ATTACHMENT A

APPLICATION FOR AMENDMENT
TO OPERATING LICENSE

Technical Specification
Page Revisions

Consolidated Edison Company of New York, Inc.

Indian Point Unit No. 2
Docket No. 50-247
Facility Operating License No. DPR-26

February, 1984

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Δ T trip setpoint for three loop operation has been set in accordance with specification 2.3.1.B-4.

d. Reactor operation with one of the four loops out of service will be permitted for up to 24 hours. If the fourth loop can not be returned to service within 24 hours, the reactor will be put in a hot shutdown condition using normal procedures.

2. Steam Generator

Two steam generators shall be capable of performing their heat transfer function whenever the reactor is critical and the average coolant temperature is above 350°F.

3. Safety Valves

a. At least one pressurizer code safety valve shall be operable, or an opening greater than or equal to the size of one code safety valve flange shall be provided to allow for pressure relief, whenever the reactor head is on the vessel except for hydrostatically testing the RCS in accordance with Section XI of the ASME Boiler and Pressure Vessel Code.

b. All pressurizer code safety valves shall be operable whenever the reactor is critical.

c. The pressurizer code safety valve lift setting shall be set at 2485 psig with $\pm$1% allowance for error.

4. Power Operated Relief Valves (PORVs)/Block Valves

a. Whenever the reactor coolant system is above 350°F, the PORVs and their associated block valves shall be operable with the block valves either open or closed.

b. If a PORV becomes inoperable when above 350°F, its associated block valve shall be maintained in the closed position.

c. If a PORV block valve becomes inoperable when above 350°F, the block valve shall be closed and deenergized.

d. If the requirements of specifications 3.1.A.4.a, 3.1.A.4.b or 3.1.A.4.c above cannot be satisfied, compliance shall be established within one (1) hour, or the reactor shall be placed in the hot shutdown condition within the next six (6) hours and subsequently cooled below 350°F.

e. With regard to the use of the PORVs/Block Valves as a reactor coolant system vent, the requirements of specification 3.16 shall be adhered to.

3.16 REACTOR COOLANT SYSTEM VENTS

Applicability

Applies to the operability of the reactor coolant system vents.

Objective

To define those limiting conditions for operation that are necessary to ensure the ability to exhaust noncondensible gases from the primary coolant system.

Specification

A. Whenever the reactor coolant system is above 350°F the reactor vessel head vent, and at least one of the two pressurizer steam space vents shall be operable with associated valve positions as follows:

1. Reactor Vessel Head Vent:
   (a) HCV-3100 and HCV-3101 closed
   (b) Manual valve 500 open

2. Pressurizer Steam Space Vent No. 1:
   (a) PCV-455C closed
   (b) MOV-535 open or closed

3. Pressurizer Steam Space Vent No. 2:
   (a) PCV-456 closed
   (b) MOV-536 open or closed

In addition, the PORVs (PCV-455C and 456) may be operated as necessary beyond their use as a reactor coolant system vent in accordance with approved plant procedures.

B. With the reactor vessel head vent inoperable or both pressurizer steam space vents inoperable*, startup and/or power operation may continue provided the inoperable vent(s) is (are) maintained closed with power removed from the valve actuator of all the valves in the inoperable vent(s). Restore either the reactor vessel head vent or one of the two pressurizer steam space vents respectively to operable status to meet the requirements of 3.16.A within 30 days, or the reactor shall be placed in the hot shutdown condition within the next 6 hours, and in cold shutdown within the following 30 hours.

* The requirements of Specification 3.1.A.4 shall also be adhered to.

C. With all of the above reactor coolant system vents inoperable*, maintain the inoperable vents closed with power removed from the valve actuators of all the valves in the inoperable vents, restore one vent to operable status within 72 hours and apply the requirements of Specification 3.16.B. If this cannot be accomplished, the reactor shall be placed in the hot shutdown condition within the next 6 hours, and in cold shutdown within the following 30 hours.

BASIS

Reactor Coolant System Vents are provided to exhaust noncondensible gases from the primary coolant system. The operability of two reactor coolant system vents from the reactor vessel head and the pressurizer steam space ensures that capability exists to perform this function.

The valve redundancy of the reactor coolant system vents serves to minimize the probability of inadvertent or irreversible actuation while ensuring that a single failure of a vent valve power supply or control system does not prevent isolation of the vent path.

The function, capabilities, and testing requirements of the reactor coolant system vent systems are consistent with the requirements of Item II.B.1 of NUREG-0737, "Clarification of TMI Action Plan Requirements", November 1980.

Reference

FSAR-Section 4.2.10

* The requirements of Specification 3.1.A.4 shall also be adhered to.

4.20 REACTOR COOLANT SYSTEM VENTS

Applicability

Applies to the periodic testing requirements for the reactor coolant system vents at refueling intervals.

Objective

To verify the operability of the reactor coolant system vents and their ability to exhaust noncondensible gases from the primary system when required.

Specification

A. Each reactor coolant system vent shall be demonstrated operable at refueling intervals by verifying flow through the reactor coolant system vents during cold shutdown.

Basis

The requirement specified in 4.20.A establishes the surveillance test to be performed at refueling intervals to verify the operability of the reactor coolant system vents. This qualitative flow test will verify that the vents identified in specification 3.16.A. will be available to exhaust gases from the primary coolant system by demonstrating that no blockage exists in the vent system paths.

The periodic testing required by the ASME Code Section XI for each valve in the vents is conducted as specified in the Indian Point Unit No. 2 Inservice Inspection and Testing Program and is therefore not included in these specifications.

ATTACHMENT B

APPLICATION FOR AMENDMENT
TO OPERATING LICENSE

Safety Assessment

Consolidated Edison Company of New York, Inc.
Indian Point Unit No. 2
Docket No. 50-247
Facility Operating License No. DPR-26

February, 1984

Safety Assessment

By letter dated November 1, 1983, the NRC requested all pressurized water reactor licensees to propose new technical specification revisions to incorporate several NUREG-0737 items which were scheduled for implementation after December 31, 1981.

This proposed technical specification revision, contained in Attachment A, incorporates the requirements pursuant to 10 CFR 50.44 and relative to NUREG-0737, Item II.B.1, "Reactor Coolant System Vents". The proposed changes contained in this Application have been based on the model technical specifications provided with the NRC's November 1, 1983 letter and will provide additional assurance that the Indian Point Unit No. 2 facility is operated within the limits determined appropriate following implementation of NUREG-0737 Item II.B.1. We propose to operate with the reactor vessel head vent (HCV-3100, 3101) valves closed and their associated power supplies either energized or de-energized.

By letter dated September 9, 1983, NRC provided a Safety Evaluation relative to the design of the reactor coolant system vents for Indian Point Unit No. 2 and found it acceptable. The NRC evaluation concluded that the design of the Indian Point Unit No. 2 RCS vent system is sufficient to effectively vent noncondensible gases from the reactor coolant system without leading to an unacceptable increase in the probability of a LOCA or a challenge to containment integrity. They further concluded that the system meets the design requirements of NUREG-0737 Item II.B.1 and conforms to the requirements of paragraph (c)(3)(iii) of 10 CFR 50.44.

The Commission has provided guidance for determining whether a significant hazards consideration exists by providing examples of amendments that are considered not likely to involve significant hazards considerations (48 FR 14870). Such examples include changes that constitute additional limitations not presently found in the Technical Specifications and that make the license conform to changes in the regulations. Since these proposed changes add requirements and ensure compliance with the 10 CFR 50.44 regulations in accordance with NRC positions, they are similar to the examples for which no significant hazards consideration exists.

The proposed change has been reviewed by the Station Nuclear Safety Committee and the Consolidated Edison Nuclear Facilities Safety Committee. Both Committees concur that these changes do not represent a significant hazards consideration and will not cause any change in the types or increase in the amounts of effluents or any change in the authorized power level of the facility.