NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)
Indian Point Unit No. 2

DOCKET NUMBER (2) 0 5 0 0 0 2 4 7

PAGE (3) 1 OF 0 3

TITLE (4)
Plant Vent Gaseous Activity Monitor ESF Acutation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 3	2 0	8 9	8 9	— 0 0 5	— 0 0	0 4	1 0	8 9		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9): N

POWER LEVEL (10): 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

	20.402(b)		20.406(c)
	20.406(a)(1)(i)		50.36(c)(1)
	20.406(a)(1)(ii)		50.36(c)(2)
	20.406(a)(1)(iii)		50.73(a)(2)(i)
	20.406(a)(1)(iv)		50.73(a)(2)(ii)
	20.406(a)(1)(v)		50.73(a)(2)(iii)
X	50.73(a)(2)(iv)		73.71(b)
	50.73(a)(2)(v)		73.71(c)
	50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	50.73(a)(2)(viii)(A)		
	50.73(a)(2)(viii)(B)		
	50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME: Jude G. Del Percio, Manager, Regulatory Affairs

TELEPHONE NUMBER: AREA CODE 9 1 4 5 2 6 - 5 1 2 7

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
B	I L	R I	T 2 6 0	No					
B	I L	R A	T 2 6 0	No					

SUPPLEMENTAL REPORT EXPECTED (14)

[ ] YES (If yes, complete EXPECTED SUBMISSION DATE) [X] NO

EXPECTED SUBMISSION DATE (15): MONTH | DAY | YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

LER 89-05

On March 20, 1989 at approximately 1605 hours the Plant Vent Gaseous Activity Monitor (R-14) experienced a spurious electronic spike and initiated Engineered Safety Feature (ESF) actuation of the Weld Channel and Containment Penetration Pressurization System (WCCPPS) and the isolation of the Containment Ventilation System. However, there was no Containment Isolation Signal generated, nor was one required. At the time the 1989 Refueling Outage was in progress with the reactor at 10 counts per second; the reactor coolant system at 120°F and 0 psig; and the pressurizer at a 20% level. There were no impacts on public health and safety.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Indian Point Unit No. 2	0 5 0 0 0 2 4 7	8 9	— 0 0 5	— 0 0	0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

PLANT AND SYSTEM IDENTIFICATION:

Westinghouse Four Loop Pressurized Water Reactor
Radiation Monitoring System
Containment Leakage Control System

IDENTIFICATION OF OCCURRENCE:

ESF actuation due to a spurious electronic spike in the Plant Vent Gaseous Activity Monitor (R-14).

REPORTABILITY DETERMINATION DATE:

March 20, 1989

REPORT DUE DATE:

April 19, 1989

REFERENCES:

SOR 89-175, dated March 20, 1989

PAST SIMILAR OCCURRENCE:

LER 87-12, dated November 18, 1987 and LER 89-03, dated April 4, 1989

DESCRIPTION OF OCCURRENCE:

On March 20, 1989 at approximately 1605 hours the Plant Vent Gaseous Activity Monitor (R-14) experienced a spurious electronic spike and initiated Engineered Safety Feature (ESF) actuation of the Weld Channel and Containment Penetration Pressurization System (WCCPPS) and the isolation of the Containment Ventilation System. However, there was no Containment Isolation Signal generated, nor was one required. At the time the 1989 Refueling Outage was in progress with the reactor at 10 counts per second; the reactor coolant system at 120°F and 0 psig; and the pressurizer at a 20% level.

This LER is required since the event constitutes an unplanned ESF actuation. All safety systems functioned as required. Other than the analysis provided below, which was initiated to determine the cause of the ESF actuation, no further analysis of the occurrence is necessary. There were no effects upon the health and safety of the public.

CAUSE OF OCCURRENCE:

A spurious electronic spike in the Plant Vent Gaseous Activity Monitor (R-14) initiated ESF actuation of the WCCPPS and isolation of the Containment Ventilation System.

ANALYSIS OF OCCURRENCE:

After R-14 tripped, a review of other radiation monitoring instrumentation was conducted to verify that the instrument behavior was not due to a legitimate increase in gaseous activity. It was determined that there had been no increase in radiation released and that the actuation was due to a spurious spike in the Plant Vent Gaseous Activity Monitor (R-14) electronics. The monitor was reset and functioned normally. This monitor (R-14) was manufactured by Tracerlab Inc.

At the time of the occurrence the 1989 Refueling Outage was in progress. The reactor vessel was still sealed and the reactor coolant system was being cooled by the Residual Heat Removal System.

CORRECTIVE ACTION:

As a result of a similar occurrence during the 1987 Refueling Outage (LER 87-12), a modification had been developed to replace certain radiation monitors, including R-14. Those radiation monitors that can only be replaced during a refueling outage are being replaced during this outage. The remaining monitors, including R-14, are to be replaced after this refueling outage.

Stephen B. Bram
Vice President

Consolidated Edison Company of New York, Inc.
Indian Point Station
Broadway & Bleakley Avenue
Buchanan, NY 10511
Telephone (914) 737-8116

April 10, 1989

Re: Indian Point Unit No. 2
Docket No. 50-247
LER 89-05-00

Document Control Desk
U.S. Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

The attached Licensee Event Report LER 89-05-00 is hereby submitted in accordance with the requirements of 10CFR50.73.

Very truly yours,

Attachment

cc: Mr. William Russell
Regional Administrator - Region I
U.S. Nuclear Regulatory Commission
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King of Prussia, PA 19406

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