

December 24, 1987

Docket No. 50-247

Mr. Stephen B. Bram  
Vice President, Nuclear Power  
Consolidated Edison Company  
of New York, Inc.  
Broadway and Bleakley Avenue  
Buchanan, New York 10511

Dear Mr. Bram:

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 2 REACTOR VESSEL  
AUGMENTED INSPECTION (TAC 64457)

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The Indian Point Nuclear Generating Unit No. 2 Technical Specifications, Section 4.2.4 requires augmented inspection of the reactor vessel at an interval of three times during the second ten year inservice inspection interval. This requirement was imposed following the discovery of a vessel indication during the 1984 refueling outage. The first augmented inspection took place during the 1987 refueling outage. The Technical Specifications also require that the inspection results be forwarded, for NRC review and approval, prior to plant startup. By letter dated November 25, 1987, Consolidated Edison provided the applicable inspection results.

The NRC staff has reviewed the results and concludes the following:

1. The licensee used state-of-the-art instrumentation during the 1987 inspection.
2. The flaw indication is essentially unchanged.
3. The most conservative dimensions of the flaw indication presented by the licensee are enveloped by the staff evaluation dated October 16, 1984.

Therefore, the staff concludes that the licensee's submittal provides sufficient information to approve startup at full power. Our Safety Evaluation supporting this conclusion is enclosed.

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The Indian Point 2 Technical Specifications also contain a provision that the requirement for augmented inspection becomes null and void if the NRC determines that the dimensions of the flaw indications are within the ASME acceptable standards. The staffs' review with respect to flaw size is ongoing and will be the subject of future correspondence.

Sincerely,

Robert A. Capra, Director  
Project Directorate I-1  
Division of Reactor Projects, I/II

Enclosure:  
As stated

cc: See next page

PDI-1 *MS*  
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