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Letter No. 81-27

February 10, 1981

Re: Indian Point Unit No. 2
Docket No. 50-247

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Denton:

This letter provides revisions to certain assumptions utilized in the Thermal-Hydraulic Analysis Report portion of the "Final Design Report for Reracking the Indian Point Unit No. 2 Spent Fuel Pool", transmitted to you by letter dated May 6, 1980. Specifically these revisions are:

- (1) Rated Power = 2758 MWt
- (2) Uncertainty Factor K, for P/Po, is 0.1
- (3) 76 assemblies discharged per cycle after 1982
- (4) Spent Fuel Pool heat removal capability at 135°F of 16.76×10^6 BTU/hr
- (5) Maximum pool temperature of 155°F, calculated from assumptions (1)-(4).

With these assumptions and the calculational methodology used in the May 6, 1980 report, the results are either unchanged or less limiting (i.e., more conservative) than those previously reported. Thus the conclusions of the May 6, 1980 report are unchanged.

Should you or your staff have any questions, please contact us.

Very truly yours,

John D. O'Toole

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