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Vice President

Consolidated Edison Company of New York, Inc.  
Indian Point Station  
Broadway & Bleakley Avenue  
Buchanan, NY 10511  
Telephone (914) 737-8116

August 18, 1989

Re: Indian Point Unit No. 2  
Docket No. 50-247  
LER 89-08-00

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Mail Station P1-137  
Washington, DC 20555

The attached Licensee Event Report LER 89-08-00 is hereby submitted in accordance with the requirements of 10 CFR 50.73.

Very truly yours,



Attachment

cc: Mr. William Russell  
Regional Administrator - Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406

Mr. Donald S. Brinkman, Project Manager  
Project Directorate I-1  
Division of Reactor Projects I/II  
U.S. Nuclear Regulatory Commission  
Mail Stop 14B-2  
Washington, DC 20555

Senior Resident Inspector  
U.S. Nuclear Regulatory Commission  
P.O. Box 38  
Buchanan, NY 10511

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LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3160-0104  
EXPIRES - 8/31/85

FACILITY NAME (1)  
**Indian Point Unit No. 2**

DOCKET NUMBER (2)  
**0 5 0 0 0 2 4 7 1**

PAGE (3)  
**0 3**

TITLE (4)  
**Technical Specification Limit Exceeded: IVSWS**

EVENT DATE (6)  
MONTH: **07** DAY: **19** YEAR: **89**

LER NUMBER (6)  
YEAR: **89** SEQUENTIAL NUMBER: **008** REVISION NUMBER: **00**

REPORT DATE (7)  
MONTH: | DAY: | YEAR: |

OTHER FACILITIES INVOLVED (8)  
FACILITY NAMES: | DOCKET NUMBER(S): **0 5 0 0 0**

OPERATING MODE (9)  
**N**

POWER LEVEL (10)  
**11010**

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(e)	<input type="checkbox"/> 60.736(i)(2)(ii)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 60.36(a)(1)	<input type="checkbox"/> 60.736(i)(2)(v)	<input type="checkbox"/> 73.71(c)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 60.36(a)(2)	<input checked="" type="checkbox"/> 60.736(i)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 60.736(i)(2)(ii)	<input type="checkbox"/> 60.736(i)(2)(vii)(A)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 60.736(i)(2)(ii)	<input type="checkbox"/> 60.736(i)(2)(vii)(B)	
<input type="checkbox"/> 20.406(a)(1)(vi)	<input type="checkbox"/> 60.736(i)(2)(iii)	<input type="checkbox"/> 60.736(i)(2)(viii)	

LICENSEE CONTACT FOR THIS LER (12)  
NAME: **Harold Reid, Senior Engineer**

TELEPHONE NUMBER  
AREA CODE: **9114** NUMBER: **512161-1541919**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
	C	B	V	A13915	N				
	B	B	O					A13915	N
	C	B	V	A13915	N				

SUPPLEMENTAL REPORT EXPECTED (14)  
 YES (if yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)  
MONTH: | DAY: | YEAR: |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 19, 1989, during a review of completed tests results, it was determined that a Technical Specification limit had been exceeded. During the 1989 refueling outage, surveillance tests of type "B" and "C" (Appendix J) valves were conducted from May through June of 1989. When test results were totalled, it was discovered that the as found total of individual valve leakage exceeded the 14,700 cubic centimeters per hour permitted by the Technical Specification for the Isolation Valve Seal Water System. Excessive leakage occurred across ten valves. The valves were promptly repaired and leakage brought to well within specification prior to heating above cold shutdown.

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A/1

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

NUCLEAR REGULATORY COMMISSION  
 APPROVED OMB NO 316C-0104  
 EXPIRES 6/30/85

PLANT NAME (1)  Indian Point Unit No. 2	DOCKET NUMBER (2)							LER NUMBER (6)			PAGE (3)		
	0 5 0 0 0 2 4 7 8 9 - 0 0 8 - 0 0							YEAR	SEQUENT. NUMBER	REVISION NUMBER	1		
											0 2	OF	0 3

AT IF more space is required, use additional NRC Form 364A (11/79)

Plant and System Identification:

Westinghouse 4-loop pressurized water reactor.

Identification of Occurrence:

Total leakage of Isolation Valve Seal Water System (IVSWS) exceeds Technical Specification Limit.

Event Date: July 19, 1989

Past Similar Occurrence: LER 84-006, LER 88-003

Significant Occurrence Report: SOR 89-421

Description of Occurrence:

During the 1989 refueling outage, surveillance testing of Containment isolation valves was performed throughout the period May through June of 1989. Valves found to be leaking in excess of their individual criterion were repaired during the outage, prior to heating the plant above cold shutdown. Post maintenance leakage rates were substantially below the maximum permitted. On July 19, 1989, it was determined that the as found aggregate leakage from the Isolation Valve Seal Water System (IVSWS) exceeded the maximum permitted by the Technical Specification. The Technical Specification limit is 14,700 cubic centimeters per hour. The valves contributing to the leakage were subjected to corrective maintenance and post maintenance leakage rates were substantially below the maximum permitted.

Analysis of Occurrence:

This event is reportable since a Technical Specification Limit was exceeded. Since the IVSWS is not relied upon in the limiting FSAR accident analysis, at no time was there any impact on the limiting event.

Cause of the Occurrence:

The leakage occurred across ten Containment isolation valves. These were:

1. Valves 791 and 798 - Component Cooling Water isolation valves.
2. Valve 518 - Containment isolation valve.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104  
EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (3)						PAGE (3)		
		YEAR	SEQUENT. NUMBER		REVISION NUMBER		I			
		0   5   0   0   0   2   4   7   8   9   -   0   0   8   -   0   0	0   3	OF	0   3					

Indian Point Unit No. 2

USE IF MORE SPACE IS REQUIRED. USE ADDITIONAL NRC Form 286A (11/77)

Cause of the Occurrence: (continued)

3. Valve 741-A - Containment penetration isolation valve.
4. Valves 1190, 1191 1229, 1230, E1 containment isolation penetration valves.
5. Valve 851A - No. 22 SIS Pump Discharge isolation valve.

The excessive leakage was due to seepage between the body and bonnet of valves and/or worn diaphragms.

Corrective Action:

The valves were repaired, tested and returned to service. Con Edison will perform a review of the results of B & C testing for adverse trends. Further action will be taken as appropriate.