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Vice President

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Indian Point Station
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September 30, 1988

Re: Indian Point Unit No. 2
Docket No. 50-247

Document Control Desk
U.S. Nuclear Regulatory Commission
Mail Station Pl-137
Washington, DC 20555

SUBJECT: Application for License Amendment to Increase
Authorized Power Level

Consolidated Edison Company of New York, Inc. transmits herewith one (1) signed original and two (2) copies of a document entitled "Application for Amendment to Operating License," sworn to on September 30, 1988, together with supporting attachments. This application for an amendment to Facility Operating License No. DPR-26 for Indian Point Unit No. 2, if approved, would revise the Indian Point Unit No. 2 License and Technical Specifications to authorize operation of the plant at a Nuclear Steam Supply System (NSSS) power level up to 3083.4 MWt (Megawatt thermal), the original plant guaranteed power level.

Such an adjustment in the authorized power level would benefit the general public and Con Edison's customers by enhancing the usefulness of Indian Point Unit No. 2 as a safe and dependable source of power for Con Edison's service territory. If approved, the application would authorize operation of the facility at a power level which is well within the capability of the facility's components.

The attachments to this application provide the results of analyses supporting the requested increase in authorized power level. These analyses include a review of the limiting facility systems, components and safety analyses (under both Loss of Coolant Accident ("LOCA") and non-LOCA conditions) at power levels up to 3083.4 MWt over a complete range of operating temperature and pressure conditions, and establish that the facility is capable of operating safely at the requested power level without major modifications to existing systems, structures, or components. These results are not unexpected since the plant was originally guaranteed for 3083.4 MWt NSSS power level and safeguards analysis of 3216 MWt. The completed LOCA analyses and limiting non-LOCA analyses are being submitted under separate cover as part of the Indian Point Unit No. 2 Cycle 10 Optimized Fuel Assembly (OFA) Transition Reload Report, application for amendment dated September 30, 1988. The present application presumes approval of the referenced OFA Transition application.

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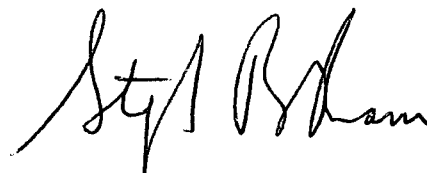
Confirmatory analyses of non-limiting evaluations will be submitted to you within 120 days of the date hereof. Additionally, any revisions to the Indian Point Unit No. 2 Final Safety Analysis Report ("FSAR") which would be required by a revision in authorized power level will be incorporated into the FSAR during the next annual update following approval and implementation of the approved application. We request an expeditious review of this application to facilitate necessary fuel management, so that all necessary adjustments associated with increased power level operation can be implemented by start-up from 1989 refueling outage currently scheduled to commence in March, 1989.

As discussed in more detail in the enclosed documentation, the suitability of the unit in its current configuration for operation at a NSSS power level of 3083.4 MWt has been confirmed on three separate occasions: a) during initial plant design, b) by analyses of plant safety features performed and accepted by the Atomic Energy Commission supporting original plant licensure (which were performed at an assumed power level of 3216 MWt), and c) during confirmatory analyses, performed using latest NRC-approved codes and methodology, in support of the current application. Because of the design and licensing history of the unit, which bounds the power level sought by the present application, we anticipate that the Notice of License Application to be published by the Commission will be accompanied by a statement that the Commission agrees with our determination that there are no significant hazards considerations with this application.

This application has been reviewed and approved by the Indian Point Station Nuclear Safety Committee and by Con Edison's Nuclear Facilities Safety Committee. Both committees concur that this application does not involve a significant hazards consideration pursuant to the criteria set out in 10 CFR §50.92(c).

Pursuant to 10 CFR §170.12, the appropriate filing fee is enclosed.

Very truly yours,

A handwritten signature in cursive script, appearing to read "Stephen R. Boham".

Attachments

cc: Mr. William Russell
Regional Administrator - Region I
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