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June 8, 1990

Re: Indian Point Unit No. 2
Docket No. 50-247

Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

SUBJECT: Indian Point Unit 2, 1990 Steam Generator Inspection Program

This letter supplements the subject inspection results presented to the NRC on May 21, 1990 as reported in WCAP-12573, (Proprietary to Westinghouse) and WCAP-12574, (Non-Proprietary), entitled "Indian Point Unit 2 Steam Generator Inspection, Repair and Restoration Program".

UT Examination:

As indicated at the May 21, 1990 meeting with the NRC, certain UT examinations were in progress or test data was awaiting resolution. UT investigations have now been completed and Section 2.2 entitled "Near Girth Weld Area", of the aforementioned WCAPs are supplemented with data now available as follows:

1. Steam Generator 21

UT investigation was extended to include 360° around the 10" band. Subsequent MT confirmed the presence of 7 additional indications. The maximum depth upon grinding was 0.71 inches.

2. Steam Generator 22

UT investigation was extended to include 360° around the 10" band. Subsequent MT confirmed the presence of 17 additional indications. The maximum depth determined upon grinding was 0.91 inches.

3. Steam Generator 23

UT investigation was extended to include 360° around the 10" band. Subsequent MT confirmed the presence of 15 additional indications. The maximum depth upon grinding was 0.70 inches.

4. Steam Generator 24

UT investigation was extended to include 360° around the 10" band. Subsequent MT confirmed the presence of 7 additional indications. The maximum depth upon grinding was 1.0 inch.

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All of the above indications were removed by grinding and contouring. The ground out geometries were evaluated according to the criteria contained in WCAP-12573 and found acceptable for the next cycle of operation.

Eight UT indications were characterized as sub-surface and are as follows:

<u>Steam Generator</u>	<u>No. of indications</u>
21	1
22	5
23	0
24	2

These indications were evaluated to the criteria of IWB-3600 of the ASME B & PV Code Sec. XI and determined to be acceptable without repair.

Inspection Port Examination:

A 2" inspection port in the transition cone of Steam Generator 22 was examined by Liquid Penetrant method. Pitting was observed on the surface of the bore but no linear indications were found.

Girth weld Area Pitting:

A band of vessel ID surface was inspected for pitting from 12" above the girth weld to the tube-sheet. The results of this examination verify that the extent of pitting diminishes significantly 4" below the center line of the girth weld.

All other sections of 2.0 "Inspection Results" remain unchanged from the status reported in the WCAPs and as presented to the NRC.

Should there be any questions, please contact Mr. Charles W. Jackson, Manager, Nuclear Safety and Licensing.

Very truly yours,



cc: Mr. Thomas T. Martin
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