

Consolidated Edison Company of New York, Inc.  
Indian Point Station  
Broadway & Bleakley Avenue  
Buchanan, NY 10511  
Telephone (914) 737-8116

August 30, 1990

Re: Indian Point Unit No. 2  
Docket No. 50-247

Document Control Desk  
US Nuclear Regulatory Commission  
Mail Station P1-137  
Washington, DC 20555

SUBJECT: Response to Generic Letter 90-03

This is in response to Generic Letter 90-03, "Relaxation of Staff Position in Generic Letter 83-28, Item 2.2 Part 2 'Vendor Interface for Safety-Related Components'". In GL 90-03, the staff concludes that an adequate vendor interface program should include:

- (a) A program with the NSSS vendor as described in the VETIP, which covers all the safety-related components within the NSSS scope of supply. This program should include provisions for assuring receipt by the licensee/applicant of all technical information provided by the NSSS vendor; and
- (b) A program of periodic contact with the vendors of other key safety-related components not included in (a) above.

Relative to (a) above, Con Edison maintains a formal signature acknowledgment program with Westinghouse which covers all the safety-related components within the NSSS scope of supply. In addition, the Con Edison Chief Plant Engineer, responsible for updating vendor manuals, is provided all technical correspondence from Westinghouse to Con Edison.

Relative to (b) above, the vendor interface program is not a formal one and therefore all contact is not necessarily documented as such. Vendor contact is on an as-needed basis (e.g. prior to component repair or maintenance). Several safety-related components are repaired or have preventive maintenance performed by the original vendor or use that vendor or his representative in a consulting or oversight capacity. Plant components found to be obsolete or unavailable are replaced as required, and the vendor is approached when possible on a case-by-case basis.

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Updates to vendor information are incorporated by several means such as vendor letter, INPO Nuclear Network, practical experience and engineering review. The modification process incorporates controls to add new vendor information or to update existing manuals. Plant and Engineering procedures (e.g., SAO 405, 409, TS.SQ.12-302, OP-290-1) have provisions for those controls and all working groups (e.g., Maintenance, I&C, Chemistry) are instructed to use only such controlled information.

Should your or your staff have any questions regarding this matter, please contact Mr. Charles W. Jackson, Manager, Nuclear Safety and Licensing.

Very truly yours,



Subscribed and sworn to  
before me this 30<sup>th</sup> day  
of August, 1990.

Karen L. Lancaster  
Notary Public

**KAREN L. LANCASTER**  
Notary Public, State of New York  
No. 60-4643659  
Qualified in Westchester County  
Term Expires 9/30/91

cc: Mr. Thomas T. Martin  
Regional Administrator - Region I  
US Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406

Mr. Donald S. Brinkman, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects I/II  
US Nuclear Regulatory Commission  
Mail Stop 14B-2  
Washington, DC 20555

Senior Resident Inspector  
US Nuclear Regulatory Commission  
PO Box 38  
Buchanan, NY 10511

August 30, 1990

Docket No. 50-247

Mr. Stephen B. Bram  
Vice President, Nuclear Power  
Consolidated Edison Company of  
New York, Inc.  
Broadway and Bleakley Avenue  
Buchanan, New York 10511

Dear Mr. Bram:

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 2 - WITHDRAWAL OF INSERVICE  
INSPECTION PROGRAM RELIEF REQUEST NO. 26 (TAC NO. 66656)

By letter dated November 9, 1987, you submitted Inservice Inspection Program  
Relief Request No. 26 to the second inspection interval (July 1, 1984-June 30,  
1994) for Indian Point 2. The proposed relief request sought relief from the  
requirements of ASME Code Section XI as it pertains to emergency  
repairs/replacements and temporary repairs/replacements and alternative  
methods for emergency repairs/replacements and temporary repairs/replacements.

On June 15, 1990, the NRC issued Generic Letter 90-05 which provided guidance  
for performing temporary non-code repairs of ASME Code Class 1, 2, and 3  
piping. Subsequently, by letter dated August 22, 1990, you withdrew the  
subject relief request based on the guidance provided in Generic Letter 90-05.  
Therefore, we consider this issue closed.

Sincerely,

ORIGINAL SIGNED BY:

Donald S. Brinkman, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

cc: See next page

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Mr. Stephen B. Bram  
Consolidated Edison Company  
of New York, Inc.

Indian Point Nuclear Generating  
Station 1/2

cc:

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