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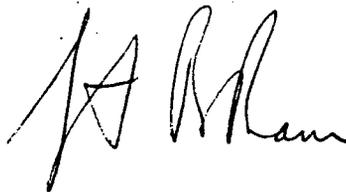
August 30, 1990

Re: Indian Point Unit No. 2
Docket No. 50-247
LER 90-05-00

Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

The attached Licensee Event Report LER 90-05-00 is hereby submitted in accordance with the requirements of 10 CFR 50.73.

Very truly yours,



Attachment

cc: Mr. Thomas T. Martin
Regional Administrator - Region I
US Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects I/II
US Nuclear Regulatory Commission
Mail Stop 14B-2
Washington, DC 20555

Senior Resident Inspector
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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Indian Point Unit No. 2	DOCKET NUMBER (2) 0 5 0 0 0 2 4 7	PAGE (3) 1 OF 03
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TITLE (4)
Failure to Monitor Service Water Inlet Temperature

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
08	01	90	90	005	00	08	30	90		0 5 0 0 0

OPERATING MODE (9) N

POWER LEVEL (10) 0 9 6

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(e)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(e)(2)(v)	<input type="checkbox"/> 73.71(c)
<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(e)(2)(vii)	<input type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)
<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(e)(2)(i)	<input type="checkbox"/> 50.73(e)(2)(viii)(A)	
<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(e)(2)(ii)	<input type="checkbox"/> 50.73(e)(2)(viii)(B)	
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(e)(2)(iii)	<input type="checkbox"/> 50.73(e)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Joan F. Etzweiler, Senior Engineer	TELEPHONE NUMBER 9 1 1 4 5 1 2 6 1 - 5 3 1 6 5
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS
B	B I T I		P 4 2 7	No					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On August 1, 1990, with the reactor at 96% of rated power, it was determined that the service water inlet temperature was at or above 80°F and not being monitored in accordance with Technical Specification 3.3.F.5. The temperature recorder in the Central Control Room was reading approximately 8°F less than manual measurements of Unit 2 inlet temperature. Based on Unit 3 temperature records, the 24 hour average temperature had reached 80°F on July 21, 1990. The required monitoring was therefore not performed over an eleven day period. Upon discovery, required four hour alternative measurements were initiated. At no time did the temperature exceed the 95°F maximum allowed by the Technical Specifications. Thus, there was no reduction in overall plant safety.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Indian Point Unit No. 2	DOCKET NUMBER (2) 0 5 0 0 0 2 4 7	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 0	— 0 0 5	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

PLANT AND SYSTEM IDENTIFICATION:

Westinghouse 4-loop pressurized water reactor.

IDENTIFICATION OF OCCURRENCE:

Failure to monitor service water inlet temperature when the intake water temperature, averaged over a 24 hour period, reached 80°F, and when the reactor was above 350°F, as required by Technical Specification 3.3.F.5.

EVENT DATE:

August 1, 1990

REPORT DUE DATE:

August 31, 1990

REFERENCES:

Significant Occurrence Report (SOR) 90-364

PAST SIMILAR OCCURRENCE:

None

DESCRIPTION OF OCCURRENCE:

On August 1, 1990, with the reactor at 96% of rated power, it was determined that the service water inlet temperature monitoring instrumentation was indicating erroneously low when the actual temperature was at or above 80°F. This condition was discovered after it was noticed that the temperature indication in the Central Control Room differed from the temperature measurements made using portable instruments. The latter measurements were being made for the purpose of monitoring unit performance and efficiency. Upon investigation, on August 1, 1990, it was found that the temperature recorder in the Central Control Room was reading approximately 8°F less than the manual measurements of Unit 2 inlet temperature. The temperature monitor was thereupon declared inoperable. Review of Unit 3 river water temperature data indicated that a 24 hour average value of 80°F had been reached on July 21, 1990, and the Unit 2 temperature monitor had

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF OCCURRENCE: (continued)

been reading low since before that date. Technical Specification 3.3.F.5 requires that the service water inlet temperature monitor be operable when the intake structure water temperature, averaged over a 24 hour period, reaches 80°F and when the reactor is above 350°F. When this monitor is inoperable, alternative measurements must be taken. This must be done every four hours between 80°F and 90°F and every hour above 90°F. The period when the required monitoring was not performed extended from July 21, 1990 to August 1, 1990. During this period, based on Unit 3 measurements, the 95°F Technical Specification maximum was not exceeded.

CAUSE OF OCCURRENCE:

The cause of the erroneous temperature readings appears to have been moisture intrusion into the probe which is immersed in the river water. Placing an additional tubular seal over the junction between the stainless steel probe tip and the silicon rubber sheathing for the wire leads resulted in acceptable performance.

CORRECTIVE ACTION:

The immediate corrective action was to initiate alternative measurements of temperature every four hours in accordance with the Technical Specifications. The malfunctioning probe was replaced. A replacement probe with stainless steel sheathing for the lead wires has been ordered. It is expected that this type of probe will prevent moisture intrusion, thereby eliminating the problem.