

Stephen B. Bram
Vice President

Consolidated Edison Company of New York, Inc.
Indian Point Station
Broadway & Bleakley Avenue
Buchanan, NY 10511
Telephone (914) 737-8116

August 25, 1989

Re: Indian Point Unit No. 2
Docket No. 50-247

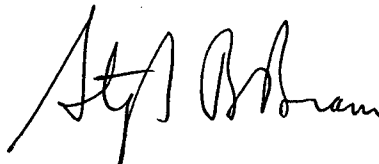
Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

SUBJECT: Response to Request for Additional Information on Spent Fuel
Pool Rerack (TAC 72962)

This is in response to your letter dated July 28, 1989 requesting additional information to support our license amendment request, dated June 20, 1989, to expand our spent fuel storage capacity. The requested information is provided in the attachment to this letter.

Should you or your staff have any questions regarding this matter, please contact Mr. Jude G. Del Percio, Manager, Regulatory Affairs and Safety Assessment.

Very truly yours,



attachment

cc: Mr. William Russell
Regional Administrator - Region I
US Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406-1498

Mr. Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects I/II
US Nuclear Regulatory Commission
Mail Stop 14B-2
Washington, DC 20555

Senior Resident Inspector
US Nuclear Regulatory Commission
PO Box 38
Buchanan, NY 10511

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INDIAN POINT UNIT NO. 2
Docket No. 50-247

SPENT FUEL POOL RERACK

Request for Additional Information

- 1.0 In Section 9.0, In-Service Surveillance Program, it is stated that in the event of degradation of the surveillance coupons, its significance will be determined and corrective action will be identified, if necessary. Describe corrective actions that would be considered to assure safe storage of the fuel if Boraflex degradation is found during in-service surveillance.

Response

In the event that degraded Boraflex is found during the Boraflex in-service surveillance program, actions will be taken to assure safe storage of fuel in the spent fuel pool. First, the degraded Boraflex would be evaluated to determine the effect on the criticality safety analysis. Based on the results of the evaluation, direct testing of the Boraflex panels in the storage racks would be considered. This testing would involve techniques such as blackness testing and/or neutron radiography. If the results of the above evaluation or direct testing indicated that the storage racks could not assure a $k_{eff} \leq 0.95$, the following corrective actions would be considered:

1. Cessation of further additions of fuel assemblies to affected cells until there is reasonable assurance that $k_{eff} \leq 0.95$ will be maintained.
2. Relocation of fuel assemblies from affected to non-affected cells.
3. Increasing the soluble boron concentration in the spent fuel pool.
4. Addition of poison material to affected cells such as control element assemblies in the stored fuel assemblies.
5. Using administrative controls to limit the enrichment and/or burnup of fuel assemblies to be placed in affected cells.

We believe that taking one or more actions similar to these would be successful in satisfying the k_{eff} value set forth in the criticality safety analysis.

DISTRIBUTION

Docket file

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August 15, 1989

DOCKET NO(S). 50-247

Mr. Stephen B. Bram
 Vice President, Nuclear Power
 Consolidated Edison Company of New York, Inc.
 Broadway and Bleakley Avenue
 Buchanan, New York 10511

SUBJECT: CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
 INDIAN POINT NUCLEAR GENERATING STATION 2

The following documents concerning our review of the subject facility are transmitted for your information.

- ☐ Notice of Receipt of Application, dated _____.
- ☐ Draft/Final Environmental Statement, dated _____.
- ☐ Notice of Availability of Draft/Final Environmental Statement, dated _____.
- ☐ Safety Evaluation Report, or Supplement No. _____ dated _____.
- ☐ Environmental Assessment and Finding of No Significant Impact, dated _____.
- ☐ Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- ☒ Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated August 9, 1989 ~~XXXXXX~~ September 8, 1989.
- ☐ Exemption, dated _____.
- ☐ Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- ☐ Facility Operating License No. _____, Amendment No. _____ dated _____.
- ☐ Order Extending Construction Completion Date, dated _____.
- ☐ Monthly Operating Report for _____ transmitted by letter dated _____.
- ☐ Annual/Semi-Annual Report- _____
 _____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

Enclosures:
 As-stated

cc: See next page

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SURNAME	CVogan						
DATE	8/15/89						