John D. O'Toole
Assistant Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, N Y 10003
Telephone (212) 460-2533

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US NRC
DISTRIBUTION SERVICES
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November 26, 1980

Re: Indian Point Unit No. 2
Docket No. 50-247

Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

ATTN: Mr. Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Dear Mr. Varga:

As requested by your October 9, 1980 letter, an analysis has been performed for the loss of main feedwater, assuming a single failure, which, due to the preset position of the auxiliary feedwater system control valves, results in 150 gpm of auxiliary feedwater being delivered to each of two steam generators. This flow rate is assumed to commence upon initiation of auxiliary feedwater for a period of ten minutes, at which time operator action is taken to increase the flow rate to 200 gpm to each of two steam generators.

The analysis was performed assuming the plant initially operating at 102% of the licensed power rating, (calorimetric error) and loss of main feedwater concurrent with a station blackout. As discussed in our August 11, 1980 letter, the loss of main feedwater concurrent with station blackout establishes the minimum flow requirements for the auxiliary feedwater system. Other values of plant parameters considered in this analysis are the same as those considered in the previous analysis described in our August 11, 1980 letter, for which no operator action was required.

The results of the analysis demonstrate that the reduced auxiliary feedwater flow rate profile of 300 gpm for the first 10 minutes, followed by 400 gpm thereafter, in conjunction with an initial

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power rating of 102% of the Indian Point Unit No. 2 power rating will satisfy the auxiliary feedwater design basis.

Should you or your staff have any additional questions, please contact us.

Very truly yours,

John D. O'Toole

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