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August 8, 1988

Re: Indian Point Station Unit No. 2
Docket No. 50-247

Document Control Desk
U.S. Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

SUBJECT: Clarification of Steam Generator Leakage
Temporary Procedural Enhancements

In Attachment I of our December 23, 1987 letter on Susceptibility of Indian Point Unit No. 2 Steam Generators to High Cycle Fatigue Tube Failures, we described our revised procedural enhancements relative to steam generator primary-to-secondary leakage. That revision superseded the procedural enhancements discussed in Section 3 of Attachment B to our November 23, 1987 submittal on the same topic.

As outlined in the December 23, 1987 letter, when plant management has determined that primary-to-secondary leakage is projected to equal or exceed 0.2 gpm within the next 24 hours, unit load is to be reduced to 50% or less within two hours. In addition, if the actual leak rate is equal to or greater than 0.2 gpm, the unit will be removed from service. This administrative limitation is imposed by us to help preclude steam generator tube degradation and provide an ample margin to ensure that the 0.3 gpm limit of Technical Specification 3.1.F.2.a is not met or exceeded.

As stated in the December 23, 1987 letter and in our March 25, 1988 response to NRC Bulletin 88-02, these procedure revisions will remain in effect pending resolution of concerns regarding the potential susceptibility of the unit's steam generators to high cycle fatigue tube failure. At such time, we will evaluate the need to continue these procedural enhancements.

If you have any questions on this matter, please contact Mr. Jude G. Del Percio, Manager of Regulatory Affairs.

Very truly yours,



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