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September 25, 1989

Re: Indian Point Station
Docket No. 50-247

Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

SUBJECT: NRC Bulletin No. 89-01: Failure of Westinghouse Steam
Generator Tube Mechanical Plugs

The purpose of this letter is to respond to NRC inquires and to provide an update to our earlier response to NRC Bulletin No. 89-01.

Our earlier response, dated June 15, 1989, cited the next refueling outage, in 1991, as when remedial action for suspect plugs would be implemented. Our plans in this regard remain unchanged. The scope of this effort would include all plugs whose estimated remaining service life, in terms of effective full power days (EFPD), does not justify operation to the next refueling outage after 1991. More specifically, all of the hot leg plugs from heats 3279, 3513, 3962 and 4523 would be included in the remedial program. The cold leg plugs have been determined to have service lives which permit this postponing action until later outages. We believe this to be the preferable course of action since it will permit incorporation of experience gained at other plants during this intervening period.

The service life of the mechanical plugs have been calculated reflecting an expected increase in Reactor Coolant System average temperature to 559°F by December 1, 1989. Westinghouse has been requested to revise in their third quarterly update of the topical report on tube plugs, the steam generator parameters as they relate to the increase in reactor coolant temperature. The pertinent tables now reflect nominal steam generator temperatures.

Since the date of our prior submittal on June 15, 1989, we have decided to perform a midcycle steam generator inspection. The scheduled duration of this outage is less than four weeks. However, regardless of the length of the midcycle outage, we interpret Bulletin 89-01 as permitting remedial action during the 1991 refueling outage due to the one-time deferral per the Bulletin for plants with partially rolled tube sheets.

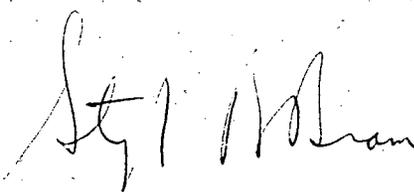
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As a point of clarification, our earlier response mentioned replacement of suspect plugs. It is our current intent to pursue the Westinghouse plug-in-a plug concept. This would mean the suspect plugs would be left in place but would be isolated from the reactor coolant by means of an inserted plug.

Should you or your staff have any questions regarding this matter, please contact Mr. Jude G. Del Percio, Manager, Regulatory Affairs and Safety Assessment.

Very truly yours,



cc: Mr. William Russell
Regional Administrator - Region I
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