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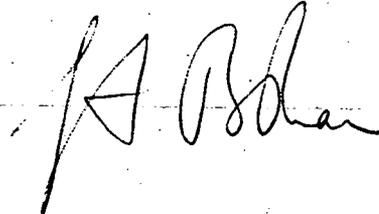
April 22, 1992

Re: Indian Point Unit No. 2
Docket No. 50-247
LER 92-06-00

Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

The attached Licensee Event Report LER 92-06-00 is hereby
submitted in accordance with the requirements of 10 CFR 50.73.

Very truly yours,



Attachment

cc: Mr. Thomas T. Martin
Regional Administrator - Region I
US Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Francis J. Williams, Jr., Project Manager
Project Directorate I-1
Division of Reactor Projects I/II
US Nuclear Regulatory Commission
Mail Stop 14B-2
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PO Box 38
Buchanan, NY 10511

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Indian Point Unit No. 2	DOCKET NUMBER (2) 0 5 0 0 0 2 4 7	PAGE (3) 1 OF 04
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TITLE (4)
Emergency Diesel Generator Fuel Oil Inventory Unavailable

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																			
03	23	92	92	006	00	04	22	92			05000																																			
<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:15%;">OPERATING MODE (9) N</td> <td colspan="11">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="5">POWER LEVEL (10) 100</td> <td>20.402(b)</td> <td>20.405(c)</td> <td>50.73(a)(2)(iv)</td> <td>73.71(b)</td> </tr> <tr> <td>20.405(a)(1)(i)</td> <td>50.38(c)(1)</td> <td>50.73(a)(2)(v)</td> <td>73.71(c)</td> </tr> <tr> <td>20.405(a)(1)(ii)</td> <td>50.38(c)(2)</td> <td>50.73(a)(2)(vii)</td> <td rowspan="3">OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td> </tr> <tr> <td>20.405(a)(1)(iii)</td> <td>X 50.73(a)(2)(i)</td> <td>50.73(a)(2)(viii)(A)</td> </tr> <tr> <td>20.405(a)(1)(iv)</td> <td>50.73(a)(2)(ii)</td> <td>50.73(a)(2)(viii)(B)</td> </tr> <tr> <td>20.405(a)(1)(v)</td> <td>50.73(a)(2)(iii)</td> <td>50.73(a)(2)(x)</td> <td></td> </tr> </table>												OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)											POWER LEVEL (10) 100	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)	20.405(a)(1)(i)	50.38(c)(1)	50.73(a)(2)(v)	73.71(c)	20.405(a)(1)(ii)	50.38(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)	20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	
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LICENSEE CONTACT FOR THIS LER (12)

NAME Michael Whitney, Senior Engineer	TELEPHONE NUMBER 9 1 4 5 2 6 1 5 1 3 1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)
		MONTH DAY YEAR 0 7 3 0 9 12

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 23, 1992 at 0200 hours, with the unit operating at 100% power, the plant entered Technical Specification 3.0.1 because the availability of the requisite emergency diesel generator (EDG) fuel oil inventory could not be assured. Although the full fuel oil inventory was actually present in the three tanks, the ability to transfer fuel oil from one of the tanks was interrupted due to blown fuses in the power supply to the EDG No. 23 auxiliaries, including the fuel transfer pump. Later it was concluded that the entry into Technical Specification 3.0.1 in this situation, although a conservative decision, was not necessary.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Indian Point Unit No. 2	DOCKET NUMBER (2) 0 5 0 0 0 2 4 7 9 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
			0 0 6	0 0	0 2	OF 0 4

TEXT (If more space is required, use additional NRC Form 386A's) (17)

PLANT AND SYSTEM IDENTIFICATION:

Westinghouse 4-Loop Pressurized Water Reactor

IDENTIFICATION OF OCCURRENCE:

Emergency Diesel Generator Fuel Oil Inventory Unavailable

EVENT DATE:

March 23, 1992

REPORT DUE DATE:

April 22, 1992

REFERENCES:

Significant Occurrence Report (SOR) 92-159, 92-165

PAST SIMILAR OCCURRENCE:

LER 92-003

DESCRIPTION OF OCCURRENCE:

On March 23, 1992 at 0125 hours, with the unit operating at 100% power, an operator conducting his rounds found that the pre-lubrication pump and jacket water heaters for Emergency Diesel Generator (EDG) No. 23 were not energized. Upon investigation the operator found that all three line fuses for the EDG No. 23 auxiliaries were blown. The EDG No. 23 support equipment that lost power because of these blown fuses included the starting air compressor, the fuel oil transfer pump, the pre-lubrication pump, the jacket water heaters and the lubricating oil heaters.

An assessment performed in connection with a previous occurrence of this type had determined that for the conditions encountered in this event the starting air compressor, the pre-lubrication pump, and the jacket water and lubricating oil heaters were not necessary for EDG operability, however the loss of the fuel oil transfer pump made the fuel oil inventory in the associated storage tank technically unavailable. This condition precluded satisfaction of the full Technical Specification 3.7.A.5 requirement that 19,000 gallons of fuel oil be available for the EDGs. Therefore, at 0200 hours, the unit entered Technical Specification 3.0.1 and the condition was reported to the NRC pursuant to 10 CFR 50.72, using the NRC emergency notification system.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Indian Point Unit No. 2	DOCKET NUMBER (2) 0 5 0 0 0 2 4 7 9 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
			0 0 6	0 0	0 3	OF 0 4

TEXT (If more space is required, use additional NRC Form 386A's) (17)

DESCRIPTION OF OCCURRENCE: (Continued)

By 0310 hours, the fuses were replaced, restoring power to the EDG No. 23 auxiliaries. These auxiliaries were restored shortly thereafter and the provisions of Technical Specification 3.0.1 were terminated.

ANALYSIS OF OCCURRENCE:

Technical Specification 3.0.1 applies when plant conditions exceed the conditions permitted by the Technical Specifications. Technical Specification 3.7.A.5 requires an onsite supply of 19,000 gallons of fuel oil to be available in the individual storage tanks for the three EDGs. Since the fuel oil transfer pump for 23 EDG was out of service due to loss of its power supply, it was concluded that its fuel supply volume, though present, was unavailable, therefore an entry into Technical Specification 3.0.1 occurred and is reportable under 10 CFR 50.73(a)(2)(i)(B).

Following a previous similar occurrence (LER 92-003), a review of the original licensing basis for the EDG fuel oil inventory was conducted. It was found that this basis recognized and considered the unavailability of one of the EDG storage tanks. An amendment to the license was planned to clarify the Technical Specifications regarding EDG fuel inventory requirements.

Following this most recent occurrence, an operability determination was developed since the Technical Specification is unclear regarding what declaration should be made if an EDG fuel oil transfer pump is out of service. The operability determination concluded that the declaration of an entry into Technical Specification 3.0.1 is not necessary for this condition. The actions determined to be appropriate were:

1. Declare the associated EDG inoperable and enter into a 7 day LCO for the affected diesel for the duration of the fuel transfer pump outage,
2. Assure that the available volume of fuel oil in the gas turbine tanks at the Buchanan Substation or onsite other than the normal supply tanks is at least 29,000 gallons, and
3. Assure that the total available volume of fuel oil in the remaining two underground supply tanks for the EDGs is at least 12,680 gallons.

This operability determination was reviewed by the Station Nuclear Safety Committee.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Indian Point Unit No. 2	DOCKET NUMBER (2) 0 5 0 0 0 2 4 7	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

CAUSE OF OCCURRENCE:

This condition resulted from the loss of power to EDG 23 support equipment resulting from all three, 40 Amp. in-line power supply fuses being blown. Since there was no obvious indication of the cause of the blown fuses a station effort was immediately initiated to determine the cause.

The three fuses were tested to determine whether the fuses opened on overload, mechanical failure or direct short. The test results indicated that the fuses blew due to a direct short condition with a current in excess of 10 times the fuse rating.

An intensive investigation of the cabling and wiring of the system was conducted including line current measurements, motor and wiring meggering, motor impedance testing, continuity checks, molded case circuit breaker trip testing, field wiring to print verification, and visual inspections of connections, wiring and conduit. No signs of damage were found and no abnormal findings were noted which would indicate a cause for the fuses blowing. Various scenarios have been evaluated and none found to be a credible cause.

CORRECTIVE ACTION:

Following the event, an investigation was launched as described above, to determine the cause of the blowing of the fuses.

The circuitry associated with the 23 EDG air compressor control power transformer which caused the previous failure described in LER 92-03 was carefully inspected. No causal relationship to this event was identified.

As discussed in the previous similar occurrence, an amendment to the license will be initiated to clarify the technical specifications regarding EDG fuel inventory requirements.

This event is under continuing investigation. Extensive personnel interviews are ongoing. In addition, an independent review is being undertaken by experts external to Con Edison. The results of these investigations are expected to be submitted in a supplement to this LER by July 30, 1992.