will be revised by deleting Pages A1-2 through A1-97.	BLN RAI LTR 146S response to RAI 13.03-040

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
						item 17 - SUPERSEDES Qb 4815
6026	BLN	Pt 05		x-App04	COLA Part 5, Emergency Plan, Appendix 4 will be revised to include new Figure A4-8 and new Table A4-4	COL-SER-OI-Ch 13 response to OI 13.03-021 (eRAI 13.03-60)
6486	BLN	Pt 05		x-App05	COLA Part 5, Emergency Plan, Appendix 5 Cover Page will be revised from: Appendix 5 - Emergency Plan Implementing Procedures - Topical List To read: Appendix 5 - Emergency Plan Implementing Procedures	Editorial
6007	BLN	Pt 05		x-App05	COLA Part 5, Emergency Plan, Appendix 5 will be replaced with Appendix 5 provided in Attachment OI 13.03-03 of this letter	COL-SER-OI-CH13 response to OI 13.03-003 (eRAI 13.03-43)
6023	BLN	Pt 05		x-App05	COLA Part 5, Emergency Plan, Appendix 5 will be replaced with Appendix 5 provided in Attachment OI 13.03-03 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-017 (eRAI 13.03-57) - This COLA change is a duplicate of Qb ID #6007
6024	BLN	Pt 05		x-App05	COLA Part 5, Emergency Plan, Appendix 5 will be replaced with Appendix 5 provided in Attachment OI 13.03-03 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-018 (eRAI 13.03-58) - This COLA change is a duplicate of Qb ID #6007
6029	BLN	Pt 05		x-App05	COLA Part 5, Emergency Plan, Appendix 5 will be replaced with Appendix 5 provided in Attachment OI 13.03-03 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-024 (eRAI 13.03-63) - This COLA change is a duplicate of Qb ID #6007
6030	BLN	Pt 05		x-App05	COLA Part 5, Emergency Plan, Appendix 5 will be replaced with Appendix 5 provided in Attachment OI 13.03-03 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-025 (eRAI 13.03-64) - This COLA change is a duplicate of Qb ID #6007
6043	BLN	Pt 05		x-App05	COLA Part 5, Emergency Plan, Appendix 5 will be replaced with Appendix 5 provided in Attachment OI 13.03-03 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-009 (eRAI 13.03-85) - This COLA change is a duplicate of Qb ID #6007
6044	BLN	Pt 05		x-App05	COLA Part 5, Emergency Plan, Appendix 5 will be replaced with Appendix 5 provided in Attachment OI 13.03-03 of this letter	COL-SER-OI-Ch13 response to CI 13.03-010 (eRAI 13.03-86) - This COLA change is a duplicate of Qb ID #6007
6020	BLN	Pt 05		x-App06	2. COLA Part 5, Emergency Plan, Appendix 6, 5th, 7th, and 16th bulleted items will be revised to read: • Protective clothing (coveralls, rubber overshoes, rubber gloves, surgeon caps, hoods, cotton glove inserts, and booties) • Respiratory protection equipment (full-face respirators with particulate filters and iodine cartridges, and self-contained breathing apparatuses (SCBAs)	COL-SER-OI-Ch13 response to OI 13.03-015 (eRAI 13.03-55) item 2

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					• Communications equipment (plant page system, hand-held radios, telephones, facsimile)	
6022	BLN	Pt 05		x-App06	2. COLA Part 5, Emergency Plan, Appendix 6, bulleted item will be revised to read: • First aid supplies, including (at a minimum): o Absorbent compresses o Adhesive bandages o Adhesive tape o Antiseptic o Burn treatment o Medical exam gloves o Sterile pads o Triangular bandages o Bandage compresses o Eye coverings with means of attachment o Eye wash o Cold pack o Roller bandages	COL-SER-OI-Ch13 response to OI 13.03-016 (eRAI 13.03-56) item 2
5792	BLN	Pt 05		x-APP07	COLA Part 5, Emergency Plan, Appendix 7, will be revised by deleting placeholder Page A7-3.	Editorial
6042	BLN	Pt 05		x-App10	COLA Part 5, Emergency Plan, Appendix 10 will be revised to read: Habitability The ventilation system is operated in accordance with approved procedures and is manually controlled from the TSC. Equipment and supplies are provided in accordance with Appendix 6 of the Emergency Plan. Permanent and portable radiation monitoring systems are available to personnel in the TSC to provide radiological protection of TSC personnel. These systems continuously indicate radiation dose rates and airborne radioactivity concentrations inside the TSC while in use during an emergency. These monitoring systems include local alarms with trip levels set low enough to provide early warning to TSC personnel of adverse conditions that may affect the habitability of the TSC. Detectors are able to distinguish the presence of radioiodine at concentrations as low as 10-7 microcuries/cc. The ventilation system includes high efficiency particulate air (HEPA) filters and charcoal filters. The ventilation system is designed to maintain exposures at or below 0.05 Sv (5 rem) total effective dose equivalent (TEDE) as defined in 10 CFR 50.2 for the duration of an accident. In conclusion, the TSC structure, shielding, and ventilation system are designed to protect the TSC personnel from radiological hazards.	COL-SER-OI-Ch13 response to OI 13.03-008 (eRAI 13.03-84)
2560	BLN	Pt 05		z-ETE Supp	COLA Part 5, EP, ETE will be updated to reflect a change to Footnote 4 to the table on ETE Page 2-7. The change is provided in the enclosed Supplement 1, Revision 1 to the Bellefonte ETE Report, Revision 1, which is included as Attachment 13.03-01A.	RAI LTR 069 S2 response to RAI 13.03-12
2561	BLN	Pt 05		z-ETE Supp	COLA Part 5, EP, ETE will be updated to include: 1) Updated and corrected tables and figures in ETE Section 5, including explanatory text, 2) Corrected ETE Table 7-2, 3) Supplemented and corrected Section 8.3 and Table 8-4, 4) Supplemented and corrected Table 8-2 and table on page E-3, 5) Revised Table G-1, and 6) Revised schematics for TCPs 5-1, 5-2, and 11-13. These changes are provided in the enclosed Supplement 1, Revision 1 to the Bellefonte ETE Report	RAI LTR 069 S2 response to RAI 13.03-13

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					which is included as Attachment 13.03-01A.	
Pt 07						**7 COLA Changes**
3504	BLN	Pt 07		A	24. COLA Part 7, Section A, STD and BLN Departures, will be revised to add the following new departure information. Departure Number Description BLN DEP 2.3-1 EAB atmospheric dispersion value	RAI LTR 129S response to RAI 15.00.03-001, item 24
3505	BLN	Pt 07		A.2	25. COLA Part 7, Section A.2, Departures That Require NRC Approval Prior To Implementation, will be revised to add new departure information for BLN DEP 2.3-1 - EAB atmospheric dispersion value	RAI LTR 129S response to RAI 15.00.03-001, item 25
3506	BLN	Pt 07		B	26. COLA Part 7, Section B, BLN Exemption Requests, will be revised from: TVA requests the following exemptions related to: 1) Fitness for Duty Program Description, and 2) Combined License Application Organization and Numbering To read: TVA requests the following exemptions related to: 1) Fitness for Duty Program Description, and 2) Combined License Application Organization and Numbering 3) Containment leak rate technical specification, and 4) AP1000 DCD Tier 1 EAB atmospheric dispersion site parameter.	RAI LTR 129S response to RAI 15.00.03-001, item 26
4953	BLN,STD	Pt 07		B	6. Change COLA Part 7, Departures and Exemptions, Section B, Introductory statements to omit the listing for the 1) Fitness for Duty Program Description, by revising the current item 1 from: 1) Fitness for Duty Program Description, and To read: 1) Not used, and	BLN-P02-VOL-SEC-FFD-20090323-OR item 6
4954	BLN,STD	Pt 07		B	7. Change COLA Part 7, Departures and Exemptions, 1) Fitness for Duty Program Description (10 CFR Part 26), by deleting the entire text for this exemption request, and replacing it with the following statement: Withdrawn – this exemption is no longer required.	BLN-P02-VOL-SEC-FFD-20090323-OR item 7
3507	BLN	Pt 07		B	27. COLA Part 7, Section B, Exemptions, will be revised to add new exemption information.	RAI LTR 129S response to RAI 15.00.03-001, item 27
5048	BLN	Pt 07		B	1. COLA Part 7, Section B, Exemptions, will be revised (from the wording added per Change 27 of the February 2, 2009, supplemental response to BLN-RAI-LTR-129) to delete the following sentence in exemption request 3 and in exemption request 4 with reference to the hardship criteria: Additionally, special circumstance (iii) is present, since compliance would necessitate expanding the exclusion area boundary, which would result in undue hardship or other costs that are significantly in excess of those contemplated when the regulation was adopted.	RAI LTR 157 in response to RAI 06.02.06-002
Pt 09						**10 COLA Changes**
5859	BLN	Pt 09		09.02-01.02F / F1.2-201	COLA Part 9, Withheld Information, is revised to reflect changes to DCD figure 1.2-18, in WEC DCD	WEC DCD Rev 17

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					Rev 17, except that Room 40318 should be only ALARA BRIEFING RM.	conforming change
3861	BLN	Pt 09		09.02-02.02.T / T2.2-208	9. COLA Part 9, Withheld Information, will be revised to include a complete Table 2.2-208 as shown in Attachment 02.02.03-08B of this RAI response.	RAI LTR 132 S1 response to RAI 02.02.03-008, item 9
3847	BLN	Pt 09		09.02-02.02.T / T2.2-215	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-215 (both pages) will be revised to include the LMA of BLN COL 2.2-1	Editorial
3848	BLN	Pt 09		09.02-02.02.T / T2.2-216	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-216 (all pages) will be revised to include the LMA of BLN COL 2.2-1	Editorial
3849	BLN	Pt 09		09.02-02.02.T / T2.2-217	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-217 (all pages) will be revised to include the LMA of BLN COL 2.2-1	Editorial
3850	BLN	Pt 09		09.02-02.02.T / T2.2-220	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-220 will be revised to include the LMA of BLN COL 2.2-1	Editorial
3851	BLN	Pt 09		09.02-02.02.T / T2.2-221	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-221 will be revised to include the LMA of BLN COL 2.2-1	Editorial
3852	BLN	Pt 09		09.02-02.02.T / T2.2-222	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-222 will be revised to include the LMA of BLN COL 2.2-1	Editorial
3853	BLN	Pt 09		09.02-02.02.T / T2.2-223	COLA Part 2, FSAR Chapter 2, Section 2.2, Table 2.2-223 will be revised to include the LMA of BLN COL 2.2-1	Editorial
5860	BLN	Pt 09		09.02-12.03 / F12.3-201, -202, -203	COLA Part 9, Withheld Information, is revised to reflect changes to DCD Figure 12.3-1, Sheet 11 of 16, Figure 12.3-2, Sheet 11 of 15, and Figure 12.3-3, Sheet 11 of 16, respectiviely, in WEC DCD Rev 17.	WEC DCD Rev 17 conforming change
Pt 10						**51 COLA Changes**
6394	BLN,STD	Pt 10		LC#02 09.01-07	2- COLA Part 10, License Conditions, COL Item No. 9.1-7 will be revised to read: A spent fuel rack Metamic coupon monitoring program is to be implemented when the plant is placed into commercial operation. This program includes tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and / or visual examination. The program will also include tests to monitor changes in physical properties of the absorber material, including neutron attenuation and thickness measurements.	SUPERSEDED by REVISED wording per Qb 6375 - RAI LTR 165 S1 response to RAI 09.01.02-001 item 1
5863	BLN,STD	Pt 10		LC#02, 09.01-07	2- COLA Part 10, License Conditions, COL Item No. 9.1-7 will be revised from: A spent fuel rack Metamic coupon monitoring program is to be implemented when the plant is placed into commercial operation. This program includes tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and or visual examination. To read: A spent fuel rack Metamic coupon monitoring program is to be implemented when the plant is placed into commercial operation. This program includes tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and / or visual examination. The program will also include tests to monitor changes in physical properties of the absorber material, including neutron attenuation and thickness measurements.	RAI LTR 165 in response to RAI 09.01.02-001 item 2 Duplicate of QB Item #5477
5901	BLN,STD	Pt 10		LC#02, 09.01-07	Revise Part 10, License Condition 2, COL Item No. 9.1-7, from: A spent fuel rack Metamic coupon monitoring program is to be implemented when the plant is placed into commercial operation. This program includes tests to monitor bubbling, blistering, cracking, or	RAI LTR 165 Supp 1 in response to RAI 09.01.02-001 item 1

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					flaking; and a test to monitor for corrosion, such as weight loss measurements and or visual examination. To read: A spent fuel rack Metamic coupon monitoring program is to be implemented when the plant is placed into commercial operation. This program includes tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and or visual examination. The program also includes test to monitor changes in physical properties of the absorber material, including neutron attenuation and thickness measurements.	Duplicate of QB Item #5477
6375	BLN	Pt 10		LC#02, 09.01-07	Revise Part 10, License Condition 2, COL Item No. 9.1-7, To read: A spent fuel rack Metamic coupon monitoring program will be implemented when the plant is placed into commercial operation. This program will include tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and/or visual examination. The program will also include testing to monitor changes in physical properties of the absorber material, including neutron attenuation and thickness measurements.	Editorial revision to RAI LTR 165 (& Supp 1) in response to RAI 09.01.02-001 item 1
2599	BLN,STD	Pt 10		LC#02, 14.04-03	3. COLA Part 10 will be revised To read: COL Item No. / Subject / From DCD Tier 2 Subsection / Implementation Milestone 14.4-3 / Conduct of Test Program / 14.4.3 / Prior to initiating test program A site-specific startup administration manual (procedure), which contains the administration procedures and requirements that govern the activities associated with the plant initial test program, as identified in FSAR Section 14.2, is provided prior to initiating the plant initial test program.	RAI LTR 139 response to RAI 14.02-012, item 3 SER with Open Items Confirmatory Item 14.2-1
4937	BLN,STD	Pt 10		LC#02, 19.59.10-01	3. COLA Part 10, License Conditions and ITAAC, BLN Proposed License Condition 2, COL Holder Items, first paragraph, will be revised to read: 19.59.10-1 / As-Built SSC HCLPF Comparison to Seismic Margin Evaluation / 19.59.10.5 / Prior to initial fuel load The Combined License holder referencing the AP1000 certified design will review differences between the as-built plant and the design used as the basis for the AP1000 seismic margins analysis prior to fuel load. A verification walkdown will be performed with the purpose of identifying differences between the as-built plant and the design. Any differences will be evaluated and the seismic margins analysis modified as necessary to account for the plant specific-design, and any design changes or departures from the certified design. Spacial interactions are addressed by COL information item 3.7-3. Details of the process will be developed by the Combined License holder.	RAI LTR 152 response to RAI 19-20 item 3 SER with Open Items Confirmatory Item 19.59-1
4938	BLN,STD	Pt 10		LC#02, 19.59.10-02	4. COLA Part 10, License Conditions and ITAAC, BLN Proposed License Condition 2, COL Holder Items, will be revised to read: 19.59.10-2 / Evaluation of As-Built Plant Versus Design in AP1000 PRA and Site-Specific PRA External Events / 19.59.10.5 / Prior to initial fuel load The Combined License holder referencing the AP1000 certified design will review differences between the as-built plant and the design used as the basis for the AP1000 PRA and Table 19.59-18 prior to fuel load. The plant specific PRA-based insight differences will be evaluated and the plant specific PRA model modified as necessary to account for the plant specific-design and, any design changes or departures from the design certification PRA.	RAI LTR 152 response to RAI 19-20 item 4 SER with Open Items Confirmatory Item 19.59-1

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
4939	BLN,STD	Pt 10		LC#02, 19.59.10-03	5. COLA Part 10, License Conditions and ITAAC, BLN Proposed License Condition 2, COL Holder Items, will be revised to read: 19.59.10-3 Internal Fire and Internal Flood Analyses 19.59.10.5 Prior to initial fuel load The Combined License holder referencing the AP1000 certified design will review differences between the as-built plant and the design used as the basis for the AP1000 internal fire and internal flood analyses prior to fuel load. Plant specific internal fire and internal flood analyses will be evaluated and the analyses modified as necessary to account for the plant-specific design, and any design changes or departures from the certified design	RAI LTR 152 response to RAI 19-20 item 5 SER with Open Items Confirmatory Item 19.59-1
4941	BLN,STD	Pt 10		LC#02, 19.59.10-04	2. COLA Part 10, License Conditions and ITAAC, BLN Proposed License Condition 2, Col Holder Items, will be revised to read: 19.59.10-4 Implement Severe Accident Management Guidance 19.59.10.5 Prior to startup testing	RAI LTR 152 response to RAI 19-21 item 2 SER with Open Items Confirmatory Item 19.59-2
4918	BLN,STD	Pt 10		LC#02, 19.59.10-05	COLA Part 10, License Condition 2, COL item No. 19.59-10-5, text will be revised to include the remaining text of the COL item To read: The Combined License holder referencing the AP1000 certified design will perform a thermal lag assessment of the as-built equipment listed in Tables 6b and 6c in Attachment A of APP-GW-GLR-069 to provide additional assurance that this equipment can perform its severe accident functions during environmental conditions resulting from hydrogen burns associated with severe accidents. This assessment is performed prior to fuel load and is required only for equipment used for severe accident mitigation that has not been tested at severe accident conditions. The Combined License holder will assess the ability of the as-built equipment to perform during severe accident hydrogen burns using the Environment Enveloping method or the Test Based Thermal Analysis method discussed in EPRI NP-4354.	Consistency with WEC DCD AP1000 COL item
5142	BLN,STD	Pt 10		LC#03	COLA Part 10, Proposed License Conditions (Including ITAAC), Proposed License Condition 3, introductory statement, will be revised To read: The licensee shall implement the programs or portions of programs identified below on or before the associated milestones identified below.	Editorial
5141	BLN	Pt 10		LC#03 C	COLA Part 10, Proposed License Conditions (Including ITAAC), Proposed License Condition 3, item C introduction, will be revised from: C. Receipt of materials To read: C. Receipt of Materials	Editorial
5355	BLN	Pt 10		LC#03 C	5. COLA Part 10, Proposed License Condition 3, Operational Program Implementation, will be revised to add the following new milestones: C.2 – Fire Protection Program (applicable portions) C.3 – Non Licensed Plant Staff Training Program (applicable portions) C.4 – Emergency Planning (applicable portions) C.5 – Security Program (applicable portions)	COL-SER-OI-Ch01 response to OI 01.05-01 item 5
4925	BLN	Pt 10		LC#03 G	COLA Part 10, Proposed License Conditions (Including ITAAC), Proposed License Condition 3, item G.3, will be revised from: G.3 - Process and Effluent monitoring and Sampling To read: G.3 - Process and Effluent Monitoring and Sampling	Editorial
4994	BLN	Pt 10		LC#03 G	COLA Part 10, Proposed License Conditions (Including ITAAC), Proposed License Condition 3, will be	Editorial

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					revised from: G.8 - Containment Leakage Rate Testing Program To read: G.8 - Containment Leakage Rate Testing G.9 - Physical Security G.10 - Cyber Security	
5684	BLN	Pt 10		LC#03 G	5. Change COLA Part 10, Proposed License Conditions (Including ITAAC), by adding the Cyber Security Program to the list of operational programs that are required to be implemented prior to initial fuel load. The following will be inserted after the programs currently listed under Proposed License Condition 3.G, Fuel Loading: G.10 – Cyber Security	VOL-SEC-CYBER-20090811 item 5 (Note - partially DUPLICATES Qb 4994)
4816	BLN	Pt 10		LC#04	11. COLA Part 10, Proposed License Conditions (Including ITAAC), Proposed License Condition 4, will be revised from: 4. Not used To read: 4. EMERGENCY PLANNING ACTIONS: Because various equipment set points and other information cannot be determined until the as-built information is available, the COL Application does not fully address certain aspects of the EAL scheme. Thus, COL applicants using EAL schemes in accordance with NEI 07-01 are proposing the following license condition, or similar. PROPOSED LICENSE CONDITION: The licensee shall submit a fully developed set of site-specific Emergency Action Levels (EALs) to the NRC in accordance with the NRC-endorsed version of NEI 07-01, Revision 0, with no deviations. These fully developed EALs shall be submitted to the NRC for confirmation at least 180 days prior to initial fuel load.	SUPERSEDED by Qb 5346 - RAI LTR 146 response to RAI 13.03-033, item 11
5346	BLN	Pt 10		LC#04	18. COLA Part 10, Proposed License Conditions (Including ITAAC), Proposed License Condition 4, will be revised from: 4. Not used To read: 4. EMERGENCY PLANNING ACTIONS: The COL Application does not fully address certain aspects of the EAL scheme because various equipment setpoints and other information cannot be determined until the as-built information is available. Thus, COL applicants using EAL schemes in accordance with NEI 07-01 are proposing the following license condition. PROPOSED LICENSE CONDITION: The licensee shall submit a fully developed set of site-specific Emergency Action Levels (EALs) to the NRC in accordance with the NRC-endorsed version of NEI 07-01, Rev. 0, with no deviations. These fully developed EALs shall be submitted to the NRC for confirmation at least 180 days prior to initial fuel load.	SUPERSEDED by Qb 6006 - BLN RAI LTR 146S response to RAI 13.03-040 item 18
6006	BLN	Pt 10		LC#04	COLA Part 10, Proposed License Condition 4, will be revised from (as included in supplemental response to BLN-RAI-LTR-063, dated July 13, 2009) to read: 4. EMERGENCY PLANNING ACTIONS:	COL-SER-OI-Ch13 response to OI 13.03-002 (eRAI 13.03-42)

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					The COL Application does not contain final versions of some implementation aspects of emergency planning such as EALs and Letters of Agreement because the information will not be developed until it is necessary to implement those aspects of the plan. Thus, COL applicants are proposing the following license condition. PROPOSED LICENSE CONDITION: A. The licensee shall submit a fully developed set of site-specific Emergency Action Levels (EALs) to the NRC in accordance with the NRC-endorsed version of NEI 07-01, Rev. 0, with no deviations. These fully developed EALs shall be submitted to the NRC for confirmation at least 180 days prior to initial fuel load. B. Prior to the full participation exercise to be conducted in accordance with the requirements of Appendix E to 10 CFR Part 50, TVA shall establish Letters of Agreement with the following entities: a. Alabama Department of Public Health b. Alabama Emergency Management Agency c. Jackson County Emergency Management Agency d. DeKalb County Emergency Management Agency. These Letters of Agreement will identify the specific nature of arrangements in support of emergency preparedness for operation of the proposed new nuclear units and certify the agency's concurrence with the emergency action levels described in Bellefonte Units 3 & 4 Combined License Application Emergency Plan Implementing Procedure, "Emergency Classification."	
6010	BLN	Pt 10		LC#04	COLA Part 10, Proposed License Condition 4 (see response to OI 13.03-02, this letter), will be revised to include: C. Prior to the full participation exercise to be conducted in accordance with the requirements of Appendix E to 10 CFR Part 50, TVA shall establish Letters of Agreement with engineering and technical services firms that may be requested to provide engineering and technical support during an emergency. These Letters of Agreement will identify the emergency response capabilities.	COL-SER-OI-Ch13 response to OI 13.03-006 (eRAI 13.03-46)
6011	BLN	Pt 10		LC#04	COLA Part 10, Proposed License Condition 4 (see response to OI 13.03-02, this letter), will be revised to include: D. Prior to the full-participation exercise to be conducted in accordance with the requirements of Appendix E to 10 CFR Part 50, TVA shall establish Letters of Agreement with the following organizations: a. Institute of Nuclear Power Operations b. Westinghouse Electric Corporation c. Radiation Emergency Assistance Center/Training Site These Letters of Agreement will identify the emergency response capabilities.	COL-SER-OI-Ch13 response to OI 13.03-007 (eRAI 13.03-47)
6012	BLN	Pt 10		LC#04	COLA Part 10, Proposed License Condition 4 will be revised as shown in the response to SER Open Item 13.03-02 (this letter).	COL-SER-OI-Ch13 response to OI-13.03-008 (eRAI 13.03-48) - This COLA Change is a duplicate of Qb ID #6006
6013	BLN	Pt 10		LC#04	COLA Part 10, Proposed License Condition 4 will be revised as shown in the response to SER Open Item 13.03-02 (this letter).	COL-SER-OI-Ch13 response to OI 13.03-009 (eRAI 13.03-49) - This

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
						COLA change is a duplicate of Qb ID #6006
6018	BLN	Pt 10		LC#04	COLA Part 10, Proposed License Condition 4, (see response to OI 13.03-02, this letter), will be revised to include: E. Prior to fuel load, TVA shall: a. Distribute written information to permanent residences within the plume exposure pathway Emergency Planning Zone (EPZ). b. Provide public postings at locations within the EPZ as agreed to with Jackson and DeKalb County Emergency Management. c. Distribute publications to hotels, motels, and campgrounds within the EPZ. d. Verify information has been published in telephone directories distributed within the EPZ.	COL-SER-OI-Ch13 response to OI 13.03-012 (eRAI 13.03-52)
6025	BLN	Pt 10		LC#04	COLA Part 10, Proposed License Condition 4 (see response to OI 13.3-02, this letter), will be revised to include: PROPOSED LICENSE CONDITION: F. Prior to the full-participation exercise required by Section IV.F.2 of Appendix E to 10 CFR Part 50, TVA shall identify the specific locations of shelter areas and reception centers, on a map, for Bellefonte Nuclear Plant workers evacuated from the site during an emergency.	COL-SER-OI-Ch13 response to OI 13.03-020 (eRAI 13.03-59)
3711	BLN	Pt 10		LC#06	COLA Part 10, Proposed License Condition 6, will be revised from: a. This schedule shall include a submittal schedule for the emergency planning implementing procedures to the NRC consistent with 10 CFR Part 50, Appendix E, Section V. b. This schedule shall include a schedule for the development of a site specific Severe Accident Management Guidance. c. This schedule shall include a submittal schedule for the reactor vessel pressurized thermal shock evaluation at least 18 months prior to initial fuel load. d. This schedule shall include a submittal schedule for approved preoperational and startup test procedures in accordance with FSAR Section 14.2.3. To read: This schedule shall include a submittal schedule for: a. the emergency planning implementing procedures to the NRC consistent with 10 CFR Part 50, Appendix E, Section V. b. the development of a site specific Severe Accident Management Guidance. c. a reactor vessel pressurized thermal shock evaluation at least 18 months prior to initial fuel load. d. approved preoperational and startup test procedures in accordance with FSAR Subsection 14.2.3. e. an emergency response data system (ERDS) implementation program plan consistent with 10 CFR Part 50, Appendix E, Section VI.	SUPERSEDED by Qb 5308 - RAI LTR 122 S5 response to RAI 13.03-025B & D
4942	BLN,STD	Pt 10		LC#06	3. COLA Part 10, License Conditions and ITAAC, BLN Proposed License Condition 6, Operational Program Readiness, will be revised to read: b. This schedule shall include a schedule for the implementation of a site specific Severe Accident Management Guidance.	SUPERSEDED by Qb 5308 - RAI LTR 152 response to RAI 19-21 item 3 SER with Open Items Confirmatory Item 19.59-2
5308	BLN	Pt 10		LC#06	COLA Part 10, proposed License Condition 6, will be revised to read: 6. OPERATIONAL PROGRAM READINESS: The NRC inspection of operational programs will be the subject of the following license condition in	COL-SER-OI-Ch10 response to OI 10.01-01

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					accordance with SECY-05-0197. PROPOSED LICENSE CONDITION: The licensee shall submit to the appropriate Director of the NRC, a schedule, no later than 12 months after issuance of the COL, that supports planning for and conduct of NRC inspections of operational programs listed in the operational program FSAR Table 13.4-201. The schedule shall be updated every 6 months until 12 months before scheduled fuel loading, and every month thereafter until either the operational programs in the FSAR table have been fully implemented or the plant has been placed in commercial service, whichever comes first. This schedule shall include a submittal schedule for: a. the emergency planning implementation procedures to the NRC consistent with 10 CFR Part 50, Appendix E, Section V. b. the implementation of site specific Severe Accident Management Guidance. c. a reactor vessel pressurized thermal shock evaluation at least 18 months prior to initial fuel load. d. approved preoperational and startup test procedures in accordance with FSAR Subsection 14.2.3. e. an emergency response data system (ERDS) implementation program plan consistent with 10 CFR Part 50, Appendix E, Section V. f. a flow accelerated corrosion (FAC) program implementation schedule, including the construction phase activities.	
6388	BLN	Pt 10		LC#06	COLA Part 10, proposed License Condition 6, will be revised from: This schedule shall include a submittal schedule for: a. the emergency planning implementation procedures to the NRC consistent with 10 CFR Part 50, Appendix E, Section V. b. the implementation of site specific Severe Accident Management Guidance. c. a reactor vessel pressurized thermal shock evaluation at least 18 months prior to initial fuel load. d. approved preoperational and startup test procedures in accordance with FSAR Subsection 14.2.3. e. an emergency response data system (ERDS) implementation program plan consistent with 10 CFR Part 50, Appendix E, Section V. f. a flow accelerated corrosion (FAC) program implementation schedule, including the construction phase activities. To read: This schedule shall address: a. the implementation of site specific Severe Accident Management Guidance. b. the reactor vessel pressurized thermal shock evaluation at least 18 months prior to initial fuel load. c. the approved preoperational and startup test procedures in accordance with FSAR Subsection 14.2.3. d. the flow accelerated corrosion (FAC) program implementation, including the construction phase activities.	Editorial
4835	BLN	Pt 10		LC#07	2. COLA Part 10, License Conditions, item 7, will be revised to read: 7. Not used	RAI LTR 148 response to RAI 01-13 item 2 SER with Open Items Confirmatory Item 1.4-1
6031	BLN	Pt 10		LC-B / T3.8-1	COLA Part 10, Appendix B, Table 3.8-1 will be replaced with Table 3.8-1 provided in Attachment OI 13.03-27 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-027

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
						(eRAI 13.03-66)
6514	BLN	Pt 10		LC-B / T3.8-1	COLA Part 10, Appendix B, Table 3.8-1 page numbering is revised to reflect the continuation of Appendix B, License Conditions to read LC-BX where X is the sequential page number.	Editorial
4957	BLN,STD	Pt 10		LC-B Phs Sec	1. Change COLA Part 10, Proposed License Conditions (Including ITAAC), Appendix B, Inspections, Tests, Analysis, and Acceptance Criteria, by changing the heading and text for Physical Security ITAAC, to read: Physical Security ITAAC The physical security ITAAC that are in the scope of the Westinghouse AP1000 standard design are included in the referenced DCD Tier 1 Subsection 2.6.9 as incorporated by reference above. Sitespecific physical security ITAAC that are outside the scope of the Westinghouse AP1000 standard design in DCD Tier 1 Subsection 2.6.9 are provided in the attached Table 2.6.9-2. Include these ITAAC after the DCD Tier 1 Table 2.6.9-1 ITAAC.	BLN-P02-VOL-SEC-FFD-20090323-OR
4959	BLN,STD	Pt 10		LC-B Phs Sec	3. Change COLA Part 10, Proposed License Conditions (Including ITAAC), Appendix B, Inspections, Tests, Analysis, and Acceptance Criteria, by adding the new Table 2.6.9-2 - SITE-SPECIFIC PHYSICAL SECURITY INSPECTIONS, TESTS, ANALYSES AND ACCEPTANCE CRITERIA after page LC-B1.	BLN-P02-VOL-SEC-FFD-20090323-OR
4919	BLN	Pt 10		LC-B Pl Sp	COLA Part 10, Proposed License Conditions (Including ITAAC), Appendix B, Plant Specific ITAAC, will be revised to include a new item that reads: 2.3.32 Yard Fire Water System No entry for this system.	Editorial
5096	BLN,STD	Pt 10		LC-B Pl Sp	4. COLA Part 10, Appendix B, is revised to include the following new site-specific ITAAC from: Add the following information to the information provided in the referenced DCD Tier 1 following Section 2.6.11: 2.6.12 Transmission Switchyard and Offsite Power System No entry for this system. To read: Add the following information to the information provided in the referenced DCD Tier 1 following Section 2.6.11: 2.6.12 Transmission Switchyard and Offsite Power System Inspection, Test, Analysis and Acceptance Criteria Table 2.6.12-1 provides a definition of the inspections, tests, and/or analyses, together with associated acceptance criteria for the offsite power system. Table 2.6.12-1 Offsite Power System [For table information see supplemental response to RAI LTR 027 - include Table 2.6.12-1 after Table 2.6.9-1.]	RAI LTR 027 S1 response to RAI 14.03-001 item 4 SER with Open Items Confirmatory Item 8.2A-1 SER with Open Items Confirmatory Item 14.3-1
4958	BLN,STD	Pt 10		LC-B EP	2. Change COLA Part 10, Proposed License Conditions (Including ITAAC), Appendix B, Inspections, Tests, Analysis, and Acceptance Criteria, by changing the text under the heading Emergency Planning ITAAC. With the addition of a new table containing PS-ITAAC, this change is necessary to differentiate the table that includes the emergency planning ITAAC from this new Table 2.6.9-2. This differentiation is accomplished by changing the text, to read:	BLN-P02-VOL-SEC-FFD-20090323-OR

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
					The emergency planning ITAAC are included in the attached Table 3.8-1. Include these ITAAC after DCD Tier 1 Section 3.7.	
4817	BLN	Pt 10		LC-B EP / T3.8-1 1.1	12. COLA Part 10, Proposed License Conditions (Including ITAAC), Appendix B, Table 3.8-1, Inspections, Tests, Analyses (1.1) and Acceptance Criteria (1.1.1 and 1.1.2) will be revised.	SUPERSEDED by Qb 5347 - RAI LTR 146 response to RAI 13.03-033, item 12
5347	BLN	Pt 10		LC-B EP / T3.8-1 1.1	19. COLA Part 10, Proposed License Conditions (Including ITAAC), Appendix B, Table 3.8-1, Inspections, Tests, Analyses (1.1) and Acceptance Criteria (1.1.1 and 1.1.2) will be revised	BLN RAI LTR 146S response to RAI 13.03-040 item 19 [shown under 3.0]
4818	BLN	Pt 10		LC-B EP / T3.8-1 6.1	13. COLA Part 10, Proposed License Conditions (Including ITAAC), Appendix B, Table 3.8-1, Inspections, Tests, Analyses (6.1) and Acceptance Criteria (6.1) will be revised.	SUPERSEDED by Qb 5348 - RAI LTR 146 response to RAI 13.03-033, item 13
5348	BLN	Pt 10		LC-B EP / T3.8-1 6.1	21. COLA Part 10, Proposed License Conditions (Including ITAAC), Appendix B, Table 3.8-1, Inspections, Tests, Analyses (6.1) and Acceptance Criteria (6.1) will be revised	BLN RAI LTR 146S response to RAI 13.03-040 item 21 [shown under 8.1]
4894	BLN	Pt 10		LC-B EP / T3.8-1 6.4	COLA Part 10, Appendix B, Table 3.8-1, item 6.4, will be revised to read: · Wind speed (at 10 m and 55 m) · Wind direction (at 10 m and 55 m) · Ambient air temperature (at 10 m and 55 m)	RAI LTR 154 response to RAI 02.03.03-007 [shown under 8.4]
6032	BLN	Pt 10		x-AppB / T3.8-1	COLA Part 10, Appendix B, Table 3.8-1 will be replaced with Table 3.8-1 provided in Attachment OI 13.03-27 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-028 (eRAI 13.03-67) - This COLA change is a duplicate of Qb ID #6031
6033	BLN	Pt 10		x-AppB / T3.8-1	COLA Part 10, Appendix B, Table 3.8-1 will be replaced with Table 3.8-1 provided in Attachment OI 13.03-27 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-029 (eRAI 13.03-68) - This COLA change is a duplicate of Qb ID #6031
6035	BLN	Pt 10		x-AppB / T3.8-1	COLA Part 10, Appendix B, Table 3.8-1 will be replaced with Table 3.8-1 provided in Attachment OI 13.03-27 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-030 (eRAI 13.03-69) - This COLA change is a duplicate of Qb ID #6031
6036	BLN	Pt 10		x-AppB / T3.8-1	COLA Part 10, Appendix B, Table 3.8-1 will be replaced with Table 3.8-1 provided in Attachment OI 13.03-27 of this letter.	COL-SER-OI-13.03-032 (eRAI 13.03-71) - This COLA change is a duplicate of Qb ID #6031
6037	BLN	Pt 10		x-AppB / T3.8-1	COLA Part 10, Appendix B, Table 3.8-1 will be replaced with Table 3.8-1 provided in Attachment OI 13.03-27 of this letter	COL-SER-OI-Ch13 response to OI 13.03-033 (eRAI 13.03-72) - This COLA change is a duplicate of Qb ID #6031
6038	BLN	Pt 10		x-AppB / T3.8-1	COLA Part 10, Appendix B, Table 3.8-1 will be replaced with Table 3.8-1 provided in Attachment OI 13.03-27 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-34

Change ID#	COLA REP	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
						(eRAI 13.03-73) - This COLA change is a duplicate of Qb ID #6031
6039	BLN	Pt 10		x-AppB / T3.8-1	COLA Part 10, Appendix B, Table 3.8-1 will be replaced with Table 3.8-1 provided in Attachment OI 13.03-27 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-035 (eRAI 13.03-74) - This COLA change is a duplicate of Qb ID #6031
6040	BLN	Pt 10		x-AppB / T3.8-1	COLA Part 10, Appendix B, Table 3.8-1 will be replaced with Table 3.8-1 provided in Attachment OI 13.03-27 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-036 (eRAI 13.03-75) - This COLA change is a duplicate of Qb ID #6031
6041	BLN	Pt 10		x-AppB / T3.8-1	COLA Part 10, Appendix B, Table 3.8-1 will be replaced with Table 3.8-1 provided in Attachment OI 13.03-27 of this letter.	COL-SER-OI-Ch13 response to OI 13.03-037 (eRAI 13.03-76) - This COLA change is a duplicate of Qb ID #6031

SUMMARY

COLA Part A	Number of COLA Changes
Pt 02	344
Pt 04	27
Pt 05	73
Pt 07	7
Pt 09	10
Pt 10	51
TOTALS (6 groups)	**512**