NRC Fee	m 364		 -								11.5 111						
					LIC	ENSE	E EVE	NT RE	PORT	(LER)	U.S. AC	APPRO EXPIRI	EQULATOR	NO. 3166			
FACILIT	Y NAME (INDI	AN POINT U	NIT NO.	2	····				OCKET NUMBER	(2)		PAC	3E (3)		
TITLE 14	1)										0 5 0 0	10 2	417	1 OF	013		
	·		Fail	ure of Safe	ety Rel	ated	Moto	r Cont	trol	Center							
EVENT DATE (5) LER NUMBER (6)						REPORT DATE (7) OTHER				OTHER	R FACILITIES INVOLVED (8)						
MONTH DAY YEAR			YEAR BEQUENTIAL REVENON			MONTH DAY YEAR			FACILITY NAMES			DOCKET NUMBER(S)					
0 9	117	8 8	8 18	- _{0 1 k}	-	110	1 lo	010		·		0 5	0101	0			
9 1 3	1 + 1 /	1 			0 0	1 0	1 7	8 8				0 5	0 10 1	0 1	1 1		
	PATING			EPORT IS SUBMITTED	PURSUANT T			ENTS OF 10	CFR S: (Check one or more o	f the following) (11)					
POWER LEVEL 1 0 0				20.402(b) 20.405(a)(1)(i) 20.405(a)(1)(ii)			20.406(e) 80.36(e)(1) 80.36(e)(2)			60.73(a)(2)(iv)			73,71(b)				
			—							50.73(a)(2)(v)	73.71(e)						
				┥								THER (Specify in Abstract elow end in Text, NRC Form					
			—	0.406(a)(1)(lv)	<u> </u>	60,73(e)				50.73(a)(2)(vHI)(A	•	366	w				
			—	0.406(a)(1)(v)	J i	50.73(c)			-	B0.73(a)(2)(vill)(B B0.73(a)(2)(x)	'						
			 _					FOR THIS	LER (12)	BU./3(8)(2)(2)							
NAME												TELEPHO	NE NUMBE	R			
	Joh	n Ell	wand	ger, Princi	inal End	rinoo	~				AREA CODE						
					rbar mi	JIIICC	T.				9 1 1 4	5 2 6 - 1 5 1 8 2					
				COMPLETE (NE LINE FOR	EACH CO	MPONENT	FAILURE	DESCRIBI	ED IN THIS REPOR	F (13)	<u></u>			ــــــــــــــــــــــــــــــــــــــ		
CAUSE SYSTEM		COMPONENT MANUFAC RETURNER		REPORTABLE TO NPRDS	LE 8		CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTO NI						
	1																
	_1		<u></u>							1 1 1	1 1 1						
				BUPPLEME	NTAL REPORT	EXPECTE	D (14)				6786		MONTH	DAY	YEAR		
YE	ill you, c	ompiete E.	XPECTE	O SUBMISSION DATE	, —	v] No				EXPECTE SUBMISSI DATE (1)	ON					

On September 17, 1988, while the plant was at full power, replacement of a fuse and other corrective maintenance on a motor control center required de-energization of the motor control center. This action disabled one of the four battery chargers. The Indian Point Unit No. 2 Technical Specifications require that all four battery chargers be operable prior to achieving criticality. When at full power operation, there are no applicable action statements addressing a battery charger becoming unavailable. It was thus concluded that the plant had entered Technical Specification provision 3.0.1 which applies when conditions are in excess of the Technical Specifications limiting condition for operation.

The battery charger was inoperable for 33 minutes. The four batteries were at full capacity and capable of coping with emergency loads during this period. Thus, there was no impact upon plant safety.

8810250433 881017 PDR ADDCK 05000247 PNU

TE27

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR RECULATORY COMMISSION APPROVED OMB NO. 3150-0104

FACILITY NAME (1)						EXPIRES: 8/31/65							
	DOCKET NUMBER 121	LER NUMBER (6)					PAGE (3)						
INDIAN POINT UNIT NO. 2		YEAR	100g 1	SEQUENTIAL		HEVISION							
TEAT III more wose is required, was additioned NHC form Their 1231	0 5 0 0 0 2 4 7	818		0 1 2	_	0 0	0,2	OF	0 13				

Plant and System Identification:

Westinghouse 4-loop pressurized water reactor.

Identification of Occurrence:

Plant entered Technical Specification provision 3.0.1, which is applicable to plant conditions in excess of those prescribed by the Technical Specifications.

Event Date: September 17, 1988

Past Similar Occurrences:

None.

Significant Occurrence Report: SOR 88-480

Description of Occurrence:

On September 17, 1988, while the plant was at full power, a loose lug in a breaker located in motor control center 29 caused overheating of a cable and opening of a fuse. In order to accomplish repairs, the motor control center was de-energized. This motor control center feeds one battery charger, No. 21, which was disabled as a result of the motor control center de-energization.

This action also disabled No. 21 Emergency Diesel Generator auxiliary equipment including the lube oil pump, jacket cooling water pump and the lube oil heaters since their power feeds were also de-energized. This rendered the diesel generator inoperable. The remaining two diesel generators were operable based on previous surveillance test results and a 7 day LCO was declared for the diesel generator.

Technical Specification 3.0.1 was exited and the 7 day LCO terminated when the MCC was restored to service in 33 minutes.

Analysis of Occurrence:

This event is reportable since it was concluded that the plant had entered Technical Specification provision 3.0.1 covering plant conditions being in excess of that permitted by the Technical Specifications. The Indian Point Unit No. 2 Technical Specifications require that all four battery chargers be operable prior to achieving criticality. There is no action statement pertaining to the availability of battery chargers during power operation. The action statement pertains only to the availability of batteries but implies that the battery chargers be available at all times.

NAC FORM 3464

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION APPROVED ONE NO. 3150-0104

FACILITY HAME (1)	·					
	DOCKET NUMBER (2)	LEH NUMBER ISI	PAGE (3)			
INDIAN POINT UNIT NO. 2	·	YEAR SEQUENTIAL MEVISION				
TEXT (II must been in required one additional NMC for 2001 122)	0 5 0 0 0 2 4 7	8,8 _ 0,1,2 _ 0,0 0	0 13 06 0 13.			

Cause of Occurrence:

The cause of the occurrence was a loose lug necessitating corrective action while at power operation.

Corrective Action:

The lug was tightened and the fuse was replaced. The motor control center was returned to service. A Technical Specification amendment will be prepared and submitted to allow a battery charger to be inoperable during power operation when all four safety related station batteries are operable. This proposed amendment will be similar to the degraded mode of operation allowed in the Standard Technical Specification for a battery charger.

NAC FORM 3444

Consolidated Edison Company of New York, Inc. Indian Point Station
Broadway & Bleakley Avenue
Buchanan, NY 10511
Telephone (914) 737-8116

October 17, 1988

Re:

Indian Point Unit No. 2 Docket No. 50-247 LER 88-12

Document Control Desk U.S. Nuclear Regulatory Commission Mail Station P1-137 Washington, DC 20555

The attached Licensee Event Report LER 88-12 is hereby submitted in accordance with the requirements of 10 CFR 50.73.

Very truly yours,

Attachment

cc: Mr. William Russell
Regional Administrator - Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Ms. Marylee M. Slosson, Project Manager Project Directorate I-l Division of Reactor Projects I/II U.S. Nuclear Regulatory Commission Mail Stop 14B-2 Washington, DC 20555

Senior Resident Inspector U.S. Nuclear Regulatory Commission P.O. Box 38 Buchanan, NY 10511

1/22/1