Murray Selman
Vice President

Consolidated Edison Company of New York, Inc.
Indian Point Station
Broadway & Bleakley Avenue
Buchanan, NY 10511
Telephone (914) 737-8116

November 26, 1986

Re: Indian Point Unit No. 2
Docket No. 50-247

Ms. Marylee Slosson, Project Manager
PWR Project Directorate No. 3
Division of PWR Licensing - A
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Ms. Slosson:

This is in response to your letter dated June 30, 1986 which forwarded a preliminary technical evaluation of our NUREG-0737, Supplement 1 (Generic Letter 82-33), review of post accident monitoring instrumentation at Indian Point Unit No. 2 (IP2) submitted on August 30, 1985.

By Consolidated Edison letter dated September 12, 1986, we submitted to Mr. Hugh L. Thompson, Jr., our response to the NRC technical evaluation report (TER). In that letter, we stated that our response to section 3.3.29 of the preliminary TER (Vent from Steam Generator Safety Relief Valves) would be provided by November 30, 1986. That response is transmitted in Attachment A to this letter.

Should you or your staff have any questions on this matter, please contact us.

Very truly yours,

John A. Basile

attach.

cc: Senior Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 38
Buchanan, New York 10511

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Attachment A

Response to Item 3.3.29

Consolidated Edison Company of New York, Inc.
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Vent From Steam Generator Safety Relief Valves (3.3.29):

The Emergency Operating Procedures (EOPs) and the Emergency Plan implementation procedures direct the Operator and/or Senior Watch Supervisor to request the Chemistry organization to obtain samples and perform isotopic analyses during an emergency. The samples are obtained and analyzed on an as needed basis and the frequency of sampling and analysis is based on the event scenario. There is no set frequency. The Technical Support Center can also request samples to be taken and analyses to be performed on an as needed basis. Additionally, the Core Damage Assessment procedure involves sampling and analyses. The instruments used to analyze the samples are calibrated to NBS standards.

The Senior Watch Supervisor and/or the Operations Manager conservatively estimate the duration of the release based on the sequence of events that occurred. If doubt exists, a conservative default value is used pursuant to TSC or EOF direction.

To determine the actual radioactive release rates conservative analysis of samples and conservative flow rates are used. Conservative default values would be used if the data is questionable. We are confident that the release assessment will be conservative and within an acceptable tolerance from the actual release.