

Stephen B. Bram
Vice President

Consolidated Edison Company of New York, Inc.
Indian Point Station
Broadway & Bleakley Avenue
Buchanan, NY 10511
Telephone (914) 737-8116

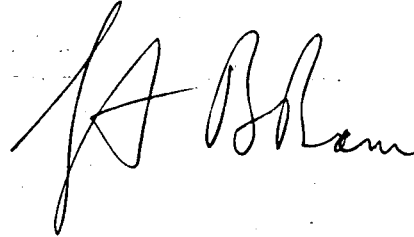
November 9, 1990

Re: Indian Point Unit No. 2
Docket No. 50-247
LER 90-12-00

Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

The attached Licensee Event Report LER 90-12-00 is hereby submitted in accordance with the requirements of 10 CFR 50.73.

Very truly yours,



Attachment

cc: Mr. Thomas T. Martin
Regional Administrator - Region I
US Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Francis J. Williams Jr., Project Manager
Project Directorate I-1
Division of Reactor Projects I/II
US Nuclear Regulatory Commission
Mail Stop 14B-2
Washington, DC 20555

Senior Resident Inspector
US Nuclear Regulatory Commission
PO Box 38
Buchanan, NY 10511

9011260215 901119
PDR ADDOCK 05000247
S PDC

IF22
11

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Indian Point Unit No. 2

DOCKET NUMBER (2)

0 5 0 0 0 2 4 7

PAGE (3)

1 OF 0 3

TITLE (4)

Planned Monitored Plant Vent Release Causes Operation of E.S.F.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)					
1	0	2	0	9	0	9	0	0	0	1	2	0	5	0	0	0
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)									73.71(b)				
N			20.402(b)			20.405(c)			<input checked="" type="checkbox"/> 50.73(a)(2)(iv)			73.71(c)				
POWER LEVEL (10)			20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
9 6			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)							
			20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)							
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)							
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Richard T. Louie, Engineer	9 1 1 4 5 2 6 4 5 1 6 7 1 8

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/>	<input checked="" type="checkbox"/>				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On October 20, 1990, at about 2120 hours, with reactor power at 96%, a planned monitored gaseous release occurred through the plant vent for a duration of approximately one minute. The cause of the release was as a result of maintenance activities associated with the venting of the suction stabilizer for Charging Pump 21. As designed, an automatic pressure relief that was in progress, and actuated the Weld Channel and Penetration Pressurization System (WCPPS) to restore the pressure between the containment isolation valves in the pressure relief line. The WCPPS is classified as an Engineered Safety Feature (ESF). The logic requirements of the ESF actuation system were neither required nor fulfilled. The release corresponded to a maximum value of only 7.5% of the maximum instantaneous concentration for an unrestricted area. The total noble gas radioactivity released was 4.52E-1 curies. No Technical Specification or NRC limits were exceeded.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Indian Point Unit No. 2	DOCKET NUMBER (2) 0 5 0 0 0 2 4 7 9 0 — 0 1 2 — 0 0 0 2 OF 0 3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

PLANT AND SYSTEM IDENTIFICATION:

Westinghouse 4-Loop Pressurized Water Reactor

IDENTIFICATION OF OCCURRENCE:

Planned monitored gaseous release through plant vent initiated operation of an Engineered Safety Feature (ESF).

EVENT DATE:

October 20, 1990

REPORT DUE DATE:

November 19, 1990

REFERENCES:

Significant Occurrence Report (SOR) 90-523

PAST SIMILAR OCCURRENCE:

LER 89-03

LER 87-10

DESCRIPTION OF OCCURRENCE:

On October 20, 1990, at about 2120 hours, with reactor power at 96%, preparations were under way to restore service to Charging Pump No. 21 following completion of maintenance activities. The suction stabilizer to the charging pump was being vented to the vent duct. This caused the Plant Vent Gaseous Radiation Monitor R-14 to spike resulting in automatic termination of a containment pressure relief which was in progress at the time. However, the source of the increased activity was not from within containment, but was from the air vented from the charging pump stabilizer. The spike was about 20,000 counts over the normal R-14 reading for a duration of approximately one minute. A calculation was subsequently performed to determine that this release resulted in a maximum value of only 7.5% of the maximum instantaneous concentration for an unrestricted area. There were no effects upon the health and safety of the public.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Indian Point Unit No. 2	DOCKET NUMBER (2) 0 5 0 0 0 2 4 7 9 0 — 0 1 2 — 0 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
					0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

ANALYSIS OF OCCURRENCE:

This event is reportable since the WCPPS, an engineered safety feature, was actuated. Although the WCPPS did function, it was not required to mitigate any radiological conditions existing in containment at the time. All plant systems functioned as designed. There was no equipment failure. The release of radioactivity through the plant vent was sufficient to exceed the setpoint of the plant vent gaseous radiation monitor. The setpoint of the monitor is set sufficiently low such that the normal containment vent release path is isolated whenever the activity level in containment is at a fraction of the level permitted by Technical Specifications for normal releases. The total noble gas radioactivity released through the plant vent was 4.52E-1 curies.

CAUSE OF OCCURRENCE:

The increased gaseous activity was determined to result from the venting of the Charging Pump 21 suction stabilizer following completion of maintenance activities coincident with performance of a containment pressure relief procedure.

CORRECTIVE ACTION:

The operators have been instructed to notify the control room to ensure that no pressure relief is in progress prior to venting a charging pump line.

Operator training will emphasize the need for increased awareness regarding conducting any equipment evolutions which could affect plant operational and or radiological conditions.