



January 28, 2010
NND-10-0040

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ATTN: Document Control Desk

Subject: Virgil C. Summer Nuclear Station (VCSNS) Units 2 and 3 Combined License Application (COLA) - Docket Numbers 52-027 and 52-028 Submittal of Roadmap of Changes for the January 28, 2010 Submittal of the COLA.

References:

1. Letter from Ronald B. Clary (SCE&G) to Document Control Desk (NRC), Submittal of Revision 1 of the Combined License Application for V.C. Summer Nuclear station Units 2 and 3, dated July 30, 2009.
2. Letter from Ronald B. Clary (SCE&G) to Document Control Desk (NRC), January 28, 2010 Submittal of the Combined License Application for V.C. Summer Nuclear Station Units 2 and 3, Including Revision 2 of the Final Safety Analysis Report (FSAR).

By letter dated March 27, 2008, South Carolina Electric & Gas Company (SCE&G) submitted a combined license application (COLA) for two Westinghouse AP1000 units, designated V.C. Summer Nuclear Station (VCSNS) Units 2 and 3, to be located at the existing VCSNS site in Fairfield County, South Carolina. By letter dated January 28, 2010, SCE&G submitted an update to the VCSNS Units 2 and 3 COLA. Enclosed is a 'roadmap' of changes included in the January 28, 2010 update, which is being provided as a reviewer aid. Also provided is an explanation of the roadmap column headings.

Should you have any questions, please contact me by telephone at (803) 345-4191, or by email at apaglia@scana.com.

Very truly yours,


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JEF/AMP/jf

Enclosure – COLA January 28, 2010 Submittal Roadmap

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NND-10-0040

Enclosure

VCSNS Units 2 and 3 COL Application

January 28, 2010 Submittal Roadmap

(49 Pages Including Coversheet and Column Index Page)

**VCSNS Units 2 & 3 COL Application January 28, 2010 Submittal Roadmap Format
Explanation
(by columns)**

Column Label	Explanation
Change ID #	Unique internal identifier for tracking purposes
COLA REP	Identifies the change as STD (standard) or VCSNS (VCS) specific
COLA Part REP	Part 1 (PT01) through Part 14 (PT14)
Chapter REP	FSAR Chapter
Section / Page REP	Identifies specific location of the change in the COLA. Page numbers, if identified, are specific to the document that was revised, i.e., Revision 2.
Complete Change Description	Short description of the change
Basis for Change	The source of the change

NuStart's COLA Tracking Management (CTM) : COLA Changes | COLA Rev 2 Roadmap

JAN-28-2010 1:04 PM

COLA Rev 2 Roadmap							VC Summer is 1 AND ...
Change ID#	COLA REP	COLA Part- REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change	
PT01 - (empty)						6 COLA Changes	
6497	VCS	PT01		01.00	Revise the first paragraph, under Section 1.0 to read (change location of headquarters from Columbia, SC to Cayce, SC): This Combined License Application (COLA) is submitted by South Carolina Electric & Gas Company (SCE&G) and the South Carolina Public Service Authority (commonly referred to as "Santee Cooper"), for construction and operation of two nuclear power generating plants designated as Virgil C. Summer Nuclear Station (VCSNS) Units 2 and 3. SCE&G is the principal subsidiary of SCANA Corporation, an energy-based holding company with headquarters in Cayce, South Carolina. Santee Cooper is South Carolina's state-owned electric and water utility, with corporate headquarters in Moncks Corner, South Carolina. SCE&G has been authorized by Santee Cooper to act as their agent in applying for a Combined License (COL) for VCSNS Units 2 and 3. In addition to seeking a COL to construct and operate VCSNS Units 2 and 3, this application also seeks, through the inclusion of appropriate provisions in the COL, authorization to possess and use such quantities of source, byproduct, and special nuclear material as are needed to construct and operate the new units.	Editorial change based on relocation of corporate offices	
6519	VCS	PT01		01.00	Add COLA Part 14 to the listing of COLA Parts as follows: Part 14 - COLA Enclosure 4 - Mitigative Strategies Description and Plans	SCE&G Letter NND-09-0162; Loss of Large Areas of the Plant due to Explosions or Fire - Mitigative Strategies Description and Plans	
6520	VCS	PT01		01.02.01	Correct AP 1000 to read AP1000 (delete extra space)	Editorial	
6835	VCS	PT01		01.02.04	Add a period at the end of each Reference to be consistent with other parts of the application.	Editorial	
6836	VCS	PT01		01.03.03.01	Revise definition for Fx from: Add a leading zero to the date for item Fx to read: Fx = 3/2007 Light Fuel Oils PPI/01/1986 industrial electric power PPI = 172.9/114.2 = 1.514 to read: Add a leading zero to the date for item Fx to read: Fx = 03/2007 Light Fuel Oils PPI/01/1986 industrial electric power PPI = 172.9/114.2 = 1.514	Editorial	
6502	VCS	PT01		Table 1	Revise COLA Part 1, Table 1 to reflect current corporate address, update officers, and officer titles:	Relocation of corporate offices and updated listing of company officers	
PT02 - FSAR01						58 COLA Changes	
6089	STD,VCS	PT02	FSAR01	01.01.05	Remove first sentence that states "Reference 1 to DCD Chapter 1 discusses the generic construction plan and startup schedule for the AP1000."	WEC DCD Rev 17 conforming change - Reference 1 was removed from DCD.	
6090	STD,VCS	PT02	FSAR01	01.01.T / T1.1-201 SNM	COLA Part 2, FSAR Chapter 1, Section 1.1, Table 1.1-201, will be revised to add a new acronym of SNM for "Special Nuclear Material"	Conforming change associated with COL-SER-01-Ch01 response to 01-05-01 item 1	
5867	VCS	PT02	FSAR01	01.01F / F1.1-202	Revise Figure 1.1-202 to add the following note: "The Offsite Water Treatment Facility is not within the scope of the COLA."	RAI 09.02.01-6 (Letter 061 Response per NND-09-0268)	
6306	VCS	PT02	FSAR01	01.01F / F1.1-202	Revise Figure 1.1-202 for Site Plan changes listed below:	To incorporate Site Plan changes.	

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					1. The intake and discharge of the Water Treatment Plant are depicted and differ from that shown on Figure 1.1-202, Revision 1. 2. The construction access road is relocated. 3. The Discharge Line access road is relocated. 4. Transmission line routing from the Unit 2 & 3 switchyard to the Unit 1 switchyard is relocated. 5. Wetlands area changes. 6. Spoils area changes. 7. Plant coordinates for Unit 3 to match the most recent coordinates in FSAR Section 2:1.1:1.	
6091	STD,VCS	PT02	FSAR01	01.02.03	COLA Part 2, FSAR Chapter 1, Section 1.2.3 will be added with an Left Margin Annotation of VCS DEP 18.8-1, to read: 1.2.3 PLANT ARRANGEMENT DESCRIPTION ----- Add the following information at the end of the first paragraph of DCD Subsection 1.2.3. Figure 1.2-201 replaces DCD Figure 1.2-18 to reflect the relocation of the Operations Support Center. -----	Consistency - Revised figures were not identified in the FSAR text
6414	VCS	PT02	FSAR01	01.04.01	The third paragraph of FSAR Subsection 01.04.01 will be revised to read (Change SCANA location from Columbia to Cayce): SCE&G is the principal subsidiary of SCANA Corporation, an energy-based holding company with headquarters in Cayce, South Carolina. SCE&G is a regulated public utility engaged in the generation, transmission, distribution, and sale of electricity in South Carolina. SCE&G is also engaged in the purchase and sale of natural gas in central and southern South Carolina.	Update of new company location
6095	STD,VCS	PT02	FSAR01	01.06.T / T1.6-201	COLA Part 2, FSAR Chapter 1, Table 1.6-201, entries for NEI 07-03, NEI 06-13A, and NEI 07-02A will be revised in accordance with response to NRC SER CI Number 12.01-01, item 1 in SNC Letter ND-09-1529.	COL-SER-CI-Ch12 response to CI 12.01.01 item 1 SNC Letter #ND-09-1529
6096	STD,VCS	PT02	FSAR01	01.06.T / T1.6-201	COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, footnote a) will be deleted from the NEI 07-02 entry, added to the column heading, and revised to read: a) The NRC-accepted NEI documents identified by the A in the document number include the accepted template, the NRC safety evaluation, and corresponding responses to the NRC Requests for Additional Information. Only the accepted template is incorporated by reference. The remainder of the document is referenced but not incorporated into the FSAR.	COL-SER-CI-Ch12 response to CI 12.01.01 item 2 SNC Letter #ND-09-1529
6098	STD,VCS	PT02	FSAR01	01.06.T / T1.6-201	COLA Part 2, FSAR Chapter 1, Table 1.6-201, entry for NEI 07-08 will be revised as indicated in response to NRC SER OI Number 12.01-01, item 1 in SNC Letter ND-09-1770.	COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 1 SNC Letter ND-09-1770
6099	STD,VCS	PT02	FSAR01	01.06.T / T1.6-201	2. COLA Part 2, FSAR Chapter 1, Table 1.6-201 footnote will be revised from (A) Denotes NRC approved document. Other listed documents are under NRC review. To read: (A) Denotes NRC approved document.	COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 2 SNC Letter ND-09-1770
6100	STD,VCS	PT02	FSAR01	01.06.T / T1.6-201	COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, NEI 06-13A title, will be revised from: Technical Report on a Template for an Industry Training Program Description To read: Template for an Industry Training Program Description	Editorial correction to be consistent with template title.
5722	VCS	PT02	FSAR01	01.08	1. FSAR Section 1.8 will be revised to include the following paragraph at the end of the section with a left margin annotation (LMA) of VCS SUP-1.8-3: DCD Table 1.8-1 presents interface items for the AP1000. FSAR section(s) addressing these interface items are tabulated in Table 1.8-203.	RAI.01-04 (Letter 055 Response per NND-09-0233)

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
6444	VCS	PT02	FSAR01	01.08.T / T1.08-201	<p>Add VCS DEP 2.0-2 to Table 1.8-201 as follows: VCS DEP 2.0-2</p> <p>The site parameter value provided in the DCD Tier 1, Table 5.0-1 for the air temperature maximum wet bulb (noncoincident) is 86.1°F. This site parameter value is listed as the maximum safety wet bulb (noncoincident) air temperature in DCD Tier 2, Table 2-1. The corresponding site characteristic value is 87.3°F as reported in FSAR Subsection 2.3.1.5. This site characteristic exceeds the DCD site parameter by 1.2°F.</p> <p>2.0 2.3.1.5 5.4.7.1 9.2.2.1</p>	Response to NRC Request for Additional Information (RAI) Letter No.064, NND-09-0274.
6651	VCS	PT02	FSAR01	01.08.T / T1.8-201	Add Subsection 12.3.1.2 to the VCS DEP 18.8-1 entry in Table 1.8-201.	Consistency. Subsection 12.3.1.2 has a LMA of VCS DEP 18.8-1 and should be listed in the corresponding entry for Table 1.8-201.
6660	VCS	PT02	FSAR01	01.08.T / T1.8-202 12.3-1	Add Subsection 12.5.4 to Table 1.8-202 for COL Item 12.3-1 entry.	Consistency with COL-SER-CI-Ch12 response to CI 12.01.01 item 8 SNC Letter #ND-09-1529
6661	VCS	PT02	FSAR01	01.08.T / T1.8-202 14.4-3	Add Subsections 14.2.1.4, 14.2.1.5, and 14.2.3 to Table 1.8-202 for COL Item 14.4-3 entry.	Consistency.
6662	VCS	PT02	FSAR01	01.08.T / T1.8-202 14.4-4	Add Subsection 14.2.3.3.1 to Table 1.8-202 for COL Item 14.4-4 entry.	Consistency.
6102	STD,VCS	PT02	FSAR01	01.08.T / T1.8-202 19.59.10-4	<p>1. COLA Part 2, FSAR Chapter 1, Table 1.8-202, COL ITEM 19.59.10-4 will be changed from:</p> <p>Develop and Implement Severe Accident Management Guidance</p> <p>To read: Implement Severe Accident Management Guidance</p>	RAI LTR 152 response to RAI 19-21 item 1
5723	VCS	PT02	FSAR01	01.08.T / T1.8-203	FSAR Section 1.8 will be revised to include new table 1.8-203 with a Left Margin Annotation of VCS SUP 1.8-3, as shown in SCE&G Response to NRC RAI 01-04 per SCE&G Letter NND-09-0233.	RAI 01-04 (Letter 055 Response per NND-09-0233)
6103	STD,VCS	PT02	FSAR01	01.09.01.01	<p>1. Revise the following text in FSAR Subsection 1.9.1.1 from:</p> <p>... One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design. Stated conformance with the programmatic and/or operational aspects is only to the extent that a design change or departure from the approved DCD is not required to implement those programmatic and/or operational aspects.</p> <p>To read:</p> <p>... One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design. (see Notes at the end of Appendix 1AA).</p>	COL-SER-OI-Ch01 S1 response to OI 01:04-02 item 1
6104	STD,VCS	PT02	FSAR01	01.09.01.02	<p>2. Revise the following text in FSAR Subsection 1.9.1.2 from:</p> <p>... One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design. Stated conformance with the programmatic and/or operational aspects is only to the extent that a design change or departure from the approved DCD is not required to implement those programmatic and/or operational aspects. ...</p> <p>To read:</p>	COL-SER-OI-Ch01 S1 response to OI 01.04-02 item 2

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					... One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design (see Notes at the end of Appendix 1AA).	
6105	STD,VCS	PT02	FSAR01	01.09.01.03	<p>3. Revise the following text in FSAR Subsection 1.9.1.3 from:</p> <p>... One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design. Stated conformance with the programmatic and/or operational aspects is only to the extent that a design change or departure from the approved DCD is not required to implement those programmatic and/or operational aspects.</p> <p>To read:</p> <p>... One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design (see Notes at the end of Appendix 1AA).</p>	COL-SER-OI-Ch01 S1 response to OI 01.04-02 item 3
6106	STD,VCS	PT02	FSAR01	01.09.01.04	<p>4. Revise the following text in FSAR Subsection 1.9.1.4 from:</p> <p>... One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design. Stated conformance with the programmatic and/or operational aspects is only to the extent that a design change or departure from the approved DCD is not required to implement those programmatic and/or operational aspects. . . .</p> <p>To read:</p> <p>... One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design (see Notes at the end of Appendix 1AA).</p>	COL-SER-OI-Ch01 S1 response to OI 01.04-02 item 4
6107	STD,VCS	PT02	FSAR01	01.09.05.01.05	<p>1. COLA Part 2, FSAR Chapter 1, will be revised to include the following new Subsection 1.9.5.1.5 (with a Left Margin Annotation of STD SUP 1.9-3):</p> <p>-----</p> <p>1.9.5.1.5 Station Blackout</p> <p>Add the following text to the end of DCD Subsection 1.9.5.1.5:</p> <p>Training and procedures to mitigate a 10 CFR 50.63 "loss of all alternating current power" (or station blackout (SBO)) event are implemented in accordance with Sections 13.2 and 13.5, respectively. As recommended by NUMARC 87-00 (Reference 201), the SBO event mitigation procedures address response (e.g., restoration of onsite power sources), ac power restoration (e.g., coordination with transmission system load dispatcher), and severe weather guidance (e.g., identification of actions to prepare for the onset of severe weather such as an impending tornado), as applicable. The AP1000 is a passive design and does not rely on offsite or onsite ac sources of power for at least 72 hours after an SBO event, as described above. In addition, there are no nearby large power sources, such as a gas turbine or black start fossil fuel plant, that can directly connect to the station to mitigate the event.</p> <p>Restoration from an SBO event will be contingent upon ac power being made available from any one of the transmission lines described in Section 8.2 or any one of the standby diesel generators.</p>	RAI LTR 025 S1 response to RAI 08.01-002 item 1 SER with Open Items Confirmatory Item 8.1-1
6108	STD,VCS	PT02	FSAR01	01.09.06	<p>2. COLA Part 2, FSAR Chapter 1, will be revised to include the following new Subsection 1.9.6 (with no Left Margin Annotation):</p> <p>-----</p> <p>1.9.6 REFERENCES</p> <p>-----</p> <p>Add the following text to the end of DCD Subsection 1.9.6.</p> <p>201. NUMARC 87-00, Guidelines and Technical Bases for NUMARC Initiatives Addressing Station Blackout at Light Water Reactors, Revision 1, August 1991.</p>	RAI LTR 025 S1 response to RAI 08.01-002 item 2 SER with Open Items Confirmatory Item 8.1-1
6109	STD,VCS	PT02	FSAR01	01.09.T/T 1.9:201	<p>3. COLA Part 2, FSAR Chapter 1, Table 1.9-201, Regulatory Guide 1.8 will be revised from:</p>	COL-SER-CI-Ch12 response to CI

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
				1.008	<p>1.8 Qualification and Training of Personnel for Nuclear Power Plants (Rev. 3, May 2000)</p> <p>12.1 (NEI 07-08) Appendix 12AA Appendix 12AA (NEI 07-03) 13.1.1.4 13.1.3.1 13.2 (NEI 06-13A) 16 (TS 5.3.1)</p> <p>To read:</p> <p>1.8 Qualification and Training of Personnel for Nuclear Power Plants (Rev. 3, May 2000)</p> <p>12.1 (NEI 07-08) Appendix 12AA Appendix 12AA (NEI 07-03A) 13.1.1.4 13.1.3.1 13.2 (NEI 06-13A) 16 (TS 5.3.1)</p>	12.01.01 item 3 SNC Letter #ND-09-1529
6110	STD,VCS	PT02	FSAR01	01.09.T / T1.9-201 1.008	3. COLA Part 2, FSAR Chapter 1, Table 1.9-201, Regulatory Guide 1.8 will be revised from "12.1 (NEI 07-08)" to read "12.1 (NEI 07-08A)" in the FSAR Chapter, Section, or Subsection column.	COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 3 SNC Letter ND-09-1770
6111	STD,VCS	PT02	FSAR01	01.09.T / T1.9-201 1.016	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Revision 1, will be revised to remove RG 1.16 from the table.	This Regulatory Guide withdrawn by NRC on 8-11-2009 via 74 FR 40244.
6114	STD,VCS	PT02	FSAR01	01.09.T / T1.9-201 1.097	<p>COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, will be revised for Regulatory Guide 1.97 (delete VCS LMA and associated separator bars) to read:</p> <p>1.97 Criteria for Accident Monitoring Instrumentation For Nuclear Power Plants (Rev. 4, June 2006) Not referenced; see Appendix 1AA</p> <p>1.97 Instrumentation For Light-Water-Cooled Nuclear Power Plants to Assess Plant Environs Conditions During and Following an Accident (Rev. 3, May 1983) Table 7.5-201 Appendix 12AA 16 (TS Bases 3.3.3)</p>	Consistency with referenced revision throughout FSAR
6693	VCS	PT02	FSAR01	01.09.T / T1.9-201 1.099	Remove the unnecessary Left Margin Annotation (LMA) "STD COL 1.9-1" adjacent to 1.99.	Editorial to remove redundant LMA
6659	VCS	PT02	FSAR01	01.09.T / T1.9-201 1.147	<p>Revise Left Margin Annotation (LMA) from:</p> <p>STD COL 5.2-2</p> <p>to read:</p> <p>STD COL 1.9-1</p>	Consistency.
6663	VCS	PT02	FSAR01	01.09.T / T1.9-201 1.149	Left Margin Annotation (LMA) for Table 1.9-201 Regulatory Guide 1.149 entry is redundant and can be eliminated.	Editorial
6117	STD,VCS	PT02	FSAR01	01.09.T / T1.9-201 1.101	COLA Part 2, Chapter 1, Section 1.9, Table 1.9-201, RG 1.101, Rev. 5 from "Not referenced" to read "Not referenced; see Appendix 1AA"	Editorial consistency of entries
6118	STD,VCS	PT02	FSAR01	01.09.T / T1.9-201 1.133	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Revision 1, will be revised to change the FSAR Chapter, Section, or Subsection reference for RG 1.133 from "DCD discussion only; see DCD Table 1.9-1" to read "Not referenced; see Appendix 1AA"	Appendix 1AA contains some FSAR position statements; not all discussion is per DCD.
6119	STD,VCS	PT02	FSAR01	01.09.T / T1.9-201 1.135	<p>COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Revision 1, will be revised for RG 1.135 from:</p> <p>"DCD discussion only; see DCD Table 1.9-1"</p>	This Regulatory Guide withdrawn by NRC on 8-6-2009 via 74 FR 39349.

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					To read: "Not referenced; see Appendix 1AA"	
6120	STD,VCS	PT02	FSAR01	01.09.T / T1.9-201 1.152	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Revision 1, will be revised to change the FSAR Chapter, Section, or Subsection reference for RG 1.152 from "DCD discussion only; see DCD Table 1.9-1" to read "Not referenced; see Appendix 1AA"	Appendix 1AA contains some FSAR position statements; not all discussion is per DCD
6121	STD,VCS	PT02	FSAR01	01.09.T / T1.9-201 1.206	5. COLA Part 2, FSAR Chapter 1, Table 1.9-201, Regulatory Guide 1.206, FSAR crossreference column entry, will be revised to add "Appendix 12AA (NEI 07-03A)"	COL-SER-CI-Ch12 response to CI 12.01.01 item 5 SNC Letter #ND-09-1529
6122	STD,VCS	PT02	FSAR01	01.09.T / T1.9-201 4.15	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Revision 1, will be revised from: 4.15 Quality Assurance for Radiological Monitoring Programs (Inception through Normal Operations to License Termination) - Effluent Streams and the Environment (Rev. 1, February 1979) 11.5.1.2 11.5.3 11.5.4 11.5.6.5 To read: 4.15 Quality Assurance for Radiological Monitoring Programs (Inception through Normal Operations to License Termination) - Effluent Streams and the Environment (Rev. 2, July 2007) 11.5.3 4.15 Quality Assurance for Radiological Monitoring Programs (Inception through Normal Operations to License Termination) - Effluent Streams and the Environment (Rev. 1, February 1979) 11.5.1.2 11.5.3 11.5.4 11.5.6.5	Address latest version of the Regulatory Guide
6123	STD,VCS	PT02	FSAR01	01.09.T / T1.9-201 8.000	6. COLA Part 2, FSAR Chapter 1, Table 1.9-201, Division 8 Regulatory Guides will be revised.	COL-SER-CI-Ch12 response to CI 12.01.01 item 6 SNC Letter #ND-09-1529
6124	STD,VCS	PT02	FSAR01	01.09.T / T1.9-201 8.000	4. COLA Part 2, FSAR Chapter 1, Table 1.9-201, Regulatory Guides 8.2, 8.7, 8.8, 8.9, 8.10, 8.13, 8.15, 8.27, 8.28, 8.29, 8.34, 8.35, 8.36, and 8.38 will be revised from "12.1 (NEI 07-08)" to read "12.1 (NEI 07-08A)" in the FSAR Chapter, Section, or Subsection column.	COL-SER-OI-Ch12: S1 response to OI 12.01.001 item 4 SNC Letter ND-09-1770
6664	VCS	PT02	FSAR01	01.09.T / T1.9-201 8.08	Revise Regulatory Guide 8.8 entry in Table 1.9-202, Appendix 12AA (NEI 07-03) reference from: Appendix 12AA (NEI 07-03) to read: Appendix 12AA (NEI 07-03A)	Editorial
6665	VCS	PT02	FSAR01	01.09.T / T1.9-201 8.10	Revise Regulatory Guide 8.10 entry in Table 1.9-202, Appendix 12AA (NEI 07-03) reference from: Appendix 12AA (NEI 07-03) to read: Appendix 12AA (NEI 07-03A)	Editorial
6534	STD,VCS	PT02	FSAR01	01.09.T / T1.9-202 Sh14	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-202, for SRP Criteria 13.2.1, revise the title of NEI 06-13A from: Technical Report on a Template for an Industry Training Program Description	Editorial revision to the template title

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					To read: Template for an Industry Training Program Description	
6126	STD,VCS	PT02	FSAR01	01.10.03	COLA Part 2, FSAR Chapter 1, Subsection 1.10.3 will be revised to add the following new paragraph at the end of the subsection (under the LMA of STD SUP 1.10-1): The above discussed controls to eliminate or mitigate construction hazards that could potentially impact operating unit SSCs important to safety are in place when there is an operating nuclear unit on the site.	COL-SER-OI-Ch01 response to OI 01.04-04
6127	STD,VCS	PT02	FSAR01	01AA RG 1.016	COLA Part 2, FSAR Chapter 1, Appendix 1AA, Revision 1, conformance statement for Regulatory Guide 1.16 will be deleted in its entirety.	This Regulatory Guide withdrawn by NRC on 8-11-2009 via 74 FR 40244.
6725	VCS	PT02	FSAR01	01AA RG 1.026	Add LMA "STD COL 1.9-1" next to RG 1.26	Consistency within the table. Re-establishes applicability of STD COL 1.9-1 for this RG.
6726	VCS	PT02	FSAR01	01AA RG 1.029	Remove LMA "STD COL 1.9-1" next to RG 1.29	Due to the arrangement of the page, this RG (and those following it) is described by the LMA in the upper left corner of the page and a separate LMA for this RG is not necessary.
6129	STD,VCS	PT02	FSAR01	01AA RG 1.053	COLA Part 2, FSAR Chapter 1, Appendix 1AA, Revision 1, title for Regulatory Guide 1.53 will be revised from: "Application of the Single-Failure Criterion to Nuclear Power Plant Protection Systems" To read: "Application of the Single-Failure Criterion to Safety Systems"	Editorial to match title of the RG
6130	STD,VCS	PT02	FSAR01	01AA RG 1.097	COLA Part 2, FSAR Chapter 1, Appendix 1AA, conformance statement for Regulatory Guide 1.97 will be revised to read: "Conformance with this Regulatory Guide for programmatic and/or operational aspects is documented below. General Exception Portable equipment outside the DCD scope conforms to Revision 3 of this Regulatory Guide for consistency with DCD scope since Revision 4 indicates that partial implementation is not advised.	FSAR Revision 1 included conformance (See Table 7.5-201) for equipment outside the DCD scope.
6133	STD,VCS	PT02	FSAR01	01AA RG 1.133	COLA Part 2, FSAR Chapter 1, Appendix 1AA, conformance statement C.6 for Regulatory Guide 1.133 will be revised from: C.6 Exception See position for Regulatory Guide 1.16 To read: C.6 Exception Regulatory Guide 1.16 has been withdrawn.	Regulatory Guide 1.16 withdrawn by NRC on 8-11-2009 via 74 FR 40244. Revises BLN RAI LTR 142.S1 response to RAI 01-011, item 6. SER with Open Items Confirmatory Item 4.4-1
6727	VCS	PT02	FSAR01	01AA RG 1.133	Remove LMA "STD COL 1.9-1" next to RG 1.133	Due to the arrangement of the page, this RG (and those following it) is described by the LMA in the upper left corner of the page and a separate LMA for this RG is not necessary.
6134	STD,VCS	PT02	FSAR01	01AA RG 1.135	COLA Part 2, FSAR Chapter 1, Appendix 1AA, conformance statement for Regulatory Guide 1.135 will be revised from: Conformance of the design aspects is as stated in the DCD. Conformance with Revision 0 of this Regulatory Guide for programmatic and/or operational aspects is documented below. General Conforms To read: Conformance of the design aspects is as stated in the DCD. The programmatic and/or operational aspects are not applicable since this guidance was withdrawn by NRC (74 FR 39349, 08/06/2009).	This RG withdrawn by NRC (See 74 FR 39349, 08/06/2009).
6135	STD,VCS	PT02	FSAR01	01AA RG 1.152	COLA Part 2, FSAR Chapter 1, Appendix 1AA, Revision 1, conformance statement for Regulatory Guide 1.152 will be revised to reflect the latest revisions to the Security regulations from:	Changes to address Security Regulation revisions

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					General Exception The Cyber Security Program is based on NEI 04-04 Revision 1 which has been identified by the NRC as an acceptable means for development of the program. To read: General Exception The Cyber Security Program is based on March 2009 revisions of the 10 CFR 73.54 regulations in lieu of Revision 2 of this Regulatory Guide.	
6136	STD,VCS	PT02	FSAR01	01AA RG 1.180	COLA Part 2, FSAR Chapter 1, Appendix 1AA, Revision 1, first sentence of the conformance statement for Regulatory Guide 1.180 will be revised from: Conformance of the design aspects with Revision 0 of the Regulatory Guide is as stated in the DCD. To read: Conformance of the design aspects is as stated in the DCD.	WEC DCD Rev 17 conformance change
6138	VCS	PT02	FSAR01	01AA Note	Add separator line after the end of Appendix 1AA to denote the differentiation between the table and notes.	Editorial
6846	STD,VCS	PT02	FSAR01	01AA Note a	5. Revise FSAR Appendix 1AA Note (at the end of the Appendix) from: Note: Above stated general alternatives regarding the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD is provided to preserve the finality of the certified design. Further, each stated conformance with the programmatic and/or operational aspects is only to the extent that a design change or departure from the approved DCD is not required to implement those programmatic and/or operational aspects. To read (the # may vary for R-COLA and S-COLA as appropriate): Note:#. Above stated general alternatives regarding the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD is provided to preserve the finality of the certified design. Further, each stated conformance with the programmatic and/or operational aspects is only to the extent that a design change or departure from the approved DCD is not required to implement those programmatic and/or operational aspects. As the operational and programmatic aspects become more fully defined (for example, during the preparation, approval, or initial implementation of plant procedures), there exists a potential that a conflict could be identified between the design as certified in the DCD and the programmatic and/or operational aspects of the guidance. In such cases, the design certification (rule) becomes the controlling factor, and the design conformance to the Regulatory Guide is per the revision stated in the DCD.	COL-SER-OI-Ch01 S1 response to OI 01:04-02 item 5
6139	STD,VCS	PT02	FSAR01	01AA Note b.	7. Revise FSAR Appendix 1AA Note (at the end of the Appendix) to include the following additional note (the # may vary for the R-COLA and S-COLA as appropriate): Note b. A "Criteria Section" entry of "General" indicates a scope for the conformance statement of "all regulatory guide positions related to programmatic and/or operational aspects." Thus, an associated conformance statement of "Conforms" indicates that the applicant "complies with all regulatory guide positions related to programmatic and/or operational aspects."	COL-SER-OI-Ch01 S1 response to OI 01.04-02 item 7
6255	STD,VCS	PT02	FSAR01	APP 01B	COLA Part 2, FSAR Chapter 1, Appendix 1B, 2nd sentence will be revised from: Rather, the severe accident mitigation design alternatives will be discussed in the Environmental Report. To read: Rather, the severe accident mitigation design alternatives are addressed in the Environmental Report.	Consistency - FSAR text is in present tense.
PT02 - FSAR02						98 COLA Changes
6292	VCS	PT02	FSAR02	02.00	Revise the last sentence of FSAR Section 2.0 to read: "Except as noted in Table 2.0-201, the DCD value envelopes the site-specific site value."	Editorial change to be consistent with departure.
6499	VCS	PT02	FSAR02	02.00 / T 2.0-201 Sheet 5	Revise the X/Q values in Table 2.0-201, Sheet 5 to agree with Tables 2.3-222 and 223 in the response to RAI 02:03:03-1.	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
5980	VCS	PT02	FSAR02	02.00.T / T 2.0-201	Revise the first three rows of Table 2.0-201 as shown in response to RAI 2.3.1-7 in SCE&G Letter NND-09-0274.	RAI 2.3.1-7 (Letter 064 Response per NND-09-0274)
5981	VCS	PT02	FSAR02	02.00.T / T 2.0-201	Add note (k) to FSAR Table 2.0-201 as shown: (k) See COLA Part 7 for the exemption and departure justification for Maximum Safety Wet Bulb Temperature (Noncoincident).	RAI 2.3.1-7 (Letter 064 Response per NND-09-0274)

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change												
6441	VCS	PT02	FSAR02	02.00.T / T 2.0-201	Add LMA of VCS DEP 2.0-2 to Table 2.0-201 adjacent to VCS Maximum Safety wet bulb (noncoincident) temperature.	Departure introduced by the Response to RAI 02.03.01-7 by NND-09-0274 dated October 8, 2009 (ML092860136).												
6478	VCS	PT02	FSAR02	02.00.T / T 2.0-201	The following FSAR changes will be made in Revision 2 of the VCSNS COLA: Revise the Atmospheric Dispersion Values of Table 2.0-201 as shown in SCE&G supplemental response to NRC RAI 02.03.03-1 per SCE&G Letter NND-09-0331.	RAI-2.3.3-1 (Letter 049 Response per NND-09-0331)												
6646	VCS	PT02	FSAR02	02.00.T / T 2.0-201	Add LMA "VCS SUP 2.0-2" to Table 2.0-201	Clarification of LMAs (to make distinct from Departure LMA)												
6423	VCS	PT02	FSAR02	02.01.01.01	In FSAR Section 2.1.1.1 page 2.1-1, revise Units 2&3 coordinates table to read: <table border="1"> <thead> <tr> <th>Units</th> <th>SC State Plane (NAD 83) (Feet)</th> <th>UTM Zone 17N (84W TO 78W) (Meters)</th> <th>Latitude/Longitude (Deg/Min/Sec)</th> </tr> </thead> <tbody> <tr> <td>2</td> <td>N 892,742.50 E 1,903,286.39</td> <td>N/S 3,793,982.323 E/W 470,529.004</td> <td>N 34 17.11.81100 W 81 19 12.75308</td> </tr> <tr> <td>3</td> <td>N 891,908.03 E 1,902,949.24</td> <td>N/S 3,793,728.043 E/W 470,426.253</td> <td>N 34 17.03.54504 W 81 19 16.74066</td> </tr> </tbody> </table>	Units	SC State Plane (NAD 83) (Feet)	UTM Zone 17N (84W TO 78W) (Meters)	Latitude/Longitude (Deg/Min/Sec)	2	N 892,742.50 E 1,903,286.39	N/S 3,793,982.323 E/W 470,529.004	N 34 17.11.81100 W 81 19 12.75308	3	N 891,908.03 E 1,902,949.24	N/S 3,793,728.043 E/W 470,426.253	N 34 17.03.54504 W 81 19 16.74066	Correction of Physical Plant Coordinates
Units	SC State Plane (NAD 83) (Feet)	UTM Zone 17N (84W TO 78W) (Meters)	Latitude/Longitude (Deg/Min/Sec)															
2	N 892,742.50 E 1,903,286.39	N/S 3,793,982.323 E/W 470,529.004	N 34 17.11.81100 W 81 19 12.75308															
3	N 891,908.03 E 1,902,949.24	N/S 3,793,728.043 E/W 470,426.253	N 34 17.03.54504 W 81 19 16.74066															
6516	VCS	PT02	FSAR02	02.01.02.01	In the first line of Subsection 2.1.2.1 change "exlusion" to "exclusion".	Editorial to correct spelling.												
6517	VCS	PT02	FSAR02	02.01.02.03	In the last line of Subsection 2.1.2.3 change "Reservior" to "Reservoir".	Editorial to correct spelling.												
6307	VCS	PT02	FSAR02	02.01F / F 2.1-203	Revise Figure 2.1-203 for Site Plan changes, relabel the "Intake Channel" to "Makeup Line, and correctly depict the EAB to include the Fairfield Pumped Storage Facility.	To incorporate Site Plan changes, correct the "Intake Channel" nonenclature, and correctly depict the EAB.												
5157	VCS	PT02	FSAR02	02.02.03.01.03	The final paragraph of Section 2.2.3.1.3 will be revised as follows: For each of the identified chemicals with the exception of ammonium hydroxide, it was conservatively assumed that the entire contents of the vessel leaked, forming a 1-centimeter-thick puddle, where accommodated by the model. For those identified hazardous materials in the gaseous state, it was conservatively assumed that the entire contents of the vessel or pipeline were released over a 10-minute period into the atmosphere as a continuous direct source (Reference 229). The effects of toxic chemical releases from onsite and offsite sources are summarized in Table 2.2-209 and are described in the following subsections relative to the release sources.	RAI 06.04-3 (Letter 047 Response per NND-09-0145)												
5158	VCS	PT02	FSAR02	02.02.03.01.03.01	The second and third paragraphs of Section 2.2.3.1.3.1 will be revised as follows: As described in Subsection 2.2.3.1.3, the identified hazardous materials were analyzed using the ALOHA dispersion model to determine whether the formed vapor cloud would reach the control room intake and what the concentration of the toxic chemical would be in the control room following an accidental release. Nitrogen concentration was determined at the control room following a release from the largest storage vessel. In this case, the concentration of asphyxiant at the control room (96.2 ppm of nitrogen) would not displace enough oxygen for the control room to become an oxygen-deficient environment, nor would it be otherwise toxic at this concentration (Reference 228). The remaining chemical analysis indicates that the control room can safely remain habitable for the worst-case toxic release scenario. In evaluating the 28% ammonium hydroxide storage tank spill the following inputs were used in the model: <ul style="list-style-type: none"> • Pasquill Stability Class F selected to represent the worst 5% of meteorological conditions observed. • A low wind speed of 1 meter-per-second selected to represent the worst 5% conditions. Low wind speed conditions prevent the vapor cloud from dispersing as it travels. 	RAI 06.04-3 (Letter 047 Response per NND-09-0145)												

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/ Page REP	Complete Change Description	Basis for Change
					<ul style="list-style-type: none"> The time of day selected was 12:00 p.m. on July 1, 2006. This day and time were chosen because temperatures are highest in the summer during the midday. Higher temperatures lead to a higher evaporation rate, and thus, a larger vapor cloud. The tank was filled to capacity and a catastrophic tank failure was assumed where the total amount of the substance leaked puddle into a basin with an equivalent radius of 6.1 meters. The ammonium hydroxide storage tank is located within a dammed area with a trough draining to the waste neutralizing basin. The total dammed area, including the drain troughs and waste neutralizing basin, has an equivalent radius of 6.1 meters. Assuming that the entire contents of the tank would be contained in the dammed area is conservative given that any fluid within the dammed area would actually drain to a waste neutralizing basin consisting of a 15 foot deep reservoir with a large liquid inventory. This would serve to dilute the ammonium hydroxide, thus lowering its vapor pressure and thereby reducing the amount that evaporates. There are no physical obstructions that interfere with the toxic vapor cloud from reaching the control room intake. 	
5159	VCS	PT02	FSAR02	02.02.T / T 2.2-209	Revise FSAR Table 2.2-209 as indicated in response to RAI 06.04-3 in SCE&G Letter NND-09-0145.	RAI 06.04-3 (Letter 047 Response per NND-09-0145)
5163	VCS	PT02	FSAR02	02.03.01.03.01	<p>After the paragraph on page 2.3-7 (Subsection 2.3.1.3.1) that starts "Subsection 2.3.1.3.3 ... ", add the following:</p> <p>The Shaw Air Force Base (AFB) wind speed for Hurricane Hugo was provided as a data point because it was a source for tropical cyclones and demonstrated the unusual nature of this hurricane; however, the Shaw AFB observation is not representative of the maximum wind speed that would be observed at the site. Shaw AFB is located approximately 50 miles to the southeast of the VCSNS site, and due to its location relative to the storm path, it received the strongest of the hurricane's winds that existed at the time. The VCSNS site received winds that were on the weaker, western side of the storm. Hurricane Hugo was noteworthy for rapid inland movement and a widespread circulation. This suggests that the winds for Hugo were stronger inland than for most storms. Hurricanes that move inland decrease in wind speed, and winds continue to decrease in intensity as the storm moves further inland due to friction and loss of warm moist inflow air. Shaw AFB is positioned closer to the coast than the site is located. Hugo had observed winds of 109 mph as it passed Shaw AFB, followed by a rapid decrease in storm intensity to 70 mph at Columbia (Reference 212). Therefore Hugo had decreased in storm intensity below the site characteristic value at Columbia. While maximum wind gusts of 109 mph were reported at Shaw AFB, the maximum wind gusts associated with Hurricane Hugo at the site were much lower due to the location of the VCSNS site. On this basis it is concluded that historical Hurricane winds that have occurred around the site would not exceed the design basis wind speed of 102 mph given above.</p>	RAI 02.03.01-2 (Letter 048 Response per NND-09-0170)
5160	VCS	PT02	FSAR02	02.03.01.03.02	<p>The following text will be added to the FSAR in Subsection 2.3.1.3.2 on page 2.3-8:</p> <p>There were 124 tornadoes (see Table 2.3-227) that occurred in the surrounding (Saluda, Chester, Lancaster, Newberry, Lexington, Kershaw, Richland, Union and Fairfield) counties during the period from 1950-August 2003 (Reference 250). Based on the 124 tornadoes during the period of record of about 54 years the annual frequency would be about 2.3 tornadoes per year within approximately 50 miles of VCSNS. This period of record was selected to follow the period of record from NUREG/CR-4461, from which the design basis tornado characteristics given in Table 2.0-201 were selected.</p>	RAI 02.03.01-1 (Letter 048 Response per NND-09-170)
5165	VCS	PT02	FSAR02	02.03.01.05	<p>Revise the 6th from the last paragraph in FSAR Subsection 2.3.1.5 as follows (to add the last sentence):</p> <p>Record minimum temperatures observed in the VCSNS site area are presented in Table 2.3-203 and summarized in Subsection 2.3.2.2.4. Among the 14 NWS and Cooperative observer network stations listed in Table 2.3-201, the overall lowest temperature recorded was -5°F at a station (Chester 1NW) (References 214 and 222) located about 30 miles to the north of the site. The period of record for Chester 1NW is July 1948 June 2006.</p>	RAI 02.03.01-5 (Letter 048 Response per NND-09-0170)
5982	VCS	PT02	FSAR02	02.03.01.05	Revise FSAR Subsection 02.03.01.05 as shown in response to RAI 2.3.1-7 in SCE&G Letter NND-09-0274.	RAI 2.3.1-7 (Letter 064 Response per NND-09-0274)
6442	VCS	PT02	FSAR02	02.03.01.05	Add LMA of VCS DEP 2.0-2 to Subsection 2.3.1.5 adjacent to the 100-year return value of 87.3°F (wet bulb) temperature.	Departure introduced by the Response to RAI 02.03.01-7 by NND-09-0274

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						dated October 8, 2009 (ML092860136).
5166	VCS	PT02	FSAR02	02.03.01.07	<p>Add the following to the end of FSAR Subsection 2.3.1.7, "Climate Changes".</p> <p>General predictions on global and US climatic changes expected during the period of reactor operation are uncertain on the regional scale. Until higher resolution, more sophisticated, Global Climate Models (GCM's) can be developed it will be difficult to determine with certainty the characteristic changes that will occur in the site region. VCSNS is in a region where forecasts show little agreement between various modeling scenarios with respect to the relative changes in modeled climatic quantities (Reference 251). Many of the environmental quantities used for design purposes are not reported in the literature from GCM output. It is unclear, and may be speculative, as to how the general large scale trends in these climatic quantities would translate to design criteria in the site region, specifically with respect to the extreme values.</p> <p>The historic data record provides the climatic trends and severe natural phenomena that are included in the site characterization. A margin of safety is provided by the difference between the site characteristics and the OCO site parameters, used for design. This margin accounts for limitations to the accuracy, quantity and period of time in which the historical data have been accumulated, in addition to the potential for increases due to changes in the climate. However, there is considerable uncertainty from GCM output as to how this will impact the characteristic quantities of the site area.</p> <p>Future changes in the climate of the site region would potentially impact environmental conditions. The increases in the air temperature can be reasonably expected to remain below the OCD (Tier 2, Table 2.0-1) 0% exceedance dry bulb temperature of 115 °F, due to the margin of safety from the site value of 105.1 °F to 115°F (9.9°F) given that the best estimate of future temperature change is about 7.2°F (4°C) based on Table TS-6, Page 70 of the Technical Summary for Reference 251.</p> <p>GCM forecasts indicate more showery precipitation, leading to increased surface runoff, which would tend to provide more water available for recharge of the Monticello Reservoir and/or higher water levels in the Broad River. The site placement on top of the hills above the Broad River provides a margin of safety for the VCSNS plant by keeping it above the flood plain. Regional forecasts are extremely uncertain at this point. The hierarchies of GCM forecasts available have little certainty with respect to many forecast parameters. The current generation of models used to produce climate forecasts are not regional models. The current generation of climate models relies on extensive parameterizations for processes that are not well understood physically. Uncertainties of future model inputs (such as future greenhouse gas reductions), make the use of regionalized GCM output highly speculative at best.</p>	RAI 02.03.01-5 (Letter 048 Response per NND-09-0170)
5431	VCS	PT02	FSAR02	02.03.02.01	<p>The last sentence of the 3rd paragraph in Subsection 2.3.2.1 will be updated to reflect the 2-year period of record for onsite data as shown below (add "and 2008").</p> <p>The data from this monitoring program, used to support Units 2 and 3, include measurements taken over the 2007 and 2008 annual cycles.</p>	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5432	VCS	PT02	FSAR02	02.03.02.02.01	<p>The sentence of the 4th paragraph in Subsection 2.3.2.2.1 will be updated to reflect the 2-year period of record.</p> <p>Figures 2.3-202 through 2.3-206 present annual and seasonal wind rose plots (i.e., graphical distributions of the direction from which the wind is blowing and wind speeds for each of sixteen, 22.5° compass sectors centered on north, north-northeast, northeast, etc.) for the 10-meter level based on measurements over the 2007 and 2008 annual cycles.</p>	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5433	VCS	PT02	FSAR02	02.03.02.02.01	<p>The last sentence of the 6th paragraph in Subsection 2.3.2.2.1 will be deleted to remove reference to monthly wind roses, which are being excluded following guidance by NRC reviewers, such that the 6th paragraph will now read:</p> <p>Seasonally, winds from the southwest quadrant predominate during the spring and summer months (see Figures 2.3-204 and 2.3-205). This is also the case during the winter, although westerly winds prevail and the relative frequency of west-northwest winds during this season is greater (see Figure 2.3-203) because of increased cold frontal passages. Winds from the northeast quadrant predominate during the autumn months (see Figure 2.3-206).</p>	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5434	VCS	PT02	FSAR02	02.03.02.02.01	<p>The first sentence of the 7th paragraph in Subsection 2.3.2.2.1 will be updated</p>	RAI 2.3.3-1 (Letter 049 Repsonse per

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					refer to seasonal and annual wind roses, rather than monthly. The last sentence of this paragraph will be deleted, such that the 7th paragraph now reads as follows: Annual and seasonal wind rose plots based on measurements at the 60-meter level are shown in Figures 2.3-208 through 2.3-212. By comparison, wind direction distributions for the 60-meter level are fairly similar to the 10-meter level wind roses on composite annual and seasonal bases in terms of the predominant directional quadrants and variation over the course of the year. Prevailing winds differ between the two levels by one adjacent direction sector, generally veering (i.e., turning clockwise) with height as might be expected.	NND-09-0185)
5435	VCS	PT02	FSAR02	02.03.02.02.01	The first sentence of the 9th paragraph in Subsection 2.3.2.2.1 will be updated to reflect the period of record from 1-year to 2-years, as shown below: Table 2.3-206 summarizes seasonal and annual mean wind speeds based on measurements from the upper and lower levels of the meteorological tower operated in support of Units 2 and 3 from January 1, 2007 through December 31, 2008, and from wind instrumentation at the Columbia, South Carolina, NWS station based on a 49-year period of record (Reference 213). The elevation of the wind instruments at the Columbia NWS station is nominally 20 feet (approximately 6.1 meters) (Reference 213), comparable to the lower (10-meter) level measurements at the VCSNS site.	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5436	VCS	PT02	FSAR02	02.03.02.02.01	The first two sentences of the 10th paragraph in Subsection 2.3.2.2.1 will be updated to incorporate the second year of data to the period of record at the 10-meter and 60-meter levels, as shown below: Annually, mean wind speeds at the 10- and 60-meter levels are 2.4 and 4.4 meters per second, respectively, at the VCSNS site. The annual mean wind speed at Columbia (i.e., 3.0 meters/second) is slightly higher than the 10-meter level at the VCSNS site, differing by only 0.6 meters/second.	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5437	VCS	PT02	FSAR02	02.03.02.02.02	The first sentence of the 2nd paragraph in Subsection 2.3.2.2.2 will be updated to reflect the change in the period of record from 1-year to 2-years for the Units 2 and 3 tower, as shown below: Tables 2.3-207 and 2.3-208 present wind direction persistence/wind speed distributions based on measurements from the Units 2 and 3 monitoring program over a period of January 1, 2007 through December 31, 2008.	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5439	VCS	PT02	FSAR02	02.03.02.02.02	The 4th paragraph in Subsection 2.3.2.2.2 will be updated to reflect 2-year period of record at the 60-meter level from the Units 2 and 3 tower to read: At the 60-meter level, the longest persistence period is 24 hours and occurs for winds from the west directional sector (see Table 2.3-208) for wind speeds greater than or equal to 5 mph and 10 mph. For wind speeds greater than or equal to 15 mph, maximum persistence periods are limited to periods of 12 hours or less in north-northeast, south, west-southwest and west sectors. Wind speeds greater than or equal to 30 mph persisted for only two hours in the southeast, west-southwest and west sectors.	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
6479	VCS	PT02	FSAR02	02.03.02.02.02	The 3rd paragraph in Subsection 2.3.2.2.2 will be updated to reflect 2-year period of record at the 10-meter level from the Units 2 and 3 tower At the 10-meter level, the longest persistence period is less than or equal to 18 hours for winds from the north-northeast and northeast sectors. This duration appears only in the lowest wind speed group (i.e., for wind speeds greater than or equal to 5 mph). Persistence periods lasting for at least 12 hours are indicated for several directional sectors for wind speeds greater than or equal to 5 mph, including winds from the north, north-northeast, northeast, south, southwest, west-southwest, west, and northwest sectors. Wind speeds greater than or equal to 20 mph persisted for three periods of less than or equal to two hours, once in the south-southwest sector and twice in the west-southwest sector. There were no periods greater than or equal to 24 hours or persistent period of wind speeds greater than or equal to 25 mph.	RAI 2.3.3-1 (Letter 049 Response per NND-09-0331)
5440	VCS	PT02	FSAR02	02.03.02.02.03	The first sentence of the 3rd paragraph in Subsection 2.3.2.2.3 will be updated to reflect the period of record for stability data from 1-year to 2-years for the Units 2 and 3 tower to read: "Over the period of record from January 1, 2007 through December 31, 2008 for the monitoring program..."	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5441	VCS	PT02	FSAR02	02.03.02.02.03	The first sentence of the 4th paragraph in Subsection 2.3.2.2.3 will be updated to	RAI 2.3.3-1 (Letter 049 Repsonse per

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					reflect the change in the period of record in the stability data from 1-year to 2-years for the Units 2 and 3 tower to read: "The data indicate a predominance of neutral stability (Class D) and slightly stable (Class E) conditions throughout the year, ranging from about 63 to 73% of the time for these stability classes combined."	NND-09-0185)
5274	VCS	PT02	FSAR02	02.03.02.02.04	Add the following sentences to the end of the fourth paragraph of FSAR Subsection 2.3.2.2.4: Maximum and minimum values from COOP and first-order stations represent sub-hourly time periods. As defined in the AP1000 DCD, the maximum/minimum safety temperatures are defined as values excluding peaks of less than 2-hour duration. Because the maximum and minimum values are not sequential 2-hour duration hourly data they are not used for comparison to site characteristic values in Table 2.0-201.	RAI 02.03.02-1 (Letter 050 Response per NND-09-0186)
5357	VCS	PT02	FSAR02	02.03.02.02.07	Revise the last paragraph in Subsection 2.3.2.2.7 as follows: There is no enhancement of naturally occurring fog conditions due to operation of the mechanical draft cooling towers associated with Units 2 and 3 because of the buoyancy of the thermal plume.	RAI 02.03.02-3 (Letter 050 Response per NND-09-0213)
5356	VCS	PT02	FSAR02	02.03.02.04	The 4th paragraph in Subsection 2.3.2.4 of the FSAR will be revised as follows: Units 2 and 3 use mechanical draft cooling towers as a means of heat dissipation during normal operation (see Subsection 1.2.2). Potential meteorological effects due to the operation of these cooling towers could include enhanced ground-level fogging and icing, cloud shadowing and precipitation enhancement, and increased ground-level humidity. These effects and other potential related environmental impacts (e.g., solids deposition, visible plume formation, transport, and extent) have been evaluated. Salt deposition in the switchyards is expected to be low, with natural wash off removing accumulation before adversely impacting operations of the electrical equipment. Water deposition would occur at a rate that is several orders of magnitude below the measured precipitation rates at Columbia (FSAR Reference 213). The thermal plume would have a higher virtual temperature (temperature that represents both temperature and moisture contributions to buoyancy) than the maximum historically observed temperature value or ambient temperature. This would cause the plume to rise away from the control room HVAC intakes and switchyard electrical equipment due to buoyancy, except in high wind situations. In high winds, turbulence would cause enough mixing to prevent any adverse effects.	RAI 02.03.02-3S01 (Letter 050 Response per NND-09-0213)
5443	VCS	PT02	FSAR02	02.03.03.03	The 3rd paragraph in Subsection 2.3.3.3 will be corrected to change the period of record to 2-years of onsite data to read: "Two years of meteorological data collected from the VCSNS Units 2 and 3 meteorological tower have been provided. The first year of meteorological data for the period, January 2007 through December 2007, was used to establish a baseline for preparing the VCSNS Units 2 and 3 COL Application. On-site meteorological data collected from January 2008 through December 2008 provide the second year of data."	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
5442	VCS	PT02	FSAR02	02.03.03.03.03.02	The sentence of the 8th paragraph in Subsection 2.3.3.3.2 will be updated to reflect the 2-year period of record for onsite data to read: "For the Years 2007 and 2008, no data substitution has been required and the annualized data recovery rates for all parameters measured at the Units 2 and 3 meteorological tower will exceed 90%."	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
5463	VCS	PT02	FSAR02	02.03.03.03.05.01	Add the following after the first paragraph of Subsection 2.3.3.3.5.1: "It should be noted that temperature data collected by the U.S. Weather Service are normally measured near the ground level (about 1.5 - 2 meters above ground level). The temperature measurement as specified in Regulatory Guide 1.23 is measured at 10-meters and higher levels. The "system accuracy" shown in Table 2.3-216 is based on an observed temperature range of -0.6°F to 107.7°F. This range is about 5°F more than the minimum recorded and 3°F less than the maximum historical extremes measured in the site region. Strong lapse rates are a necessary condition under which extreme temperatures occur. Values that are less extreme than the historical limits would be measured at the 10-m or higher levels on the VCSNS Units 2 and 3 tower than would be observed close to the surface by temperature probes at U.S. Weather Service observation sites."	RAI 2.3.3-2 (Letter 049 Response per NND-09-0185)
5444	VCS	PT02	FSAR02	02.03.03.03.05.02	The last sentence of the 1st paragraph in Subsection 2.3.3.3.5.2 will be updated to reflect the 2-year period of record for onsite data and will be revised to read:	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)

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					"Overall, the data recovery rate meets the requirements of Regulatory Guide 1.23. Specifically, the annual data recovery rates for data period from January 2007 through December 2008 are greater than 90% for the three primary variables (i.e., wind speed, wind direction, and temperature difference)."	
5445	VCS	PT02	FSAR02	02.03.03.05.01	The 2nd sentence of the paragraph in Subsection 2.3.3.5.1 will be updated to reflect the 2-year period of record for on site data to read: "The data collection system of the new meteorological tower, dedicated to serve Units 2 and 3 conforms to Regulatory Guide 1.23. In support of the VCSNS Units 2 and 3 COL application, two years of available onsite data (i.e., 1/1/2007 - 12/31/2008) from the Units 2 and 3 tower was used to make the atmospheric dispersion estimates. The results of these dispersion estimates are reported in Subsections 2.3.4 and 2.3.5."	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5446	VCS	PT02	FSAR02	02.03.03.05.03	The 1st sentence of the 1st paragraph in Subsection 2.3.3.5.3 will be updated to reflect the 2-year period of record for onsite data to read: "Data are provided for the collection period from January 1, 2007, through December 31, 2008. Specifically, an electronic sequential, hour-by-hour listing of the data set, in the format specified in Appendix A of Regulatory Guide 1.23, is provided."	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5447	VCS	PT02	FSAR02	02.03.03.05.03	The 1st and 2nd sentences of the 2nd paragraph in Subsection 2.3.3.5.3 will be updated to reflect the 2-year period of record for onsite data to read: "Two-years of available onsite data were was used to calculate both the short-term and long-term atmospheric dispersion estimates presented in Subsections 2.3.4 and 2.3.5, in accordance with Regulatory Guide 1.206."	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5448	VCS	PT02	FSAR02	02.03.04.01	The 1st paragraph in Subsection 2.3.4.1 will be updated to reflect the 2-year period of record for onsite data to read: "To evaluate potential health effects for the AP1000 reactor design basis accidents, a hypothetical accident is postulated to predict upper-limit concentrations and doses that might occur in the event of a containment release to the atmosphere. Site-specific meteorological data, covering a period from January 1, 2007 through December 31, 2008, was used to quantitatively evaluate such a hypothetical accident at the site. Onsite data provide representative measurements of local dispersion conditions appropriate to Units 2 and 3, and the two-year period is considered to be reasonably representative of long-term conditions as discussed in Subsection 2.3.3."	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5449	VCS	PT02	FSAR02	02.03.04.02	The 2nd and 6th sentences (1st and 5th bullets) of the 10th paragraph in Subsection 2.3.4.2 will be updated to reflect the 2-year period of record for onsite data to read: "The PAVAN model input data is presented below: • Meteorological data: 2-years (January 1, 2007 to December 31, 2008) composite onsite joint frequency distributions of wind speed, wind direction, and atmospheric stability • Type of release: ground-level (a default height of 10 meters as suggested by Reference 230 was used) • Wind sensor height: 10 meters • Vertical temperature difference: (60 meters - 10 meters) • Number of wind speed categories: 11 (including calm) • Distances from release points along the PBAC to Dose Evaluation Periphery: 805 meters, for all downwind sectors • Distances from release point to LPZ boundary for all downwind sectors (see Table 2.3-219)"	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5450	VCS	PT02	FSAR02	02.03.04.02.01.01	The table after the 2nd paragraph in Subsection 2.3.4.2.1.1 will be updated to reflect the PAVAN model run with 2-years of onsite data as shown in response to RAI 2.3.3-1 in SCE&G Letter NND-09-0185.	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)
5451	VCS	PT02	FSAR02	02.03.04.02.01.01	The 1st sentence of the 3rd paragraph in Subsection 2.3.4.2.1.1 will be updated to reflect the PAVAN model run with 2-years of onsite data to read: "The results provided in Table 2.3-220 show that the maximum 0-2-hour X/Q value	RAI 2.3.3-1 (Letter 049 Repsonse per NND-09-0185)

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					(3.57E-04) determined by the PAVAN modeling analyses at the Dose Evaluation Periphery is bounded by the 0-2-hour DCD X/Q value of 5.1 E-04 as described in DCD Tier 1, Table 5.0-1 and DCD Tier 2, Table 2-1."	
5452	VCS	PT02	FSAR02	02.03.04.02.01.02	The 2nd and 3rd sentences of the 2nd paragraph in Subsection 2.3.4.2.1.2 will be updated to reflect the ARCON96 model run with 2-years of onsite data to read: "Control room X/Qs were estimated using the ARCON96 dispersion model as described in NUREG/CR-6331 (Reference 232) and considered receptor height, release height, release type, and building area. Two annual cycles (January 1, 2007 - December 31, 2008) of hourly meteorological data collected onsite were used as part of the input for the ARCON96 program. The two years of meteorological data have a data recovery rate of more than 90% and are representative of the site dispersion characteristics as described in Subsection 2.3.3.	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
5453	VCS	PT02	FSAR02	02.03.05.01	The last sentence of the 2nd paragraph in Subsection 2.3.5. 1 will be updated to correct the XOQDOQ model run with receptors of interest to read: "The NRC-sponsored XOQDOQ computer program (Reference 231) was used to estimate X/Q values due to routine releases of gaseous effluents to the atmosphere. The XOQDOQ computer code has the primary function of calculating annual average X/Q values and annual average relative deposition (D/Q) values at receptors of interest (e.g., the Dose Evaluation Periphery; the nearest: milk animal, residence, garden, and meat animal)."	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
5454	VCS	PT02	FSAR02	02.03.05.01	The 2nd sentence of the 4th paragraph in Subsection 2.3.5.1 will be updated to reflect the XOQDOQ model run with 2-years of onsite data to read: The following input data and assumptions have been used in the XOQDOQ modeling analysis: <ul style="list-style-type: none"> • Meteorological Data: 2-year (January 1, 2007 to December 31, 2008) composite onsite joint frequency distribution of wind speed, wind direction, and atmospheric stability • Type of release: Ground-level (a default height of 10 meters as suggested by Reference 231 was used) • Wind sensor height: 10 meters • Vertical temperature difference: (60 meters - 10 meters) • Number of wind speed categories: 11 (including calm). 	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
5976	VCS	PT02	FSAR02	02.03.05.01	Add the following text to the end of the 3rd paragraph in FSAR Subection 2.3.5.1: Regulatory Guide 1.111 states that a constant mean wind direction (straight-line trajectory) model may be used provided that the single station used is representative of the site region (within 50 miles of the site). Onsite meteorological data collected at the VCSNS Units 2 and 3 meteorological tower, under the guidance specified in Regulatory Guide 1.23 (Reference 238), is considered representative of the site region when compared to the National Weather Service first-order observations from Columbia, SC. The site region has relatively homogeneous topography. The wind roses from both stations are similar and the mean wind speeds for the onsite observations are somewhat lower than those at Columbia, leading to more conservative transport and diffusion estimates (as discussed in Subsections 2.3.1.6 and 2.3.2.2). The spatial homogeneity and similar wind characteristics are considered reasonable justification for the use of the constant mean direction (straight-line trajectory) model XOQDOQ.	RAI 2.3.5-1 (Letter 065 Response per NND-09-273)
5455	VCS	PT02	FSAR02	02.03.05.02	The 2nd paragraph in Subsection 2.3.5.2 will be updated to reflect the XOQDOQ model run with 2-years of onsite data to read: "The overall maximum annual average X/Q value is 1.7E-05 sec/m3 (no decay, undepleted) and occurs at Unit 3 due to the release from Unit 2. The maximum annual average X/Q values (along with the direction and distance of the receptor locations relative to the Units 2 and 3 site) for the other sensitive receptor types are: <ul style="list-style-type: none"> • 5.8E-06 sec/m3 for the Dose Evaluation Periphery occurring in the southeast sector at a distance of 0.5 mile • 8.7E-07 sec/m3 for the nearest residence occurring in the southeast sector at a distance of 1.68 miles • 4.6E-07 sec/m3 for the nearest meat animal occurring in the west-northwest sector at a distance of 1.74 miles • 1.7E-07 sec/m3 for the nearest milk animal in the northwest west sector at a receptor distance of 4.14 miles 	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					<ul style="list-style-type: none"> • 8.7E-07 sec/m3 for the nearest vegetable garden occurring in the southeast sector at a distance of 1.68 miles" 	
5977	VCS	PT02	FSAR02	02.03.05.02	<p>Revise the last paragraph of FSAR Subsection 2.3.5.2 as follows:</p> <p>Finally, Table 2.3-226 presents annual average X/Q values (for no decay and the default half-life radioactive decay periods of 2.26 and 8 days) and D/Q values at the XOQDOQ model's 22 standard radial distances (between 0.25 and 50 miles) and for the model's 10 distance-segment boundaries (between 0.5 and 50 miles downwind). The results along the southeast and east-northeast radials presented in Table 2.3-226 represent the highest X/Q and D/Q values, respectively, from among all the direction radials modeled.</p>	RAI 02.03.05-2 (Letter 065 Response per NND-09-0273)
5161	VCS	PT02	FSAR02	02.03.07	<p>Add the following reference to FSAR Section 2.3:</p> <p>250. Storm Events, National Climatic Data Center, web site: http://www4.ncdc.noaa.gov/cgi-win/wwwcgi.dll?wwevent-storms, Accessed May 2009</p>	RAI 02.03.01-1 (Letter 048 Response per NND-09-0170)
5167	VCS	PT02	FSAR02	02.03.07	<p>Add the following reference to FSAR Section 2.3:</p> <p>251. IPCC, 2007: Climate Change 2007: The Physical Science Basis. Contribution of Working Group I to the Fourth Assessment Report of the Intergovernmental Panel on Climate Change [Solomon, S., D. Qin, M. Manning, Z. Chen, M. Marquis, K.B. Averyt, M. Tignor and H.L. Miller (eds.)]. Cambridge University Press, Cambridge, United Kingdom and New York, NY, USA, 996 pp.</p>	RAI 02.03.01-5 (Letter 048 Response per NND-09-0170)
5456	VCS	PT02	FSAR02	02.03.07	<p>Reference 221 will be updated to reflect data available in the 2008 VCSNS RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT to read:</p> <p>"221. SCE&G, Radiological Environmental Operating Report, Virgil C. Summer Nuclear Station, for the Operating Period January 1, 2008 - December 31, 2008, SCE&G, April 2009."</p>	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
5457	VCS	PT02	FSAR02	02.03.T / T 2.3-206	<p>Table 2.3-206 will be updated to reflect the 2-year period of record for onsite data as indicated in response to NRC RAI 2.3.3-1 in SCE&G Letter NND-09-0185.</p>	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
5458	VCS	PT02	FSAR02	02.03.T / T 2.3-207-211, 217 and 220-226	<p>Tables 2.3-207-211, 217, and 220-226 will be completely replaced as a result of the incorporation of 2-years of onsite data as shown in response to NRC RAI 2.3.3-1 in SCE&G Letter NND-09-0185.</p>	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
5978	VCS	PT02	FSAR02	02.03.T / T 2.3-226	<p>Replace FSAR Table 2.3-226 with the X/Q and D/Q values based on the 2-year (2007-2008) composite meteorological data as shown in response to RAI 2.3.5-2 (12 sheets total).</p>	RAI 02.03.05-2 (Letter 065 Response per NND-09-0273)
5162	VCS	PT02	FSAR02	02.03.T / T 2.3-227	<p>Revise FSAR Section 2.3 to include Table shown in response to RAI 02.03.01-1 per SCE&G Letter NND-09-0170 as Table 2.3-227. Add the above table "Tornadoes That Occurred in Counties Surrounding VCSNS (Saluda, Chester, Lancaster, Newberry, Lexington, Kershaw, Richland, Union and Fairfield) During the Period From January 1950 Through August 2003" to the tables in FSAR Section 2.3 as Table 2.3-227.</p>	RAI 02.03.01-1 (Letter 048 Response per NND-09-0170)
5459	VCS	PT02	FSAR02	02.03F / F 2.3-202-206	<p>Figures 2.3-202-206 will be updated as a result of the incorporation of 2-years of onsite data as shown in response to NRC RAI 2.3.3-1 in SCE&G Letter NND-09-0185.</p>	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
5460	VCS	PT02	FSAR02	02.03F / F 2.3-207	<p>Figure 2.3-207 (12 sheets) will be deleted as shown in response to NRC RAI 2.3.3-1 in SCE&G Letter NND-09-0185.</p>	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
5462	VCS	PT02	FSAR02	02.03F / F 2.3-208-212	<p>Figures 2.3-208-212 will be updated as a result of the 2 years of meteorological data as shown in response to NRC RAI 2.3.3-1 in SCE&G Letter NND-09-0185.</p>	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
5461	VCS	PT02	FSAR02	02.03F / F 2.3-213	<p>Figure 2.3-213 (12 sheets) will be deleted as shown in response to NRC RAI 2.3.3-1 in SCE&G Letter NND-09-0185.</p>	RAI 2.3.3-1 (Letter 049 Response per NND-09-0185)
6303	VCS	PT02	FSAR02	02.03F / F 2.3-221	<p>Revise Figure 2.3-221 for Site Plan changes, relabel the "Intake Channel" to "Makeup Line, and correct the EAB.</p>	To incorporate Site Plan changes, correct the "Intake Channel" nonenclature, and correctly show the EAB.
5861	VCS	PT02	FSAR02	02.04.01.02.02	<p>In the 3rd sentence of the second paragraph of Section 2.4.1-2.2 change "272 ft." to "272.1 ft. NGVD29" for the western end of Parr Shoals Dam crest elevation.</p>	Editorial change necessary to be consistent with FSAR Subsection 2.4.3.3 and response to RAI 2.4.2-2

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						in SCE&G Letter NND-09-0272.
5979	VCS	PT02	FSAR02	02.04.13	Revise FSAR Subsections 2.4.13, 2.4.15. and associated tables and figures in accordance with response to RAI 2.4.13-12 in SCE&G Letter NND-09-0272.	RAI 2.4.13-12 (Letter 063 Response per NND-09-0272)
6314	VCS	PT02	FSAR02	02.04F / F.2.4-233	Revise Figure 2.4-233 to correct well labels.	To correct well labels that do not show or are reversed.
6312	VCS	PT02	FSAR02	02.04F / F.2.4-237 thru 244 and 252	Revise Figures 2.4-237 thru 244 and 252 for hydraulic contour line corrections.	To correct hydraulic contour lines.
5475	VCS	PT02	FSAR02	02.05.01	Add the following new reference to FSAR Section 2.5.1: "433. Garihan, J.M., Preddy, M.S., and Ranson, W.A., Summary of Mid-Mesozoic Brittle Faulting in the Inner Piedmont and Nearby Charlotte Belt of the Carolinas, in Carolina Geological Society Field Trip Guidebook - Studies of Inner Piedmont Geology with a Focus on the Columbus Promontory, p. 55-66; 1993."	RAI 2.5.1-55 (Letter 058 Response per NND-09-0212)
5464	VCS	PT02	FSAR02	02.05.01.01.02.03.02	Revise the third paragraph of COLA Part 2, FSAR, Chapter 2, Section 2.5.1.1.2.3.2, as follows: "Not all mapped faults in the site region display a recognizable magnetic signature. For example, the southern segment of the East Coast Fault System has no expression in the magnetic field and cuts across anomalies with wavelengths on the order of tens of kilometers without noticeably perturbing or affecting them. If the fault exists as mapped, then it has not accumulated sufficient displacement to juxtapose rocks of varying magnetic susceptibility, and thus does not produce an observable magnetic anomaly at the scale of Figures 2.5.1-205 and 2.5.1-206."	RAI 2.5.1-51 (Letter 058 Response per NND-09-0212)
5465	VCS	PT02	FSAR02	02.05.01.01.02.04	Revise COLA Part 2, FSAR, Chapter 2, Section 2.5.1.1.2.4, as follows: "2.5.1.1.2.4 Principal Regional Tectonic Structures Principal tectonic structures and features in the southeastern United States and within the 200-mile VCSNS site region are divided into four categories based on their age of formation or reactivation, and are shown in Figures 2.5.1-211 and 2.5.1-212. These categories include structures that were most active during Paleozoic, Mesozoic, Tertiary, or Quaternary time. Most of the Paleozoic and Mesozoic structures are regional in scale and are recognized on the basis of geologic and/or geophysical data. The Mesozoic rift basins and bounding faults show a high degree of parallelism with the structural grain of the Paleozoic Appalachian orogenic belt, which generally reflects reactivation of preexisting Paleozoic structures. Tertiary and Quaternary structures are generally more localized and may be related to reactivation of portions of older bedrock structures."	RAI 2.5.1-51 (Letter 058 Response per NND-09-0212)
5466	VCS	PT02	FSAR02	02.05.01.01.02.04.02	Revise COLA Part 2, FSAR Chapter 2, Section 2.5.1.1.2.4.2, the Mulberry Creek Fault paragraph as follows: "Mulberry Creek Fault The Mulberry Creek fault is located approximately 45 miles northwest of the VCSNS site (Figure 2.5.1-212). This sub-vertical fault contains silicified breccia, microbreccia, and cataclasite (Reference 403). The age of the Mulberry Creek fault is poorly constrained but, based on 180 +/- 3 Ma whole rock dates (Reference 429) from similar silicified breccias and cataclasites elsewhere in the Carolinas, West (Reference 403) suggests a Late Triassic to Early Jurassic age for the Mulberry Creek fault. As additional support for a Mesozoic age for the Mulberry Creek fault, Secor et al. (Reference 368) suggest that silicified breccias are characteristic of Mesozoic faults in the Piedmont and likely reflect hydrothermal activity indicative of a Mesozoic age. Moreover, Hatcher (Reference 430) indicates silicified cataclasite fault zones in the Piedmont formed coevally with Mesozoic (170-190 Ma) diabase dikes."	RAI 2.5.1-52 (Letter 058 Response per NND-09-0212)
5469	VCS	PT02	FSAR02	02.05.01.01.02.04.04	Revise COLA Part 2, FSAR, Chapter 2, Section 2.5.1.1.2.4.4, third paragraph, as follows: "Table 2.5.1-202 presents orientation and length information for these fourteen potential Quaternary features. The Charleston features (including the East Coast Fault System, the Cooke fault, the Helena-Banks fault zone, and the Charleston, Georgetown, and Bluffton paleoliquefaction features) are discussed in Subsection 2.5.1.1.3.2.1. The Eastern Tennessee Seismic Zone is discussed in Subsection 2.5.1.1.3.2.2. The remaining seven potential Quaternary features (namely, the Fall Lines of Weems (Reference 398), the Belair fault zone, the Pen Branch fault, the Cape Fear arch, the Hares Crossroads fault, the Stanley town Villa	RAI 2.5.1-53 (Letter 058 Response per NND-09-0212)

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					Heights faults, and the Pembroke faults) are discussed in detail below:"	
5472	VCS	PT02	FSAR02	02.05.01.01.03.02.01	<p>The following paragraph will be added as the last paragraph under the "Paleoliquefaction Features in Coastal South Carolina" bullet in Subsection 2.5.1.1.3.2.1:</p> <p>"In an abstract published after Talwani and Schaeffer's (Reference 386) compilation and interpretation of South Carolina liquefaction and paleoliquefaction data, Talwani et al. (Reference 432) describe a previously undiscovered paleoliquefaction feature near Fort Dorchester in the meizoseismal area of the 1886 Charleston, South Carolina, earthquake. Talwani et al. (Reference 432) describe this feature as a 1-m-wide sandblow at a depth of approximately 0.5 m below the ground surface. There are no radiocarbon or other quantitative age constraints on this feature. Talwani et al. (Reference 432), however, indicate a pre-1886 age for this sandblow, presumably on the basis of burial depth and degree of soil formation. Based on unspecified back calculation techniques, Talwani et al. (Reference 432) estimate a magnitude of -6.9 (magnitude scale unspecified) for the causative earthquake. Very little is known about the earthquake that produced Talwani et al.'s (Reference 432) paleoliquefaction feature. As such, the discovery of this paleoliquefaction feature does not provide any additional constraints on the timing, magnitude, or location of Charleston paleoearthquakes, beyond those presented in Talwani and Schaeffer (Reference 386)."</p>	RAI 2.5.1-54 (Letter 058 Response per NND-09-0212)
5474	VCS	PT02	FSAR02	02.05.01.02.04	<p>COLA Part 2, FSAR Chapter 2, Section 2.5.1.2.4, 3rd full paragraph, will be revised as follows:</p> <p>These minor shears and fractures are common to rocks throughout the Piedmont (References 364 and 433) and may be encountered within the foundation excavations for Units 2 and 3. During excavation for these units, detailed mapping of the foundation exposures will provide the ability to document the presence or absence of these minor bedrock shears, which typically cannot be recognized nor adequately characterized by surficial mapping or analysis of drill core."</p>	RAI 2.5.1-55 (Letter 058 Response per NND-09-0212)
5467	VCS	PT02	FSAR02	02.05.01.03	<p>Add the following new references to FSAR Section 2.5.1.3 as the next reference number in sequence:</p> <p>"429 Fullagar P.D. and Butler, J.R., Radiometric Dating in the Sauratown Mountains Area, North Carolina, in Geological Investigations of Piedmont and Triassic Rocks, Central North Carolina and Virginia, Carolina Geological Society Field Trip Guidebook. V. Price, P.A. Thayer, and W.A. Ranson (eds), p. 1-11, Virginia of Division Mineral Resources, 1980.</p> <p>430 Hatcher, R.D. Jr., Juxtaposed Mesozoic Diabase Dikes and Siliceous Catac/asite Fault Zones in the Carolinas and the Mechanics of Dike Emplacement, Geological Society of America, Southeastern Section Abstracts with Programs, v. 38, no. 3, p. 8, 2006."</p>	RAI 2.5.1-52 (Letter 058 Response per NND-09-0212)
5470	VCS	PT02	FSAR02	02.05.01.03	<p>Add the following new reference to FSAR Section 2.5.1.3:</p> <p>"431 Conley, J.F. and Toewe, E.C., Geology of the Martinsville West Quadrangle, Virginia, Virginia Division of Mineral Resources Report of Investigations 16, 1:24,000-scale, 1968."</p>	RAI 2.5.1-53 (Letter 058 Response per NND-09-0212)
5473	VCS	PT02	FSAR02	02.05.01.03	<p>Add the following new reference to FSAR Section 2.5.1.3:</p> <p>"432 Talwani, P., Dura-Gomez, I., Gassman, S., Hasek, M., and Chapman, A., Studies Related to the Discovery of a Prehistoric Sandblow in the Epicentral Area of the 1886 Charleston SC Earthquake: Trenching and Geotechnical Investigations. Program and Abstracts. Eastern Section of the Seismological Society of America. p. 50, 2008."</p>	RAI 2.5.1-54 (Letter 058 Response per NND-09-0212)
6650	VCS	PT02	FSAR02	02.05.01.T/T 2.5.1-202	In row 10 of Table 2.5.1-202 change Reference "Behrendt et al. (1983)" to "Behrendt et al. (1983)"	Editorial correction to RAI 2.5.1-53 (Letter 058 Response per NND-09-0212)

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5471	VCS	PT02	FSAR02	02.05.01.T / T 2.5.1-202	Add new table (Table 2.5.1-202) to FSAR Section 2.5.1 as shown in RAI 2.5.1-53 response in SCE&G Letter NND-09-0212.	RAI 2.5.1-53 (Letter 058 Response per NND-09-0212)
5468	VCS	PT02	FSAR02	02.05.01F / F 2.5.1-212	Revise FSAR Figure 2.5.1-212 to include the "diagonal line" symbol in the figure key, as shown in Attachment 1 to RAI 2.5.1-52 in SCE&G Letter NND-09-0212.	RAI 2.5.1-52 (Letter 058 Response per NND-09-0212)
6724	VCS	PT02	FSAR02	02.05.02.T / T 02.05.02-208	On sheet 2 of 2 of Table 2.5.2-208, add parenthesis for footnote c. Change from "(c" to "(c)"	Editorial. Changed to be consistent with other footnotes in that table.
5779	VCS	PT02	FSAR02	02.05.02.T / T 2.5.2-218	VCSNS FSAR Table 2.5.2-218 will be replaced with the Table shown in response to RAI 02.05.02-27 in the next VCSNS FSAR revision.	RAI 02.05.02-27 (Letter 060 Response per NND-09-0266)
5778	VCS	PT02	FSAR02	02.05.02F / F 2.5.2-239	VCSNS FSAR Figure 2.5.2-239 will be updated to reflect the revised mean magnitudes and distances shown in Figure RAI 2.5.2-27.8 in response to RAI 02.05.02-27 in the next VCSNS FSAR revision.	RAI 02.05.02-27 (Letter 060 Response per NND-09-0266)
5129	VCS	PT02	FSAR02	02.05.03.02	<p>FSAR Subsection 2.5.3.2, page 2.5.3-9, paragraph 2, will be revised as follows:</p> <ul style="list-style-type: none"> • Unnamed Fault near Parr, South Carolina. As part of an investigation performed for the Parr Hydroelectric Project, Dames & Moore (Reference 207) describes a postulated fault 3 miles south-southwest of the VCSNS site (Figures 2.5.1-224 and 2.5.1-225). Evidence for this fault includes slickensides observed in a boring at Parr Dam and four bedrock exposures described as "faulted rock", "dip-reversal across narrow disrupted zone", "discordance in foliation and beds", and "shear features." The postulated unnamed fault near Parr is based on a limited number of exposures and the assumption that these exposures all represent the same structure. With the exception of the outcrop in Parr and the boring on Parr Dam, the exposures are separated by distances greater than 1 mile. In addition, none of these exposures provide kinematic indicators and only one of the exposures yields information on orientation. Alternatively, the exposures observed by Dames & Moore (Reference 239) could represent individual local features of limited extent, similar to the minor faults and shears studied in the VC Summer Unit 1 exposure. More recent mapping of the area at 1:24,000 scale (References 363 and 364) does not include this postulated fault. For completeness, the inferred fault was conservatively included on Figures 2.5.1-224 and 2.5.1-225, even though the existence of a single fault connecting each of the Dames & Moore (Reference 239) exposures is highly speculative. This postulated fault, if it exists, is assigned a Paleozoic age, however, there are no data to constrain timing at any of the exposures. It is permissible that some could be as young as Mesozoic in age if they are similar to the bedrock shears mapped in the VC Summer Unit 1 excavation. The brief descriptions of the exposures by Dames & Moore (Reference 239) do not provide sufficient information to even classify the minor deformational features as having formed under ductile or brittle conditions. Field reconnaissance performed for Units 2 and 3 did not recognize evidence for faulting in the vicinity of Dames & Moore's (Reference 207) postulated fault near Parr, South Carolina (Reference 221). 	RAI 02.05.01-14 (Letter 033 Response per NND-09-0113)
5948	VCS	PT02	FSAR02	02.05.03.02	From the response to RAI Number 02.05.01-14; changes to FSAR Subsection 2.5.3.2, paragraph 2: References 363 and 364 should be clarified as being references from Section 2.5.1.3. "VC Summer" should be "V.C. Summer", two instances. The sentence with "however" should be "; however,".	Editorial changes to the response to RAI 02.05.01-14 as submitted by SCE&G Letter NND-09-0113, dated May 6, 2009.
6445	VCS	PT02	FSAR02	02.05.03.02	Add "Subsection 2.5.1.3" in front of it; Reference 239 [i.e. "(Subsection 2.5.1.3; Reference 239)"] in the bullet "Unnamed Fault near Parr, South Carolina", three instances.	Editorial clarification to RAI response 2.5.1-14 (SCE&G Letter NND-09-0113), Reference 239 is a Section 2.5.1 reference.
6633	VCS	PT02	FSAR02	02.05.03.02	Remove "a" in reference to Figure 2.5.1-202 on page 2.5.3-8, paragraph 1. Revise from: Figure 2.5.1-202a to read: Figure 2.5.1-202	Editorial error
5703	VCS	PT02	FSAR02	02.05.04.05.01	Revise the 2nd paragraph of FSAR Subsection 2.5.4.5.1 to read as follows: To obtain plant grade of about EL. 400 feet, the natural ground surface is leveled by excavating up to 28 feet of residuum and saprolite. The remainder of the residuum, saprolite and partially weathered rock beneath the power block is excavated down to top of rock using temporary retaining walls (soldier-pile and lagging with	RAI 2.5.4-37 (Letter 056 Response per NND-09-0191)

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					tiebacks) for support of the near-vertical excavations (as depicted on Figures 2.5.4-219 through 2.5.4-223). Temporary construction slopes are used in some limited areas beyond the retaining wall where the excavation is deeper than about 40 to 50 feet and for the access ramps and circulating water pipe trenches. The natural soil at the two units is excavated to the top of sound rock which varies from as high as EL. 384 feet to as deep as about EL. 312 feet. The temporary construction slopes are (typical) 2 horizontal to 1 vertical (2H:1V), benched about every 20 feet.	
5931	VCS	PT02	FSAR02	02.05.04.05.02.01	Revise FSAR Subsection 2.5.4.5.2.1 to read as follows: Excavation in the soils (Layers I and II) and any existing fills is achieved with conventional excavating equipment. Excavations will adhere to OSHA regulations (Reference 236) when less than 20 feet high. As noted in Subsection 2.5.4.5.1 temporary tied-back retaining walls are used to support the near-vertical excavations. Temporary construction slopes are used in some limited areas beyond the retaining wall where the excavation is deeper than about 40 to 50 feet and for the access ramps and circulating water pipe trenches. Since the saprolitic soils can be highly erosive, even temporary slopes cut into the saprolite are sealed and protected.	RAI 02.05.04-37 (Letter 056 Response per SCE&G Letter NND-09-0191)
5940	VCS	PT02	FSAR02	02.05.04.05.02.02	Revise the 1st paragraph of FSAR Subsection 2.5.4.5.2.2 to read as follows: Excavation in Layer III (PWR) rock is achieved using conventional earthmoving equipment. Temporary retaining walls are used to support the near-vertical excavations.	RAI 02.05.04-37 (Letter 056 Response per NND-09-0191)
5476	VCS	PT02	FSAR02	02.05.04.10.02	The 2nd paragraph of FSAR Subsection 2.5.4.10.2 will be revised in a future COLA update to read: "Ei = elastic modulus of layer i (high strain value used)"	RAI 2.5.4-35 (Letter 056 Response per NND-09-0191)
6315	VCS	PT02	FSAR02	02.05.04F / F 2.5.4-236	Revise Figure 2.5.4-236 to correct well labels.	To correct well labels that do not show or are reversed.
6313	VCS	PT02	FSAR02	02.05.04F / F 2.5.4-237	Revise Figure 2.5.4-237 for hydraulic contour line corrections.	To correct hydraulic contour lines.
5943	VCS	PT02	FSAR02	02.05.04F / F 2.5.4-219	Revise Figure 2.5.4-219 as indicated in response to RAI 02.05.04-37.	RAI 02.05.04-37 (Letter 056 Response per NND-09-0191)
5944	VCS	PT02	FSAR02	02.05.04F / F 2.5.4-220	Revise Figure 2.5.4-220 as indicated in response to RAI 02.05.04-37.	RAI 02.05.04-37 (Letter 056 Response per NND-09-0191)
5945	VCS	PT02	FSAR02	02.05.04F / F 2.5.4-221	Revise Figure 2.5.4-221 as indicated in response to RAI 02.05.04-37.	RAI 02.05.04-37 (Letter 056 Response per NND-09-0191)
5946	VCS	PT02	FSAR02	02.05.04F / F 2.5.4-222	Revise Figure 2.5.4-222 as indicated in response to RAI 02.05.04-37.	RAI 02.05.04-37 (Letter 056 Response per NND-09-0191)
5947	VCS	PT02	FSAR02	02.05.04F / F 2.5.4-223	Revise Figure 2.5.4-223 as indicated in response to RAI 02.05.04-37.	RAI 02.05.04-37 (Letter 056 Response per NND-09-0191)
PT02 - FSAR03						8 COLA Changes
6707	VCS	PT02	FSAR03	03.06.04.01	In the last sentence of the paragraph, capitalize Subsections in the sentence "The as-built reconciliation of the pipe break hazards analysis in accordance with the criteria outlined in DCD Subsections 3.6.1.3.2 and 3.6.2.5 will be completed prior to fuel load."	Conformance to R-COLA
6141	STD,VCS	PT02	FSAR03	03.07.04.02.01	4. COLA Part 2, FSAR Chapter 3, Subsection 3.7.4.2.1, will be revised to add the following sentence to the end of the existing FSAR added text: The trigger value is initially set at 0.01g.	SUPERSEDED/Incorporated into Qb 5991 BLN COL-SER-OI-Ch01 response to OI 01.04.01 item 4 Supplement 1 VCS-RAI 01-08 (Letter 55-Response)
6143	STD,VCS	PT02	FSAR03	03.09.03.04.04, item a.1	2. COLA, Part 2, Revision 1, FSAR Chapter 3, Subsection 3.9.3.4.4, item a.1, will be revised from: A list of snubbers on systems which experience sufficient thermal movement to measure cold to hot position is included as part of the testing program after the piping analysis has been completed.	RAI LTR 007 S2 response to RAI 03.09.06-003 item 2

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					To read: A list of snubbers on systems which experience sufficient thermal movement to measure cold to hot position is included in Table 3.9-201.	
6144	STD,VCS	PT02	FSAR03	03.09.03.04.04; item a.3	3. COLA, Part 2, Revision 1, FSAR Chapter 3, Subsection 3.9.3.4.4, item a.3, will be revised from: Safety-related components which utilize snubbers in their support systems will be identified in a future revision to the FSAR in table format that will include the following: <ul style="list-style-type: none"> • identification of systems and components • number of snubbers utilized in each system and on that component • snubber type(s) – (hydraulic or mechanical) – and name of supplier • constructed to ASME Code Section III, Subsection NF or other snubber use such as shock, vibration, or dual purpose • snubber use such as shock, vibration, or dual purpose • those snubbers identified as dual purpose or vibration arrestor type will include an indication if both snubber and component were evaluated To read: Safety-related snubbers are identified in Table 3.9-201, including the snubber identification and the associated system or component, e.g., line number. The snubbers on the list are hydraulic and constructed to ASME Section III, Subsection NF. The snubbers are used for shock loading only. None of the snubbers are dual purpose or vibration arrestor type snubbers.	RAI LTR 007 S2 response to RAI 03.09.06-003 item 3
6709	VCS	PT02	FSAR03	03.09.06	Change the sentence of the second revision to DCD Subsection 3.9.6 from: "Alternate means of performing these tests and inspections that provide equivalent demonstration may be developed in the inservice test program." to: "Alternate means of performing these tests and inspections that provide equivalent demonstration may be developed in the inservice test program as described in subsection 3.9.8." with "as described in subsection 3.9.8" shown as black strikeout text.	Conformance to the R-COLA
6145	STD,VCS	PT02	FSAR03	03.09.06:02.02	Add "(Reference 201)" {red, hyperlinked text} after "MPR-2524-A" at bottom of page 3.9-8 (new text to be inserted under the bulleted item titled, "Risk Ranking.")	Consistent method for identifying referenced documents
6146	STD,VCS	PT02	FSAR03	03.09.T / T3.9-201	4. COLA, Part 2, FSAR Chapter 3, Table 3.9-201 will be added to read: TABLE 3.9-201 SAFETY RELATED SNUBBERS	RAI LTR 007 S2 response to RAI 03.09.06-003 item 4
6149	STD,VCS	PT02	FSAR03	03.09.T / T3.9-201	COLA, Part 2, FSAR Chapter 3, Table 3.9-201 will be revised to add LMA of STD SUP 3.9-3.	Editorial
PT02 - FSAR05						3 COLA Changes
6147	STD,VCS	PT02	FSAR05	05.02.04.01	COLA Part 2, FSAR, Subsection 5.2.4.1, fifth paragraph, will be revised from: The inservice inspection program is augmented for reactor vessel top head inspections by use of the ASME Code Case N-729-1, "Alternative Examination Requirements for Pressurized-Water Reactor (PWR) Vessel Upper Heads With Nozzles Having Pressure-Retaining Partial-Penetration Welds," as modified by the NRC Staff position on the use of ASME Code Case N-729-1 shown in the proposed rulemaking dated April 5, 2007 (72 FR 16740). To read: The inservice inspection program is augmented for reactor vessel top head inspections by use of the ASME Code Case N-729-1, "Alternative Examination Requirements for Pressurized-Water Reactor (PWR) Vessel Upper Heads With Nozzles Having Pressure-Retaining Partial-Penetration Welds," as modified by the conditions specified in 10 CFR 50.55a(g)(6)(ii)(D).	COL-SER-OI-Ch05 response to 05.02-01
6150	STD,VCS	PT02	FSAR05	05.03.02:06	COLA Part 2, FSAR, Chapter 5, Subsection 5.3.2.6 (as revised by the Supplement 2 response to BLN-RAI-LTR-002) will be revised from: The type and quantity of test specimens exceed the minimum requirements of E185-82.	COL-SER-OI-Ch05 response to OI 05.03-01

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					To read: The type, quantity, and storage conditions (e.g., surveillance capsules backfilled with inert gas) of test specimens meet or exceed the minimum requirements of ASTM E-185.	
6436	VCS	PT02	FSAR05	05.04.07.01	Add the following new subsection 5.4.7.1 after FSAR Subsection 5.4.2.5 with LMA VCS DEP 2.0-2 to read: 5.4.7.1 Design Basis ----- Replace the second bulleted item in DCD Subsection 5.4.7.1.2.3 with the following: - The component cooling water system supply temperature to the normal residual heat removal system heat exchangers is based on an ambient design wet bulb temperature of no greater than 87.3°F (100 year return estimate of 2-hour duration). The 87.3°F value is assumed for normal conditions and transients that start at normal conditions. -----	RAI 2.3.1-7 (Letter 064 Response per NND-09-0274)
PT02 - FSAR06						8 COLA Changes
6151	STD,VCS	PT02	FSAR06	06.01.02.01.06	1. COLA Part 2, FSAR Chapter 6, Subsection 6.1.2.1.6 will be revised to add: Add the following after the third paragraph of the subsection titled "Service Level II Coatings" within DCD Subsection 6.1.2.1.6: Coating system inspection and monitoring requirements for the Service Level II coatings used inside containment will be performed in accordance with a program based on ASTM D5144 (Reference 201), "Standard Guide for Use of Protective Coating Standards in Nuclear Power Plants" and the guidance of ASTM D5163 (Reference 202), "Standard Guide for Establishing Procedures to Monitor the Performance of Coating Service Level I Coating Systems in an Operating Nuclear Power Plant." Any anomalies identified during coating monitoring are resolved in accordance with applicable quality requirements.	BLN-VOL-LTR-005 item 1
6152	STD,VCS	PT02	FSAR06	06.01.02.01.06	3. COLA Part 2, FSAR Chapter 6, Subsection 6.1.2.1.6, Service Level I and III Coatings, 1st paragraph, will be revised from: Regulatory Guide 1.54 and ASTM D5144 form the basis for the coating program. To read: Regulatory Guide 1.54 and ASTM D5144 (Reference 201) form the basis for the coating program.	BLN-VOL-LTR-005 item 3
6153	STD,VCS	PT02	FSAR06	06.01.02.01.06	4. COLA Part 2, FSAR Chapter 6, Subsection 6.1.2.1.6, Service Level I and III Coatings, 2nd paragraph, will be revised from: Coating system monitoring requirements for the containment coating systems are based on ASTM D5163, "Establishing Procedures to Monitor the Performance of Coating Service Level I Coating Systems in an Operating Nuclear Power Plant," and ASTM D7167, "Establishing Procedures to Monitor the Performance of Safety-Related Coating Service Level III Lining Systems in an Operating Nuclear Power Plant." To read: Coating system monitoring requirements for the containment coating systems are based on ASTM D5163 (Reference 202), "Standard Guide for Establishing Procedures to Monitor the Performance of Coating Service Level I Coating Systems in an Operating Nuclear Power Plant," and ASTM D7167 (Reference 203), "Standard Guide for Establishing Procedures to Monitor the Performance of Safety-Related Coating Service Level III Lining Systems in an Operating Nuclear Power Plant."	BLN-VOL-LTR-005 item 4
6530	VCS	PT02	FSAR06	06.01.03.02	Editorial correction to QB Change ID 6154 to capitalize Subsection in "DCD Subsection" as follows: The following information supplements the information provided in DCD Subsection 6.1.4.	Editorial Correction to BLN-VOL-LTR-005 item 2

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6154	STD,VCS	PT02	FSAR06	06.01.04	<p>2. COLA Part 2, FSAR Chapter 6, Add the following new subsection after subsection 6.1.3.2:</p> <p>The following information supplements the information provided in DCD subsection 6.1.4.</p> <p>6.1.4 References</p> <p>201. ASTM 5144-08, "Standard Guide for Use of Protective Coating Standards in Nuclear Power Plants"</p> <p>202. ASTM D5163-05a, "Standard Guide for Establishing Procedures to Monitor the Performance of Coating Service Level I Coating Systems in an Operating Nuclear Power Plant"</p> <p>203. ASTM D7167-05, "Standard Guide for Establishing Procedures to Monitor the Performance of Safety-Related Coating Service Level III Lining Systems in an Operating Nuclear Power Plant"</p>	BLN-VOL-LTR-005-item 2
6529	STD,VCS	PT02	FSAR06	06.01.04	<p>COLA Part 2, FSAR Chapter 6, revise Subsection 6.1.4, Reference 201 as added by Qb 6154, from:</p> <p>201. ASTM 5144-08, "Standard Guide for Use of Protective Coating Standards in Nuclear Power Plants"</p> <p>To read:</p> <p>201. ASTM D5144-08, "Standard Guide for Use of Protective Coating Standards in Nuclear Power Plants"</p>	Editorial revision to BLN-VOL-LTR-005 item 2
6155	STD,VCS	PT02	FSAR06	06.03.08.01	<p>4. COLA Part 2, FSAR Chapter 6, Subsection 6.3.8.1 will be revised from:</p> <p>6.3.8.1 Containment Cleanliness Program</p> <p>Insert the following information at the end of DCD Subsection 6.3.8.1:</p> <p>This COL Item is addressed below.</p> <p>Administrative procedures implement the containment cleanliness program. Implementation of the program minimizes the amount of debris that might be left in containment following refueling and maintenance outages. The program is consistent with the containment cleanliness program used in the evaluation discussed in DCD Subsection 6.3.8.2. The program includes as a minimum the following:</p> <p>Responsibilities</p> <p>The program defines the organizational responsibilities for implementing the program; defines personnel and material controls; and defines the inspection and reporting requirements.</p> <p>Implementation</p> <p>Containment Entry</p> <ul style="list-style-type: none"> • Controls to account for the quantities and types of materials introduced into the containment. • Limits on the types and quantities of materials, including scaffolding and tools, for a particular entry. • Defined prohibited materials and limits on quantities of materials that may generate hydrogen when exposed to the containment environment. • Personnel responsible for authorizing the types and quantities of material that may be introduced into containment, and approving the leaving of these materials unattended in containment. • Controls for loose items, such as keys and pens, which could be inadvertently left in containment. • Methods and controls for securing any items and materials left unattended in containment. <p>Containment Exit</p> <ul style="list-style-type: none"> • Controls for accounting for tools, equipment and other material left unattended in containment necessary for ongoing work. • Controls for accounting of the permanent removal of materials previously introduced into the containment. • Limits on the types and quantities of materials, including scaffolding and tools, 	<p>RAI LTR 030 S2 response to RAI 06.02.02-001 item 4</p> <p>SUPERSEDES RAI LTR 030 S1 response to RAI 06.02.02-001</p>

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					<p>that may be left unattended in containment during outages and power operation. Types of materials considered are tape, labels, plastic film, and paper and cloth products.</p> <ul style="list-style-type: none"> • Requirements and actions to be taken for unaccounted for material. • Requirements for final containment cleanliness inspections. • Record keeping requirements for entry/exit logs. <p>To read:</p> <p>6.3.8.1 Containment Cleanliness Program Insert the following information at the end of DCD Subsection 6.3.8.1:</p> <p>This COL Item is addressed below.</p> <p>Administrative procedures implement the containment cleanliness program. Implementation of the program minimizes the amount of debris left in containment following personnel entry and exits. The program is consistent with the containment cleanliness program limits discussed in DCD Subsection 6.3.8.1. The program includes, as a minimum, the following:</p> <p>Responsibilities</p> <p>The program defines the organizational responsibilities for implementing the program; defines personnel and material controls; and defines the inspection and reporting requirements.</p> <p>Implementation</p> <p>Containment Entry/Exit</p> <ul style="list-style-type: none"> • Controls to account for the quantities and types of materials introduced into the containment. • Limits on the types and quantities of materials, including scaffolding and tools, to ensure adequate accountability controls. This may be accomplished by the work management process. Storage of aluminum is prohibited without engineering authorization. Cardboard boxes or miscellaneous packing material is not brought into containment without approval. • If entries are made at power, prohibited materials and limits on quantities of materials that may generate hydrogen are established. • Controls for loose items, such as keys and pens, which could be inadvertently left in containment. • Methods and controls for securing any items and materials left unattended in containment. • Administrative controls for accounting for tools, equipment and other material are established. • Administrative controls for accounting of the permanent removal of materials previously introduced into the containment. • Limits on the types and quantities of materials, including scaffolding and tools, that may be left unattended in containment during outages and power operation. Types of materials considered are tape, labels, plastic film, and paper and cloth products. • Requirements and actions to be taken for unaccounted for material. • Requirements for final containment cleanliness inspections consistent with the design bases provided in DCD Subsection 6.3.8.1. • Record keeping requirements for entry/exit logs. <p>Housekeeping</p> <p>Housekeeping procedures require that work areas be maintained in a clean and orderly fashion during work activities and returned to original conditions (or better) upon completion of work.</p> <p>Sampling Program</p> <p>A sampling program is implemented consistent with NEI Guidance Report 04-07, "Pressurized Water Reactor Sump Performance Evaluation Methodology" as</p>	

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					supplemented by the NRC in the "Safety Evaluation by The Office of Nuclear Reactor Regulation Related to NRC Generic Letter 2004-02, Nuclear Energy Institute Guidance Report (Proposed Document Number NEI 04-07), "Pressurized Water Reactor Sump Performance Evaluation Methodology." Latent debris sampling is implemented before startup. The sampling is conducted after containment exit cleanliness inspections to provide reasonable assurance that the plant latent debris design bases are met. Sampling frequency and scope may be adjusted based on sampling results. Results are evaluated post-start up and any nonconforming results will be addressed in the Corrective Action Program.	
6158	STD,VCS	PT02	FSAR06	06.04.T / T6.4-201	COLA Part 2, FSAR Chapter 6, Table 6.4-201 - Onsite Chemicals, footnote a, will be revised to read(adds new last sentence): a) This Table supplements DCD Table 6.4-1. Quantities are by largest container content for the specified location per unit. To read: a) This Table supplements DCD Table 6.4-1. Quantities are by largest container content for the specified location per unit. Quantities and distances are bounding calculation values and not actual amounts and distances.	Clarification of table added by RAI LTR 137 response to 02.02.03-10, item 3
PT02 - FSAR08						5 COLA Changes
5358	VCS	PT02	FSAR08	08.02.01.02.02	Add the following to the end of the first paragraph in COLA Part 2, FSAR Subsection 8.2.1.2.2: "The protective devices controlling the switchyard breakers are set with consideration given to preserving the plant grid connection following a turbine trip."	RAI 08.02-9 (Letter 059 Response per NND-09-0217)
6311	VCS	PT02	FSAR08	08.02F / F 8.2-201	Revise Figure 8.2-201 for Site Plan changes.	To incorporate Site Plan changes. The construction access road is relocated and Transmission line routing from the Unit 2 & 3 switchyard to the Unit 1 switchyard is relocated. This figure should also be changed to show the transformer area changes.
6162	STD,VCS	PT02	FSAR08	08.03.01.01.02.04	COLA Part 2, FSAR Chapter 8, Section 8.3.1.1.2.4 will be revised from: Operation, inspection and maintenance procedures consider both the diesel generator manufacturer's recommendations and industry diesel working group recommendations. To read: Operation, inspection and maintenance (including preventive, corrective, and predictive maintenance) procedures consider both the diesel generator manufacturer's recommendations and industry diesel working group recommendations.	RAI LTR 149 response to RAI 08.03.01-002 SER with Open Items Confirmatory Item 8.3-1
6164	STD,VCS	PT02	FSAR08	08.03.01.04	COLA Part 2, FSAR Chapter 8, will be revised to add the following paragraph at the end of Subsection 8.3.1.4. 8.3.1.4 Inspection and Testing Add the following text at the end of DCD Subsection 8.3.1.4 Procedures are established for periodic verification of proper operation of the Onsite AC Power System capability for automatic and manual transfer from the preferred power supply to the maintenance power supply and return from the maintenance power supply to the preferred power supply.	RAI LTR 151 response to RAI 08.02-010(b) SER with Open Items Confirmatory Item 8.2-1
6165	STD,VCS	PT02	FSAR08	08.03.01.04 08.03.02.01.01	COLA Part 2, FSAR Chapter 8, Subsection 8.3.1.4, will be revised to add an LMA of STD SUP 8.3-4. Also revise LMA on 8.3.2.1.1.1 from STD SUP 8.3-1 to STD SUP 8.3-3. There is a previous use of SUP 8.3-1.	Revises BLN RAI LTR 151 response to RAI 08.02-010(b) SER with Open Items Confirmatory Item 8.2-1
PT02 - FSAR09						10 COLA Changes
5777	VCS	PT02	FSAR09	09.01.05	FSAR Section 9.1.5, 4th - 7th bullets - Correct missing links to appropriate referenced Subsections.	Editorial
6166	STD,VCS	PT02	FSAR09	09.01.06	1- COLA Part 2, FSAR Chapter 9, Subsection 9.1.6 will be revised from:	RAI LTR 165 in response to RAI 09.01.02-001 item 1

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					<p>STD COL 9.1-7 A spent fuel rack Metamic coupon monitoring program is to be implemented when the plant is placed into commercial operation. This program includes tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and or visual examination.</p> <p>To read:</p> <p>STD COL 9.1-7 A spent fuel rack Metamic coupon monitoring program is to be implemented when the plant is placed into commercial operation. This program includes tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and / or visual examination. The program will also include tests to monitor changes in physical properties of the absorber material, including neutron attenuation and thickness measurements.</p>	Duplicate of QB Item # 5035
6539	STD,VCS	PT02	FSAR09	09.01.06	<p>COLA Part 2, FSAR Chapter 9, Subsection 9.1.6 will be revised from:</p> <p>STD COL 9.1-7 A spent fuel rack Metamic coupon monitoring program is to be implemented when the plant is placed into commercial operation. This program includes tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and / or visual examination. The program will also include tests to monitor changes in physical properties of the absorber material, including neutron attenuation and thickness measurements.</p> <p>To read:</p> <p>STD COL 9.1-7 A spent fuel rack Metamic coupon monitoring program will be implemented when the plant is placed into commercial operation. This program will include tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and / or visual examination. The program will also include testing to monitor changes in physical properties of the absorber material, including neutron attenuation and thickness measurements.</p>	Editorial revision of RAI LTR 165 in response to RAI 09.01.02-001 item 1
6437	VCS	PT02	FSAR09	09.02.02.01	<p>Add the following new subsection following FSAR Subsection 9.2.1.2.2 with LMA VCS DEP 2.0-2 to read:</p> <p>9.2.2.1 Design Bases</p> <p>Replace the first bullet item in the criteria for normal operation in DCD Subsection 9.2.2.1.2.1 with the following information:</p> <p>The component cooling water supply temperature to plant components is not more than 100°F assuming a 100-year return estimate of 2-hour duration wet bulb temperature of 87.3°F for service water cooling (per Table 2.0-201).</p>	RAI 2.3.1-7 (Letter 064 Response per NND-09-0274)
5377	VCS	PT02	FSAR09	09.02.05.03	<p>COLA Part 2, FSAR Subsection 9.2.5.3 will be revised in a future update as indicated below.</p> <p>9.2.5.3 System Operation</p> <p>Add the following after the first paragraph of DCD Subsection 9.2.5.3 as follows:</p> <p>The site specific water source described above is considered to be the off-site water treatment facility. Filtered water described above is generated and disinfected at the off-site water treatment facility to provide a make-up source of drinking water to the Potable Water System (PWS). The location of the off-site water treatment facility is shown on FSAR Figure 1.1-202, "VCSNS Site Plan." This facility also provides a make-up source of filtered water to the Raw Water System (RWS) to support loads described in Section 9.2.11 of the FSAR. The facility is depicted on FSAR Figure 9.2-201, "Raw Water System Flow Diagram."</p> <p>Add the following after the second paragraph of DCD Subsection 9.2.5.3 as follows:</p> <p>The onsite water supply system described above is considered to be the off-site water treatment facility.</p> <p>Add the following after the fourth paragraph of DCD Subsection 9.2.5.3 as follows:</p> <p>The possibility for PWS to become contaminated radioactively does not exist. The Raw Water System (RWS) does</p>	RAI 09.02.04-2 (Letter 057 Response per NND-09-0211). This change supersedes the proposed change provided by RAI 09.02.04-1 (Letter 042 Response per NND-09-0111).

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					not have the potential to be a flowpath for radioactive fluids. Because RWS does not have the potential to be a flowpath for radioactive fluids, its filtered water make-up source from the off-site water treatment facility does not have the potential to be contaminated radioactively. Since the only association the make-up water to RWS has with the make-up water supply to PWS is the off-site water treatment facility, the possibility for PWS to become contaminated radioactively does not exist.	
6540	VCS	PT02	FSAR09	09.02.05.03	<p>COLA Part 2, FSAR Subsection 9.2.5.3 will be revised in a future update as indicated below.</p> <p>9.2.5.3 System Operation</p> <p>Add the following after the first paragraph of DCD Subsection 9.2.5.3 as follows:</p> <p>The site specific water source described above is considered to be the off-site water treatment facility. Filtered water described above is generated and disinfected at the off-site water treatment facility to provide a make-up source of drinking water to the Potable Water System (PWS). The location of the off-site water treatment facility is shown on FSAR Figure 1.1-202, "VCSNS Site Plan." This facility also provides a make-up source of filtered water to the Raw Water System (RWS) to support loads described in Subsection 9.2.11 of the FSAR. The facility is depicted on FSAR Figure 9.2-201, "Raw Water System Flow Diagram."</p> <p>Add the following after the second paragraph of DCD Subsection 9.2.5.3 as follows:</p> <p>The onsite water supply system described above is considered to be the off-site water treatment facility.</p> <p>Add the following after the fourth paragraph of DCD Subsection 9.2.5.3 as follows:</p> <p>The possibility for the PWS to become contaminated radioactively does not exist. The Raw Water System (RWS) does not have the potential to be a flowpath for radioactive fluids. Because RWS does not have the potential to be a flowpath for radioactive fluids, its filtered water make-up source from the off-site water treatment facility does not have the potential to be contaminated radioactively. Since the only association the make-up water to RWS has with the make-up water supply to PWS is the off-site water treatment facility, the possibility for PWS to become contaminated radioactively does not exist.</p>	Editorial Changes to RAI 09.02.04-2 (Letter 057 Response per NND-09-0211). This change supersedes the proposed change provided by RAI 09.02.04-1 (Letter 042 Response per NND-09-0111).
6004	VCS	PT02	FSAR09	09.02.11.02.02	<p>Add the following sentence at the end of the last paragraph in FSAR Subsection 9.2.11.2.2:</p> <p>The RWS piping is designed to ASME Standard B31.1.</p>	RAI 09.02.01-7 S1 (Letter 061 Responder per NND-09-0289)
6447	VCS	PT02	FSAR09	09.02.11.04	<p>Revise FSAR Section 9.2.11.4, 6th sentence from:</p> <p>When CWS is not operational, RWS provides dilution flow by an interconnection with the CWS blowdown line well upstream of the WLS connection.</p> <p>to read:</p> <p>When CWS is not operational, RWS provides dilution flow by an interconnection with the circulating water blowdown line well upstream of the WLS connection.</p>	Editorial clarification
6425	VCS	PT02	FSAR09	09.02F / F 9.2-201	Revise FSAR Figure 9.2-201 needs to remove secondary fire water storage tank connection to SWS cooling tower basins.	Conformance to DCD Rev. 17
6485	STD,VCS	PT02	FSAR09	09.05.04.05.02	Revise the reference to ASTM 4176 to read ASTM D4176	Editorial
PT02 - FSAR10						5 COLA Changes
6170	STD,VCS	PT02	FSAR10	10.01.03.01	<p>1. In Revision 1, the COLA Part 2, FSAR Subsection 10.1.3.1, last sentence of the paragraph was revised from:</p> <p>In addition, the FAC monitoring program considers the information of Generic Letter 89-08 and industry guidelines.</p> <p>To read:</p> <p>In addition, the FAC monitoring program considers the information of Generic Letter 89-08, EPRI NSAC-202L-R3,</p>	RAI LTR 18 S2 response to RAI 10.03.06-002 item 1 SER with Open Items Confirmatory Item 10.1-1

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					<p>and industry operating experience. The program requires a grid layout for obtaining consistent pipe thickness measurements when using Ultrasonic Test Techniques. The FAC program obtains actual thickness measurements for highly susceptible FAC locations for new lines as defined in EPRI NSAC-202L-R3. At a minimum, a Pass 1 analysis is used for low and highly susceptible FAC locations and a Pass 2 analysis is used for highly susceptible FAC locations when the Pass 1 analysis results warrant. To determine wear of piping and components where operating conditions are inconsistent or unknown, the guidance provided in EPRI NSAC-202L is used to determine wear rates.</p> <p>In a future revision, the above revised material will be further revised from:</p> <p>The FAC program obtains actual thickness measurements for highly susceptible FAC locations for new lines as defined in EPRI NSAC-202L-R3. At a minimum, a Pass 1 analysis is used for low and highly susceptible FAC locations and a Pass 2 analysis is used for highly susceptible FAC locations when the Pass 1 analysis results warrant.</p> <p>To read:</p> <p>The FAC program obtains actual thickness measurements for highly susceptible FAC locations for new lines as defined in EPRI NSAC-202L-R3 (Reference 201). At a minimum, a CHECWOKRS type Pass 1 analysis is used for low and highly susceptible FAC locations and a CHECWOKRS type Pass 2 analysis is used for highly susceptible FAC locations when Pass 1 analysis results warrant.</p>	
6173	STD,VCS	PT02	FSAR10	10.01.04	<p>2. COLA Part 2, FSAR Section 10.1, will be further revised to include a new Subsection 10.1.4, References, following Subsection 10.1.3:</p> <p>Add the following after DCD Subsection 10.1.3:</p> <p>10.1.4 References</p> <p>201: EPRI NSAC-202L-R3, Recommendations for an Effective Flow-Accelerated Corrosion Program (NSAC-202L-R3); Electric Power Research Institute (EPRI) Technical Report 1011838, Palo Alto, CA, 2006.</p>	BLN-RAI LTR 18 S2 response to RAI 10.03.06-002 item 2 SER- with Open Items Confirmatory Item 10.1-1
6174	STD,VCS	PT02	FSAR10	10.02.02	COLA Part 2, FSAR Chapter 10, Subsection 10.2.2, will be revised to add a separator bar between STD SUP 10.2-1 text and the STD SUP 10.2-4 text.	Consistency of separating LMA associated text.
6175	STD,VCS	PT02	FSAR10	10.03.02.02.01	COLA Part 2, FSAR Chapter 10, Subsection 10.3.2.2.1, will be revised to remove the "will" from the sentence beginning "Operations and maintenance procedures will include precautions,..."	Consistency - FSAR text is present tense
6177	STD,VCS	PT02	FSAR10	10.04.07.02.01	COLA Part 2, FSAR Chapter 10, Subsection 10.4.7.2.1, will be revised to remove the "will" from the sentence beginning "Operations and maintenance procedures will include precautions,..."	Consistency - FSAR text is present tense
PT02 - FSAR11						9 COLA Changes
6729	VCS	PT02	FSAR11	11.02	Incorporate changes described in SCE&G letter NND-10-0001.	To be consistent with the most recent liquid effluent dose analysis, which was revised to address NRC review comments for the Environmental Report.
6296	VCS	PT02	FSAR11	11.02.03.03	<p>Change first two sentences of 2nd paragraph from:</p> <p>There is no dilution of the effluent discharge prior to entering the Broad River at the Parr Reservoir. The effluent discharge is diluted by the flow rate of the Broad River.</p> <p>to:</p> <p>In calculating the effluent doses, it is assumed that there is no dilution of the effluent discharge prior to entering the Broad River at the Parr Reservoir. Neglecting the blowdown flow rate of 6000 gpm, the effluent discharge is assumed to be directly diluted by the flow rate of the Broad River.</p>	Editorial - Clarification of assumption in dose evaluation.
6480	VCS	PT02	FSAR11	11.03	Revise FSAR 11.3 in accordance with SCE&G response to NRC RAI 11.03-3 in SCE&G Letter NND-09-0313.	RAI- 11.3-3 (Letter 068 Response per NND-09-0313)

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
5724	VCS	PT02	FSAR11	11.03.03	COLA Part 2, FSAR Chapter 11, Section 11.3 will be revised to add the following subsection with left margin annotation (LMA) of STD SUP 11.3-2 and appropriate separator lines: ----- 11.3.3 Radioactive Releases ----- Add the following new paragraph at the end of DCD Subsection 11.3.3: There are no gaseous effluent site interface parameters outside of the Westinghouse scope. -----	VCS RAI 01-04 (Letter 055 Response per NND-09-0233) based on BLN RAI Letter 0156 response to RAI 01-014 item 10.
5865	VCS	PT02	FSAR11	11.03.03.04.03	Add a horizontal separator line between FSAR Subsections 11.3.3.4.3 and 11.3.3.4.4.	Editorial
6181	STD,VCS	PT02	FSAR11	11.04.06	1. COLA Part 2, FSAR Chapter 11, Subsection 11.4.6, will be revised from: This section adopts NEI 07-10 (Reference 201) which is currently under review by the NRC staff. The Process Control Program (PCP) describes the administrative and operational controls used for the solidification of liquid or wet solid waste and the dewatering of wet solid waste. To read: A Process Control Program (PCP) is developed and implemented in accordance with the recommendations and guidance of NEI 07-10A (Reference 201). The PCP describes the administrative and operational controls used for the solidification of liquid or wet solid waste and the dewatering of wet solid waste.	BLN-VOL-LTR-003 response to NEI 07-10 item 1 SER with Open Items Confirmatory Item 11.4-1
6182	STD,VCS	PT02	FSAR11	11.04.07	2. COLA Part 2, FSAR Chapter 11, Subsection 11.4.7, will be revised from: 201: NEI 07-10, "Generic FSAR Template Guidance for Process Control Program (PCP) Description," Revision 2, February 2008. To read: 201: NEI 07-10A, "Generic FSAR Template Guidance for Process Control Program (PCP)," Revision 0, March 2009.	BLN-VOL-LTR-003 response to NEI 07-10 item 2 SER with Open Items Confirmatory Item 11.4-1
6183	STD,VCS	PT02	FSAR11	11.05.07	1. COLA Part 2, FSAR Chapter 11, Subsection 11.5.7, will be revised from: This section adopts NEI 07-09 (Reference 202), which is currently under review by NRC staff. The ODCM program description contains the methodology and parameters used for calculating doses resulting from liquid and gaseous effluents. The ODCM program description addresses operational setpoints, including planned discharge rates, for radiation monitors and monitoring programs (process and effluent monitoring and environmental monitoring) for the control and assessment of the release of radioactive material to the environment. The ODCM program description provides the limitations on operation of the radwaste systems, including functional capability of monitoring instruments, concentrations of effluents, sampling, analysis, 10 CFR Part 50, Appendix I dose and dose commitments, and reporting. The ODCM program description will be finalized prior to fuel load with site-specific information. To read: An Offsite Dose Calculation Manual (ODCM) is developed and implemented in accordance with the recommendations and guidance of NEI 07-09A (Reference 202). The ODCM contains the methodology and parameters used for calculating doses resulting from liquid and gaseous effluents. The ODCM addresses operational setpoints, including planned discharge rates, for radiation monitors and monitoring programs (process and effluent monitoring and environmental monitoring) for the control and assessment of the release of radioactive material to the environment. The ODCM provides the limitations on operation of the radwaste systems, including functional capability of monitoring instruments, concentrations of effluents, sampling, analysis, 10 CFR Part 50, Appendix I dose and dose commitments, and reporting. The ODCM will be finalized prior to fuel load with site-specific information.	BLN-VOL-LTR-003 response to NEI 07-09 item 1 SER with Open Items Confirmatory Item 11.5-1
6184	STD,VCS	PT02	FSAR11	11.05.08	2. COLA Part 2, FSAR Chapter 11, Subsection 11.5.8, will be revised from: 202: NEI 07-09, "Generic FSAR Template Guidance for Offsite Dose Calculation Manual (ODCM) Program Description," Revision 1, February 2008. To read: 202: NEI 07-09A, "Generic FSAR Template Guidance for Offsite Dose Calculation Manual (ODCM) Program	BLN-VOL-LTR-003 response to NEI 07-09 item 2 SER with Open Items Confirmatory Item 11.5-1

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					Description, Revision 0, March 2009	
PT02 - FSAR12					18 COLA Changes	
6185	STD,VCS	PT02	FSAR12	12.01	<p>5. COLA Part 2, FSAR Chapter 12, Section 12.1, will be revised from:</p> <p>This section incorporates by reference NEI 07-08, Generic FSAR Template Guidance for Ensuring That Occupational Radiation Exposures Are As Low As Is Reasonably Achievable (ALARA), Revision 3, which is currently under review by the NRC staff. See Table 1.6-201. ALARA practices are developed in a phased milestone approach as part of the procedures necessary to support the Radiation Protection Program. Table 13.4-201 describes the major milestones for ALARA procedures development and implementation.</p> <p>To read:</p> <p>This section incorporates by reference NEI 07-08A, Generic FSAR Template Guidance for Ensuring That Occupational Radiation Exposures Are As Low As Is Reasonably Achievable (ALARA), Revision 0. See Table 1.6-201. ALARA practices are developed in a phased milestone approach as part of the procedures necessary to support the Radiation Protection Program. Table 13.4-201 describes the major milestones for ALARA procedures development and implementation.</p>	COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 5 SNC Letter ND-09-1770
6186	STD,VCS	PT02	FSAR12	12.01	<p>6. COL Part 2 FSAR Chapter 12, Section 12.1, will be revised to add new text (with an LMA of STD COL 12.1-1) that reads</p> <p>Revise the last sentence of NEI 07-08A Subsection 12.1.2 to read:</p> <p>ALARA procedures are established, implemented, maintained and reviewed consistent with 10 CFR 20.1101 and the quality assurance criteria described in Part III of the Quality Assurance Program Description, which is discussed in Section 17.5.</p>	COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 6 SNC Letter ND-09-1770
6187	STD,VCS	PT02	FSAR12	12.01.03	<p>7. COL Part 2 FSAR Chapter 12, Section 12.1.3, will be revised from:</p> <p>This COL item is addressed in NEI 07-08 and Appendix 12AA.</p> <p>To read:</p> <p>This COL item is addressed in NEI 07-08A and Appendix 12AA.</p>	COL-SER-OI-Ch12 S1 response to OI 12.01-001 item 7 SNC Letter ND-09-1770
6188	STD,VCS	PT02	FSAR12	12.03.01.02	<p>COLA Part 2, FSAR Chapter 12, Subsection 12.3.1.2 will be added with an LMA of VCS DEP 18.8-1; to read:</p> <p>12.3.1.2 Radiation Zoning and Access Control</p> <p>Add the following information at the end of the second paragraph of DCD Subsection 12.3.1.2.</p> <p>Figure 12.3-201, Figure 12.3-202, and Figure 12.3-203 replace DCD Figure 12.3-1 (Sheet 11), DCD Figure 12.3-2 (Sheet 11), and DCD Figure 12.3-3 (Sheet 11), respectively, to reflect the relocation of the Operations Support Center</p>	Consistency - Revised figures were not identified in the FSAR text
6189	STD,VCS	PT02	FSAR12	12.03.05.01	<p>7. COLA Part 2, FSAR Chapter 12, Section 12.3.5.1, will be revised from:</p> <p>This COL item is addressed in Appendix 12AA.</p> <p>To read:</p> <p>This COL item is addressed in Subsection 12.5.4 and Appendix 12AA.</p>	COL-SER-CI-Ch12 response to CI 12.01.01 item 7 SNC Letter #ND-09-1529
6389	VCS	PT02	FSAR12	12.04.01:09:03	<p>4th paragraph - 2nd sentence - Change from:</p> <p>The atmospheric dispersion calculation used the guidance provided in Regulatory Guide 1.111, meteorological data for the years 2003 to 2006, and downwind distances to the construction worker locations.</p> <p>to read:</p>	RAI 12-03-12-04-3 (Letter 072 Response per NND-09-0354)

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					The atmospheric dispersion calculation used the guidance provided in Regulatory Guide 1.111, meteorological data for the years 2007 and 2008, and downwind distances to the construction worker locations.	
6390	VCS	PT02	FSAR12	12.04.01.09.04	3rd Paragraph, 5th sentence - Change from: For an occupational year, dose at the Unit 3 construction area is 1.2 mrem TEDE. to read: For an occupational year, dose at the Unit 3 construction area is 1.3 mrem TEDE.	RAI 12.03-12.04-3 (Letter 072 Response per NND-09-0354)
6391	VCS	PT02	FSAR12	12.04.01.09.05	1st Paragraph, 3rd Sentence - Change from: The total worker collective maximum annual dose is 4.3 person-rem. to read: The total worker collective maximum annual dose is 4.6 person-rem.	RAI 12.03-12.04-3 (Letter 072 Response per NND-09-0354)
6190	STD,VCS	PT02	FSAR12	12.04.01.09.06	COLA Part 2, FSAR Chapter 12, Subsection 12.4.1.9, will be revised to include the following new subsection (12.4.1.9.6 for VCS) at the end of the section with a Left Margin Annotation (LMA) of STD SUP 12.4-1: STD SUP 12.4-1 12.4.1.9.x Operating Unit Radiological Surveys The operating unit conducts radiological surveys in the unrestricted and controlled area and radiological surveys for radioactive materials in effluents discharged to unrestricted and controlled areas in implementing 10 CFR 20.1302. These surveys demonstrate compliance with the dose limits of 10 CFR 20.1301 for construction workers.	COL-SER-OI-Ch12 response to OI 12.04-01
6392	VCS	PT02	FSAR12	12.04.T/ T12.4-201	Change from: Annual total effective dose equivalent 100 mrem 1.2 mrem Maximum dose in any hour 2 mrem 6.0 E-04 mrem to read: Annual total effective dose equivalent 100 mrem 1.3 mrem Maximum dose in any hour 2 mrem 6.5 E-04 mrem	RAI 12.03-12.04-3 (Letter 072 Response per NND-09-0354)
6191	STD,VCS	PT02	FSAR12	12.05	8. COLA Part 2, FSAR Chapter 12, Section 12.5, will be revised to add new text after Section 12.5.3.2 (with an LMA of STD COL 12.3-1) that reads: 12.5.4 Controlling Access and Stay Time Add the following text to the end of DCD Subsection 12.5.4. STD COL 12.3-1 A closed circuit television system may be installed in high radiation areas to allow remote monitoring of individuals entering high radiation areas by personnel qualified in radiation protection procedures.	COL-SER-CI-Ch12 response to CI 12.01.01 item 8 SNC Letter #ND-09-1529
6193	STD,VCS	PT02	FSAR12	12AA	9. COLA Part 2, FSAR Chapter 12, Appendix 12AA will be revised in its entirety as shown in Attachment 12.01-01A of response to confirmatory item COL-SER-CI-CH12 (SNC Letter #ND-09-1529). The changes include those provided in the response to BLN-RAI-LTR-142 (NRC/RAI Number 01-11, ADAMS ML083510576).	COL-SER-CI-Ch12 response to CI 12.01.01 item 9 SNC Letter #ND-09-1529
6194	STD,VCS	PT02	FSAR12	12AA.05.04.08	1. COLA Part 2, FSAR Chapter 12, Appendix 12AA, text after the last bullet of NEI 07-03 Subsection 12.5.4.8 will be revised from: This subsection adopts NEI 08-08 (Reference 201), which is currently under review by the NRC staff, for discussion of compliance with 10 CFR 20.1406. To read: This subsection adopts NEI 08-08A (Reference 201), for a description of the operational and programmatic elements and controls that minimize contamination of the facility, site, and the environment, to meet the requirements of 10 CFR 20.1406.	COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 1 SNC Letter #ND-09-1770
6197	STD,VCS	PT02	FSAR12	12AA.05.04.14	2. COL Part 2 FSAR Chapter 12, Appendix 12AA 5.4.14 last paragraph, will be revised from:	COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 2

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					This subsection adopts NEI 08-08 (Reference 201), which is currently under review by the NRC staff, for the Groundwater Monitoring Program description. To read: This subsection adopts NEI 08-08A (Reference 201) for the Groundwater Monitoring Program description.	SNC Letter #ND-09-1770
6198	STD,VCS	PT02	FSAR12	12AA.05.04.15	3. COL Part 2 FSAR Chapter 12, Appendix 12AA.5.4.15, will be revised from: This subsection adopts NEI 08-08 (Reference 201), which is currently under review by the NRC staff, for discussion of record keeping practices important to decommissioning. To read: This subsection adopts NEI 08-08A (Reference 201) for discussion of record keeping practices important to decommissioning.	COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 3 SNC Letter #ND-09-1770
6195	STD,VCS	PT02	FSAR12	12AA.Refs	4. COLA Part 2 FSAR Chapter 12, Appendix 12AA reference to NEI 07-03 References Section will be revised from: 201. NEI 08-08, Generic FSAR Template Guidance for Life Cycle Minimization of Contamination, Revision 0. To read: 201. NEI 08-08A, Generic FSAR Template Guidance for Life Cycle Minimization of Contamination, Revision 0, October 2009.	COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 4 SNC Letter #ND-09-1770
6531	STD,VCS	PT02	FSAR12	12AA.Refs	4. COL Part 2 FSAR Chapter 12, Appendix 12AA reference to NEI 07-03 References will be revised from: 201. NEI 08-08A, Generic FSAR Template Guidance for Life Cycle Minimization of Contamination, Revision 0, October 2009. To read: 201. NEI 08-08A, Generic FSAR Template Guidance for Life Cycle Minimization of Contamination, Revision 0, October 2009 (ML093220445).	Editorial revision to COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 4 SNC Letter #ND-09-1770 to add ML number to reference
6196	STD,VCS	PT02	FSAR12	12AA.T / T12AA-201	10. COLA Part 2 FSAR Chapter 12, Appendix 12AA, add new Table 12AA-201 (with an LMA of STD COL 12.3-1) as shown in Attachment 12.01-01B of response to confirmatory item COL-SER-CI-CH12 (SNC Letter #ND-09-1529).	COL-SER-CI-Ch12 response to CI 12.01-01 item 10 SNC Letter #ND-09-1529
PT02 - FSAR13						21 COLA Changes
6401	VCS	PT02	FSAR13	13.01.01.01	Revise FSAR Subsection 13.1.1.1 to read: The President and Chief Operating Officer has overall responsibility for functions involving design, construction, and operation. Line responsibilities for those functions are assigned to the Executive Vice President-Generation (EVPG) through the Senior Vice President Nuclear Operations (SVPNO) via the Vice President, New Nuclear Deployment (VPNND) (Figure 13AA-201) for the design and construction of new nuclear plants. At the appropriate time after construction, direct control of nuclear plant operation is assigned to the site executive in charge of VCSNS, the Vice President Nuclear Operations, (VPNO), and his direct reports. The first priority and responsibility of each member of the nuclear staff throughout the life of the plant is nuclear safety. Decision-making for station activities is performed in a conservative manner with expectations of this core value regularly communicated to appropriate personnel by management interface, training, and station directives.	Organizational Changes
6402	VCS	PT02	FSAR13	13.01.01.02	Revise FSAR Subsection 13.1.1.2, first paragraph, 1st sentence to read (change GMNND to VPNND): Before beginning preoperational testing, the VPNND and the VPNO will establish the organization of managers, functional managers, supervisors, and staff sufficient to perform required functions for support of safe plant operation.	Organizational Changes
6403	VCS	PT02	FSAR13	13.01.01.03.01.01	Revise FSAR Subsection 13.1.1.3.1.1 to read (change SVPG to EVPG):	Organizational Changes

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					The Chief Executive Officer (CEO) has the ultimate responsibility for the safe and reliable operation of each nuclear unit owned and/or operated by SCE&G. The CEO is responsible for the overall direction and management of the corporation, and the execution of the company policies, activities, and affairs. The CEO is assisted by the EVPG (also the chief nuclear officer), and other executive staff in the nuclear division of the corporation.	
6404	VCS	PT02	FSAR13	13.01.01.03.01.03	<p>Revise FSAR Subsection 13.1.1.3.1.3 to read:</p> <p>13.1.1.3.1.3 Executive Vice President - Generation (EVPG)/Chief Nuclear Officer</p> <p>The EVPG reports to the CEO through the COO. The EVPG/Chief Nuclear Officer (CNO) is responsible for electric generation and overall plant nuclear safety and takes the measures needed to provide acceptable performance of the staff in operating, maintaining, and providing technical support to the nuclear site. The EVPG/CNO delegates authority and responsibility for the operation and support of the site through the SVPNO. It is the responsibility of the EVPG/CNO to provide guidance and direction such that safety-related activities, including engineering, construction, operations, operations support, maintenance, and planning, are performed following the guidelines of the quality assurance program.</p> <p>The EVPG/CNO is responsible for new nuclear plant licensing, design, and construction through the SVPNO.</p>	Organizational Changes
6406	VCS	PT02	FSAR13	13.01.01.03.01.04	<p>Renumber Subsection 13.1.1.3.1.4 to Subsection 13.1.1.3.1.5 and revise as follows (change SVPG/CNO to SVPNO in the first sentence):</p> <p>13.1.1.3.1.5 Vice President Nuclear Operations (VPNO)</p> <p>The VPNO reports to the SVPNO. The VPNO is directly responsible for management and direction of activities associated with the efficient, safe, and reliable operation of the nuclear station. The VPNO is assisted in management and technical support activities by the GMNPO, GMES, GMNSS, and the General Manager Organizational Effectiveness (GMOE). The VPNO is responsible for the site fire protection program through the engineer in charge of fire protection. See Subsection 13.1.1.2.10.</p> <p>Insert the following information as Subsection 13.1.1.3.1.4:</p> <p>13.1.1.3.1.4 Senior Vice President Nuclear Operations (SVPNO)</p> <p>The SVPNO reports to the EVPG. The SVPNO is responsible for the safe operation of all current nuclear plant operations along with the design, licensing, and construction of new nuclear plants. The SVPNO delegates authority and responsibility for the operation and support of the operating nuclear plants through the VPNO. The SVPNO is responsible for new nuclear plant licensing, design, and construction via the VPNNND who maintains control of nuclear plant construction through construction completion.</p>	Organizational Changes
6407	VCS	PT02	FSAR13	13.01.T / T 13.1-201	<p>Revise Table 13.1-201, Nuclear Plant Position Senior Vice President/CNO to read Executive Vice President, Generation/CNO. Add position to executive, nuclear generation, Nuclear Plant Positions (Site Specific) column above Vice President, Nuclear Operations to read "Senior Vice President, Nuclear Operations," expected positions column "1*"</p>	Organizational Changes
6648	VCS	PT02	FSAR13	13.01F / F 13.1-201	<p>In the figure footnote change GMNND to VPNNND. Editorial change to put acronyms in parentheses.</p>	Organizational Changes
6408	VCS	PT02	FSAR13	13.01F / F 13.1-203	<p>Revise Figure 13.1-203 to reflect current organization structure.</p>	Organizational Changes
6199	STD,VCS	PT02	FSAR13	13.02	<p>4. COLA Part 2, FSAR Chapter 13, Section 13.2, first sentence will be revised from:</p> <p>This section incorporates by reference NEI 06-13A, Technical Report on a Template for an Industry Training Program Description.</p>	BLN-VOL-LTR-004 response to NEI 06-13 item 4

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					To read: This section incorporates by reference NEI 06-13A, Template for an Industry Training Program Description.	
6200	STD,VCS	PT02	FSAR13	13.04.T / T13.4-201-08	1. COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, item 8, Fire Protection Program, will be revised (to add the following new milestone): (portions applicable to SNM) 10 CFR 30.32 Prior to initial receipt of byproduct source, or special nuclear materials (excluding Exempt Quantities as described in 10 CFR 30.18)	COL-SER-OI-Ch01 response to OI 01.05-01 item 1
6201	STD,VCS	PT02	FSAR13	13.04.T / T13.4-201-08	COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, items 8, 11, 14 and 15, will be revised to add a comma after "byproduct" in the milestone "Prior to initial receipt of byproduct..."	Editorial revision to COL-SER-OI-Ch01 response to OI 01.05-01 item 1
6205	STD,VCS	PT02	FSAR13	13.04.T / T13.4-201-10	COL Part 2 FSAR Chapter 13, Table 13.4-201 Item 10 will be revised to add a reference to 10 CFR 20.1406 to the Program:Source (Required by): column.	COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 5 SNC Letter #ND-09-1770
6206	STD,VCS	PT02	FSAR13	13.04.T / T13.4-201-10	COL Part 2 FSAR Chapter 13, Table 13.4-201 Item 10 will be revised to add a new sub-bullet "Minimization of Contamination" to the Program Title column.	COL-SER-OI-Ch12 S1 response to OI 12.03-001 item 6 SNC Letter #ND-09-1770
6202	STD,VCS	PT02	FSAR13	13.04.T / T13.4-201-11	2. COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, item 11, Non Licensed Plant Staff Training Program, will be revised (to add the following new milestone): (portions applicable to SNM) 10 CFR 30.32 Prior to initial receipt of byproduct source, or special nuclear materials (excluding Exempt Quantities as described in 10 CFR 30.18)	COL-SER-OI-Ch01 response to OI 01.05-01 item 2
6203	STD,VCS	PT02	FSAR13	13.04.T / T13.4-201-14	3. COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, item 14, Emergency Planning, will be revised (to add the following new milestone): (portions applicable to SNM) 10 CFR 30.32 Prior to initial receipt of byproduct source, or special nuclear materials (excluding Exempt Quantities as described in 10 CFR 30.18)	COL-SER-OI-Ch01 response to OI 01.05-01 item 3
6204	STD,VCS	PT02	FSAR13	13.04.T / T13.4-201-15	4. COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, item 15, Security Program, will be revised (to add the following new milestone): (portions applicable to SNM) 10 CFR 30.34 Prior to initial receipt of byproduct source, or special nuclear materials (excluding Exempt Quantities as described in 10 CFR 30.18)	COL-SER-OI-Ch01 response to OI 01.05-01 item 4
6208	STD,VCS	PT02	FSAR13	13.04.T / T13.4-201	COLA Part 2, FSAR Chapter 13, Table 13.4-201, Item 15, will be revised to move FFD to new line item along	Changes to address Security

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				15	<p>with new Cyber Security Program, and modify implementation milestones for Security Program items and FFD items.</p> <p>Current text: 15. Security Program: 10 CFR 50.34(c); Physical Security Program 10 CFR 73.55; 10 CFR 73.56; 10 CFR 73.57; 13.6 Prior to receipt of fuel onsite License Condition Safeguards Contingency Program 10 CFR 50.34(d); 10 CFR Part 73, Appendix C 13.6 Prior to receipt of fuel onsite License Condition Training and Qualification Program 10 CFR Part 73, Appendix B 13.6 Prior to receipt of fuel onsite License Condition Fitness for Duty Program (Construction - Mgt. & Oversight Personnel) 10 CFR Part 26, Subparts A-H, N, and O 13.7 Prior to initiating construction License Condition Fitness for Duty Program (Construction - Workers & First Line Supv.) 10 CFR Part 26, Subpart K 13.7 Prior to initiating construction License Condition Fitness for Duty Program (Operation) 10 CFR Part 26 13.7 Prior to initial fuel load License Condition</p> <p>Revised text: 15. Security Program: 10 CFR 50.34(c); Physical Security Program 10 CFR 73.55; 10 CFR 73.56; 10 CFR 73.57; 13.6 Prior to receipt of fuel onsite (protected area) License Condition Safeguards Contingency Program 10 CFR 50.34(d); 10 CFR Part 73, Appendix C 13.6 Prior to receipt of fuel onsite (protected area) License Condition Training and Qualification Program 10 CFR Part 73, Appendix B 13.6 Prior to receipt of fuel onsite (protected area) License Condition</p> <p>20. Fitness For Duty Programs: Fitness for Duty Program (Construction - Mgt. & Oversight Personnel) 10 CFR Part 26, Subparts A-H, N, and O 13.7 Prior to initiating onsite construction License Condition Fitness for Duty Program (Construction - Workers & First Line Supv.) 10 CFR Part 26, Subpart K 13.7 Prior to initiating onsite construction License Condition Fitness for Duty Program (Operation) 10 CFR Part 26 13.7 Prior to initial fuel load License Condition</p> <p>21. Cyber Security Program 10 CFR 73.54 13.6 Prior to initial fuel load License Condition</p>	Regulation revisions
6409	VCS	PT02	FSAR13	13AA.01.01.01.01.08	Revise FSAR Subsection 13AA.1.1.1.1.8, in the first two sentences change GMNND to VPNND.	Organizational Changes
6411	VCS	PT02	FSAR13	13AA.01.01.01.02.03	Revise FSAR Subsection 13AA.1.1.1.2.3, first paragraph, second sentence change GMNND to VPNND.	Organizational Changes
6412	VCS	PT02	FSAR13	13AAF / F 13AA-201	Revise Figure 13AA-201 to reflect current organizational structure.	Organizational Changes
6446	VCS	PT02	FSAR13	13AAF / F 13AA-202	Revise Figure 13AA-202 to reflect current hiring schedule for plant staff.	Revised hiring schedule
PT02 - FSAR14						13 COLA Changes
6219	STD,VCS	PT02	FSAR14	14.02.01	Revise reference to Regulatory Guide 1.206, Part 1, Section C.1.14.2 as identified in FSAR Section 14.2; first paragraph, last line to read Section C.1.14.2.	Editorial revision

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
6220	STD,VCS	PT02	FSAR14	14.02.01.04	Add LMA of STD COL 14.4-3 to the additions of 14.2.1.4 and 14.2.1.5 that follow 14.2.1.3.	Editorial revision
6221	STD,VCS	PT02	FSAR14	14.02.02.04 - 06	Revise Subsections 14.2.2.4 to remove untitled subsection numbers 14.2.2.4.1 and 14.2.2.4.2. The text remains, only the subsection numbers are removed. Revise Subsections 14.2.2.5 to remove untitled subsection numbers 14.2.2.5.1 and 14.2.2.5.2. The text remains, only the subsection numbers are removed. Revise Subsections 14.2.2.6 to remove untitled subsection numbers 14.2.2.6.1 and 14.2.2.6.2. The text remains, only the subsection numbers are removed.	Editorial revision
6223	STD,VCS	PT02	FSAR14	14.02.03	Add LMA of STD COL 14.4-3 to Section 14.2.3.	Editorial revision
6603	VCS	PT02	FSAR14	14.02.03.01	Insert horizontal separator lines between the following FSAR Sections: 14.2.3.1.1 and 14.2.3.1.2 14.2.3.1.2 and 14.2.3.1.3 14.2.3.1.3 and 14.2.3.1.4 14.2.3.1.4 and 14.2.3.1.5	Editorial
6226	STD,VCS	PT02	FSAR14	14.02.03.02.01	COLA Part 2, FSAR Subsection 14.2.3.2.1, fourth paragraph will be changed from: Startup test reports are prepared in accordance with the guidance in position C.1.a of Regulatory Guide 1.16, "Reporting of Operating Information -- Appendix A Technical Specifications" and position C.9 of Regulatory Guide 1.68, "Initial Test Programs for Water-Cooled Nuclear Power Plants." To read: Startup test reports are prepared in accordance with the guidance in position C.9 of Regulatory Guide 1.68, "Initial Test Programs for Water-Cooled Nuclear Power Plants."	Regulatory Guide 1.16 withdrawn by NRC 8-11-2009 via 74 FR 40244. This modifies the change in RAI LTR 139 S1 response to RAI 14.02-012, item 3 SER with Open Items Confirmatory Item 14.2-5
6227	STD,VCS	PT02	FSAR14	14.02.03.03.01	Add LMA of STD COL 14.4-4 to text beginning at Subsection 14.2.3.3.1.	Editorial revision
6228	STD,VCS	PT02	FSAR14	14.02.03.03.01	COLA Part 2, FSAR Subsection 14.2.3.3.1, first paragraph will be changed from: A startup report is submitted per Regulatory Guide 1.16 at the earliest of: To read: A startup report is submitted at the earliest of:	Regulatory Guide 1.16 withdrawn by NRC 8-11-2009 via 74 FR 40244.
6230	STD,VCS	PT02	FSAR14	14.02.05.01	COLA Part 2, FSAR Chapter 14, Subsection 14.2.5, as shown in Revision 1, will be revised to omit the subsection number of 14.2.5 and align the subtitle of "Utilization of Operating Experience," with the left margin.	Editorial; this text is added to DCD 14.2.5.
6484	STD,VCS	PT02	FSAR14	14.02.05.01	Revise the header from: Use of OE during Test Procedure Preparation To read: Use of OE During Test Procedure Preparation	Editorial
6234	STD,VCS	PT02	FSAR14	14.03.T / T14.3-201	3. COLA Part 2, Section 14.3, Table 14.3-201 legend, is revised to add: XX = Selected for ITAAC	RAI LTR 027 S1 response to RAI 14.03-001 item 3 SER with Open Items Confirmatory Item 8.2A-1 SER with Open Items Confirmatory Item 14.3-1
6235	STD,VCS	PT02	FSAR14	14.03.T / T14.3-201	2. COLA Part 2, Section 14.3, Table 14.3-201 entry for offsite power (with VCS SUP 14.3-2), is revised from: ZBS Transmission Switchyard and Offsite Power System XX(underlined) To read: ZBS Transmission Switchyard and Offsite Power System XX	RAI LTR 027 S1 response to RAI 14.03-001 item 2 SER with Open Items Confirmatory Item 8.2A-1 SER with Open Items Confirmatory Item 14.3-1
6236	STD,VCS	PT02	FSAR14	14.04.02	5. COLA Part 2, FSAR Chapter 14, Subsection 14.4.2, as shown in letter 139, will be revised from:	RAI LTR 139 S1 response to RAI 14.02-012, item 5 SER with Open Items Confirmatory

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					<p>Preoperational and startup procedures are provided for the NRC in accordance with the requirements of DCD Subsection 14.2.3. The controls for development of test specifications and procedures are also described in Subsection 14.4.3.</p> <p>A cross reference list is provided between ITAACs and test procedures and/or sections of test procedures.</p> <p>To read: Preoperational and startup test specifications and procedures are provided to the NRC in accordance with the requirements of DCD Subsection 14.2.3. The controls for development of test specifications and procedures are also described in Subsection 14.2.3.</p> <p>A cross reference list is provided between ITAACs and test procedures and/or sections of test procedures.</p>	Item 14.2.3
PT02 - FSAR16						1 COLA Change
6237	STD,VCS	PT02	FSAR16	16.01	<p>COLA Part 2, FSAR Chapter 16, Section 16.1, last two sentences, will be revised from: However, the generic technical specifications and bases provided with Chapter 16 of the DCD are incorporated by reference into the plant-specific technical specifications provided in Part 4 of this COL application. In addition, a full information set of the plant-specific technical specifications and bases are provided in Part 4 of this COL application.</p> <p>To read: However, the generic technical specifications and bases provided with Chapter 16 of the DCD are incorporated directly into the plant-specific technical specifications and bases provided in Part 4 of this COL application.</p>	Consistency with actual Part 4 information as of Revision 1.
PT02 - FSAR17						5 COLA Changes
6636	VCS	PT02	FSAR17	17.01	Change LMA for FSAR Section 17.1 from VCS SUP 17.1-1 to read VCS COL 17.5-1.	Correct left margin annotation.
6637	VCS	PT02	FSAR17	17.01	Revise V.C. Summer Nuclear Station Units 2 & 3 COL application to read V.C. Summer Nuclear Station Units 2 and 3.	Editorial
6238	STD,VCS	PT02	FSAR17	17.04	<p>COLA Part 2, FSAR, Chapter 17, Section 17.4 "Design Reliability Assurance Program" will be revised from: This section of the referenced DCD is incorporated by reference with no departures or supplements.</p> <p>To read: This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.</p> <p>STD SUP 17.4-1 The quality assurance requirements for non-safety related SSCs within the scope of D-RAP is in accordance with the Quality Assurance Program Description (QAPD), Part III.</p>	RAI LTR 150 response to RAI 17.04-002 SER with Open Items Confirmatory Item 17.4-1
6239	STD,VCS	PT02	FSAR17	17.05	<p>6. Revise FSAR Subsection 17.5 to include the following new paragraph following the existing first paragraph (with the same LMAs as the existing first paragraph):</p> <p>Conformance statements for QA-related Regulatory Guides (including Regulatory Guides 1.28, 1.30, 1.33, 1.38, 1.39, 1.94, and 1.116) are provided in Appendix 1AA. While many Regulatory Guide positions can be identified as applicable to the scope of work identified and addressed by the DCD and others can be identified as applicable to the scope of work identified and addressed by the COLA, some QA guidance related positions could be accomplished by either scope of work and thus be addressed in either the DCD or the COLA. These positions are primarily dependent on who performs the work. The DCD conformance statement indicates an exception to apply NQA-1. The COLA identifies an exception to apply NQA-1. Per DCD Section 17.3, WEC work performed up to March 15, 2007 applied a 1991 version of the standard. A 1994 version of the standard is applied for work performed after that date by WEC. If the work is performed under the applicant's COL program, the 1994 version of NQA-1 identified in the COLA QAPD is applied. Thus, DCD scope (identified in DCD Appendix 1A) and "remaining scope" differentiate the application of the guidance identified in these Regulatory Guides.</p>	COL-SER-OI-Ch01 S1 response to OI 01.04-02 item 6
6240	STD,VCS	PT02	FSAR17	17.06	Add "(Reference 208)" {red; hyperlinked text} after "...10 CFR Part 52;" as "...NEI-07-02A, Generic FSAR Template Guidance for Maintenance Rule Program Description for Plants Licensed Under 10 CFR Part 52;" (Reference 208); with the following supplemental information	Consistent method for identifying referenced documents.

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
PT02 - FSAR19						4 COLA Changes
6244	STD,VCS	PT02	FSAR19	19.59.10.05	<p>1. COLA Part 2, FSAR Chapter 19, subsection 19.59.10.5, STD COL 19.59.10-1, first three sentences will be changed from:</p> <p>A review of the differences between the as-built plant and the design used as the basis for the AP1000 seismic margins analysis will be completed prior to fuel load. A verification walkdown will be performed with the purpose of identifying differences between the as-built plant and the design. Any differences will be evaluated to determine if there is a significant adverse effect on the seismic margins analysis results.</p> <p>To read: A review of the differences between the as-built plant and the design used as the basis for the AP1000 seismic margins analysis will be completed prior to fuel load. A verification walkdown will be performed with the purpose of identifying differences between the as-built plant and the design. Any differences will be evaluated and the seismic margins analysis modified as necessary to account for the plant-specific design, and any design changes or departures from the certified design.</p>	RAI LTR 152 response to RAI 19-20 item 1
6245	STD,VCS	PT02	FSAR19	19.59.10.05	<p>2. COLA Part 2, FSAR Chapter 19, Subsection 19.59.10.5, STD COL 19.59.10-3 will be revised from:</p> <p>A review of the differences between the as-built plant and the design used as the basis for the AP1000 internal fire and internal flood analysis will be completed prior to fuel load. Differences will be evaluated to determine if there is significant adverse effect on the internal fire and internal flood analysis results.</p> <p>To read: A review of the differences between the as-built plant and the design used as the basis for the AP1000 internal fire and internal flood analyses will be completed prior to fuel load. Plant specific internal fire and internal flood analyses will be evaluated and the analyses modified as necessary to account for the plant-specific design, and any design changes or departures from the certified design.</p>	RAI LTR 152 response to RAI 19-20 item 2
6246	STD,VCS	PT02	FSAR19	19.59.10.06	Add "(Reference 201)" {red, hyperlinked text} under heading "PRA Input to the Reactor Oversight Process" at the end of the first paragraph to read - "The mitigating systems performance indicators (MSPI) are evaluated based on the indicators and methodologies defined in NEI 99-02 (Reference 201)."	Consistent method for identifying referenced documents.
6247	STD,VCS	PT02	FSAR19	19.59.11	Add the following to include new Reference 201: 19.59.11 References [separator bar] Add the following text to the end of DCD Subsection 19.59.11: 201. NEI 99-02, Nuclear Energy Institute, "Regulatory Assessment Performance Indicator Guideline," Technical Report NEI 99-02, Revision 5, July 2007.	Consistent method for identifying referenced documents.
PT04 - (empty)						7 COLA Changes
6304	VCS	PT04		04 F / F4.1-2	Revise Tech Spec Figure 4.1-2 for corrected EAB.	Revise figure to show corrected EAB.
6250	STD,VCS	PT04		A, A.2	<p>1) At the end of the Justification sentence for GTS 3.3.1 and 3.3.2 add the words "the plant specific" in front of "technical specifications".</p> <p>2) At the end of the Justification sentence for the 1st GTS 5.2.2 and 5.3 add the word "the" in front of "plant specific technical specifications".</p>	Editorial
6258	STD,VCS	PT04		B, 01.03	COLA Part 4, Section B, Technical Specification 1.3 will be revised on page 1.3-6 under EXAMPLES in the next to last sentence on the page to move the period inside the quotation marks around "clock."	Consistency with WEC AP1000 GTS
6262	STD,VCS	PT04		B, 03.07.09	COLA Part 4, Section B, Technical Specification SR 3.7.9.3 will be revised to add a comma after SFS-PL-V066.	Consistency with WEC AP1000 GTS
6645	VCS	PT04		B, B00 TOC/ Rev Summary	Technical Specifications Table of Contents/Revision Summary page, under Revision Column, Replace FSAR 1 with FSAR 2.	Conform to revision status of COLA
6267	STD,VCS	PT04		B, B03.03.02 -38	COLA Part 4, Section B, Bases 3.3.2, APPLICABLE SAFETY ANALYSES, LCOs, and APPLICABILITY Item 18.a, second paragraph will be revised to capitalize the first letter of "functions" in the sentence that begins "The functions of the P-4 interlock..." to read "The Functions of the P-4 interlock..."	Consistency with WEC AP1000 GTS

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
6270	STD,VCS	PT04		B, B03.03.02 -54	COLA Part 4, Section B, Bases 3.3.2, ACTIONS item J.1 and J.2, will be revised to remove the inadvertent line space in the middle of the third paragraph.	Consistency with WEC AP1000 GTS
PT05 - (empty)						55 COLA Changes
5868	VCS	PT05		05.00.T / T 5-1	Add the following footnote to ETE Table 5-1, column labeled "Residents With Commuters (Distribution.C)" (See page 5-13): (a) Distribution C also applies to vehicles evacuating from the Shadow Region.	RAI 13.03-37 (Letter 062 Response per NND-09-279)
6796	VCS	PT05		All Sections as applicable	Capitalize proper nouns. Examples include: Exclusion Area Boundary, Ingestion Pathway, Emergency Plan Procedures, Emergency Operating Procedures, Protection Action Directives, Department Names: Operations, Health Physics, Chemistry, Engineering, Maintenance, Security	Editorial Corrections
6797	VCS	PT05		All Sections as applicable	Editorial Consistency - Remove periods after bulleted items	Editorial Corrections
5871	VCS	PT05		Annex 2 / Page 2-9	Revise Annex 2, page 2-9 (Revision 1 page number) from: Unit 2 EALs to be developed in accordance with NEI 07-01, Methodology for Development of Emergency Action Levels Advanced Passive Light Water Reactors. to read: Intentionally left blank	RAI 13.03-41 (Letter 062 Response per NND-09-0279)
5872	VCS	PT05		Annex 3 / Page 3-9	Revise Annex 3, page 3-9 (Revision 1 page number) from: Unit 2 EALs to be developed in accordance with NEI 07-01, Methodology for Development of Emergency Action Levels Advanced Passive Light Water Reactors. to read: Intentionally left blank	RAI 13.03-41 (Letter 062 Response per NND-09-0279)
6740	VCS	PT05		Part 1 Figure 1-2 / Page 7	Correctly depict EAB Boundary to include tail race and generation facility at Fairfield Pump Storage. Update Units 2 and 3 site layout to include relocation of plant access roads.	Update Figure to show corrected EAB and updated Site Plan.
6798	VCS	PT05		Part 1 Section H / Page 4	Add comma after "Unusual Event"	Editorial Corrections
6809	VCS	PT05		Part 2 / Figures B-1a, B-1b, B-1c, B-1d	Update Organizational Charts for ERO: 1) Update position titles for consistency with changes made in Plan. 2) Rearranged physical alignment of organizational charts, without altering reporting structures.	Editorial Consistency with revised ERO titles in body of plan.
6799	VCS	PT05		Part 2 / Page A-4	Replace "PARs" with "Protective Action Directives (PAD)"	Consistency with operational protocol with State Government. Governor issues directives, not recommendations.
6742	VCS	PT05		Part 2 / Page A-5	Revise the first paragraph under "3) County Government Agencies" to read: "VCSNS and the surrounding counties of Fairfield, Newberry, Lexington, and Richland that comprise the plume exposure pathway EPZ have developed integrated emergency response programs that call upon the resources of their county. The county organizations are responsible for implementing and coordinating the county response to an emergency."	Remove redundant sentence repeating county names.
6800	VCS	PT05		Part 2 / Page A-6	Revise the last paragraph from: "The state and counties have emergency response plans that specify the responsibilities and functions for the major agencies, departments, and key individuals of their EROs. This information is located in their respective plans." To read:	Consistency with state operating organization titles. The state does not use "EROs", they use "organizations"

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					"The state and counties have emergency response plans that specify the responsibilities and functions for the major agencies, departments, and key individuals of their organizations. This information is located in their respective plans."	
6801	VCS	PT05		Part 2 / Page A-7	In the first paragraph, correct the run-on sentence from: "Agreement letters are not necessary with federal agencies that are legally required to respond based on federal law; however, agreements are necessary if the agency was expected to provide assistance not required by law." To Read: "Agreement letters are not necessary with federal agencies that are legally required to respond based on federal law. However, agreements are necessary if the agency was expected to provide assistance not required by law."	Editorial Correction
6802	VCS	PT05		Part 2 / Page A-7	In the last paragraph, correct the spelling of "facility".	Editorial Corrections
6803	VCS	PT05		Part 2 / Page B-1	In the first sentence of the second paragraph, add "Sr" to the "VP" title.	Updated to reflect organizational changes
6860	VCS	PT05		Part 2 / Page B-1	In the second sentence of the second paragraph, change "site" to "unit".	Consistency with naming convention
6861	VCS	PT05		Part 2 / Page B-1	In the 3rd paragraph delete "Contained in each Unit Annex"	Remove redundant wording
6862	VCS	PT05		Part 2 / Page B-1	In the fourth paragraph, delete "On-Shift Personnel!" heading.	Paragraph description applies to all subsequent positions described.
6863	VCS	PT05		Part 2 / Page B-1	In the fifth paragraph, spell out acronym "Interim Emergency Director".	Acronym not previously used in document, needs to be spelled out.
6864	VCS	PT05		Part 2 / Page B-1	In the sixth paragraph, change title of "Shift Technical Advisor" to "Shift Engineer".	VCSNS does not use title "STAs" rather "Shift Engineer"
6810	VCS	PT05		Part 2 / Page B-17 / Tables B-1a	Consolidate Table B-1a with B-1b	Consolidate tables to reference all staffing requirements. Table B1a shows Minimum Shift staffing and B1b shows minimum ERO Staffing.
6804	VCS	PT05		Part 2 / Page B-2	In the 6th paragraph, change the position title "First Aid" to "Medical Emergency Response Team (MERT)".	Consistency with plant nomenclature
6865	VCS	PT05		Part 2 / Page B-2	In the 7th paragraph, change "Emergency Communicator" to "State/County Communicator".	Alignment of position titles to be consistent with position responsibilities (previously had multiple position titles with same responsibility)
6805	VCS	PT05		Part 2 / Page B-2 through B-4	In the 2nd paragraph under Item 2, revise from: Control Room: IED (Shift Supervisor), initially in command until relieved by on-call ERO members in the EOF or by the Emergency Plant Operations Supervisor (EPOS) in the Control Room or other designed response location. to read: Control Room: IED (Shift Supervisor) is initially in command and control until relieved by the Emergency Plant Operations Supervisor (EPOS) in the Control Room or other designated response location or by the Emergency Director (ED) in the EOF.	Properly designate turnover of command and control when the ERO is activated based on actions and timings to activate the ERO.
6806	VCS	PT05		Part 2 / Page B-4	In the first paragraph, update the Table number from "B-1A" to "2-1"	consistency with table changes made in Item 6810.
6866	VCS	PT05		Part 2 / Page B-4	In the first paragraph, update the Table number from "B-1A" to "2-1"	consistency with table changes made in Item 6810
6867	VCS	PT05		Part 2 / Page B-4	In the third bulleted item, define the acronym "Emergency Public Information".	Acronym not previously established.
6868	VCS	PT05		Part 2 / Page B-4	In the sixth paragraph, change "suborganization" to "sub organization".	Grammatical correction.

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
6869	VCS	PT05		Part 2 / Page B-5	In the second paragraph, change "page / phone call" to "pager or telephone call" - grammatical correction.	Grammatical correction
6870	VCS	PT05		Part 2 / Page B-5	In the sixth paragraph, abbreviate "Control Room" to "CR".	Grammatical correction
6871	VCS	PT05		Part 2 / Page B-5	In the sixth paragraph, abbreviate "Control Room" to "CR".	Grammatical correction
6807	VCS	PT05		Part 2 / Page B-5 and B-6	In the first paragraph, change position title from "Technical Assessment Coordinator" to "Technical Support Manager".	Title change for consistency with ERO position nomenclature.
6808	VCS	PT05		Part 2 / Page B-5 through B-10	Update ERO position titles for consistency with the other parts of the E-Plan. ie: "#4 Technical Support Center Manager" changed to "Technical Support Manager"	Consistency with organizational title nomenclature and consistency with other parts of the E-Plan.
6811	VCS	PT05		Part 2 / Page C-2	Revise the first sentence of the 2nd paragraph from: "An additional laboratory facility is available at DHEC. DHEC also has a mobile laboratory for analyzing environmental samples." to read: "An additional laboratory facility is available at DHEC, other nuclear facilities, or private labs. DHEC also has a mobile laboratory for analyzing environmental samples."	Consistency with Page H-9 Item c, describing the availability of external laboratories in addition to DHEC.
6812	VCS	PT05		Part 2 / Page D-5	Correct the reference to the initiating conditions having the designators as EALs	Initiating conditions are not designated by alphanumeric notation, however EALs are.
6814	VCS	PT05		Part 2 / Page F-2	Add Virtual Private Network (VPN) as an option for providing electronic plant data to the NRC.	Added option for VPN for future flexibility in addition to traditional modem link to the NRC.
6815	VCS	PT05		Part 2 / Page G-2 and G-3	Under the Joint Information Center description, remove bulleted item for providing telephones for use by the news media personnel.	Telephones are not provided for use due to availability of cellular technology.
6817	VCS	PT05		Part 2 / Page H-03	Under The Emergency Operations Facility, Correct the location of the EOF from: "corner of Bickley Road ..." to "near the intersection of Bickley Road."	Accurately describe the physical location of the EOF.
6816	VCS	PT05		Part 2 / Page H-1	Under the Control Room heading, Add the statement "In the event of an emergency only affecting Units 2 & 3, the Unit 2 Control Room will take the lead and will house the IED."	Ensure proper command and control for events involving Units 2 and 3.
6818	VCS	PT05		Part 2 / Page H-12	Correct the location of the Environmental Lab from the EOF to the Nuclear Operations Building (NOB).	The environmental lab is no longer located in the EOF.
6819	VCS	PT05		Part 2 / Page I-04	Correct the location of the field sample collection point from the EOF to the Nuclear Operations Building.	The field sample collection point is no longer located in the EOF.
6820	VCS	PT05		Part 2 / Page J-01	Revise the last sentence under Radiological Monitoring of Evacuees from: "If there is no release of radioactive materials within the affected unit, limited monitoring (less than 100% of evacuees) may be used to speed the evacuation process." To read: "If there is no release of radioactive materials within the affected unit, limited monitoring may be used to speed the evacuation process."	Remove redundant wording.
6821	VCS	PT05		Part 2 / Page K-03	Revise the last sentence under item a. from: "An ambulance responding and transporting injured contaminated personnel will be monitored and decontaminated before departing the medical facility by VCSNS personnel." to read: "An ambulance responding and transporting injured contaminated personnel will be monitored and decontaminated before departing the medical facility by VCSNS personnel or sent to the county emergency worker decon facility, during a declared emergency."	Add the condition that ambulances could be sent to the county emergency worker decon per the county plans.

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
6822	VCS	PT05		Part 2 / Page L-01	Consolidate the "Medical Services Facilities" section with the "Offsite Hospital and Medical Services."	Align all medical facilities under one section
6823	VCS	PT05		Part 2 / Page N-02	On Page N-2, relocated "Pre-Exercises" subsection from Section 1. Exercises to Section 2. Drills and retitled to "Training Drill"	Alignment to proper procedure section
6813	VCS	PT05		Part 2 / Sections E and G	Add Television to references of the Emergency Alert System.	South Carolina EAS is broadcast over radio and television.
6827	VCS	PT05		Part 2 Annex 1 / EAL Matrix	Update the EAL Matrix to reflect the EALs approved by the NRC and implemented in April 2009.	VCSNS Unit 1 EAL SER Issued April 1, 2009.
6829	VCS	PT05		Part 2 Annex 1 / Page Annex 1-31	In the first paragraph on page Annex 1-31, change "raised radiation levels" to "elevated radiation levels"	Editorial
6830	VCS	PT05		Part 2 Annex 1 / Page Annex 1-32	Remove reference to specific version of Windows.	Add flexibility to use future versions of Windows software.
6825	VCS	PT05		Part 2 Annex 1 / Page Annex 1-7	Update the ERO titles for consistency with Section B and Table B-1a and the multiple unit command and control structure.	Conforming change with Item 6816 and 6810
6826	VCS	PT05		Part 2 Annex 1 / Page Annex 1-7	Update table based on revisions to Table B-1a.	Editorial Consistency
6828	VCS	PT05		Part 2 Annex 1 / Table 4-1	In Table 4-1, under Atmospheric Radiation Monitors, right-justify "Gas" and "Iodine" for consistency with "Particulate" entry.	Formatting Change
6831	VCS	PT05		Part 2 Annex 2 and 3	Update the ERO titles based on corrections made within Section B and Table B-1a.	Conforming Change with Item 6810
6832	VCS	PT05		Part 2 Appendix 4	Updated Appendix formatting such that acronyms are left justified and all accompanying text is center justified away from acronym.	Editorial formatting correction
6833	VCS	PT05		Part 2 Appendix 5	Add reference to the current revision of the ETE.	Reflect the most current revision of the ETE: Revision 3, November 2009.
6834	VCS	PT05		Part 2 Appendix 6 / Table 3	Make the following changes to Appendix 6, Table 3. Update tables to remove unnecessary highlighting and update table references for the conforming changes made in item 6810.	Formatting update and conforming changes made in Item 6810
PT07 - (empty)						1 COLA Change
6398	VCS	PT07			Revise Part 7 of the VCSNS COL Application to add a departure and exemption as described in response to RAI 2.3.1-7 in SCE&G Letter NND-09-0274.	RAI 2.3.1-7 (Letter 064 Response per NND-09-0274)
PT10 - (empty)						16 COLA Changes
5873	VCS	PT10		11	License Condition 11 will be revised to read: The licensee shall submit a fully developed set of plant-specific Emergency Action Levels (EALs) for VCSNS Units 2 and 3 in accordance with NET 07-01 Revision 0. These fully developed EALs shall be submitted to the NRC for confirmation at least 180 days prior to initial fuel load. The submitted EALs will be written with no deviations.	RAI 13-03-41 (Letter 062 Response per NND-09-0279)
6273	STD,VCS	PT10		LC#02, 09.01-07	2- COLA Part 10, License Conditions, COL Item No. 9.1-7 will be revised from: A spent fuel rack Metamic coupon monitoring program is to be implemented when the plant is placed into commercial operation. This program includes tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and or visual examination. To read: A spent fuel rack Metamic coupon monitoring program is to be implemented when the plant is placed into commercial operation. This program includes tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and / or visual examination. The program will also include tests to monitor changes in physical properties of the absorber material, including neutron attenuation	RAI LTR 165 in response to RAI 09.01.02-001 item 2 Duplicate of QB Item #5477

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change																
					and thickness measurements.																	
6642	STD,VCS	PT10		LC#02, 09.01-07	<p>Revise Part 10, License Condition 2, COL Item No. 9.1-7, from:</p> <p>A spent fuel rack Metamic coupon monitoring program is to be implemented when the plant is placed into commercial operation. This program includes tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and/or visual examination. The program also includes test to monitor changes in physical properties of the absorber material, including neutron attenuation and thickness measurements.</p> <p>To read:</p> <p>A spent fuel rack Metamic coupon monitoring program will be implemented when the plant is placed into commercial operation. This program will include tests to monitor bubbling, blistering, cracking, or flaking; and a test to monitor for corrosion, such as weight loss measurements and/or visual examination. The program will also include testing to monitor changes in physical properties of the absorber material, including neutron attenuation and thickness measurements.</p>	Editorial revision to RAI LTR 165 (& Supp 1) in response to RAI 09.01-02-001 item 1																
6275	STD,VCS	PT10		LC#02, 14.04-03	<p>3. COLA Part 10 will be revised from:</p> <table border="1"> <thead> <tr> <th>COL Item No.</th> <th>Subject</th> <th>From DCD Tier 2 Subsection</th> <th>Implementation Milestone</th> </tr> </thead> <tbody> <tr> <td>14.4-3</td> <td>Conduct of Test Program</td> <td>14.4.3</td> <td>Prior to initiating test program</td> </tr> </tbody> </table> <p>A site-specific startup administration manual (procedure), which contains the administration procedures and requirements that govern the activities associated with the plant initial test program, as identified in DCD Subsection 14.2.3 and as described in APP-GW-GLR-038 (DCD Reference 2), is provided prior to initiating the plant initial test program.</p> <p>To read:</p> <table border="1"> <thead> <tr> <th>COL Item No.</th> <th>Subject</th> <th>From DCD Tier 2 Subsection</th> <th>Implementation Milestone</th> </tr> </thead> <tbody> <tr> <td>14.4-3</td> <td>Conduct of Test Program</td> <td>14.4.3</td> <td>Prior to initiating test program</td> </tr> </tbody> </table> <p>A site-specific startup administration manual (procedure), which contains the administration procedures and requirements that govern the activities associated with the plant initial test program, as identified in FSAR Section 14.2, is provided prior to initiating the plant initial test program.</p>	COL Item No.	Subject	From DCD Tier 2 Subsection	Implementation Milestone	14.4-3	Conduct of Test Program	14.4.3	Prior to initiating test program	COL Item No.	Subject	From DCD Tier 2 Subsection	Implementation Milestone	14.4-3	Conduct of Test Program	14.4.3	Prior to initiating test program	RAI LTR 139 response to RAI 14.02-012, item 3 SER with Open Items Confirmatory Item 14.2-1
COL Item No.	Subject	From DCD Tier 2 Subsection	Implementation Milestone																			
14.4-3	Conduct of Test Program	14.4.3	Prior to initiating test program																			
COL Item No.	Subject	From DCD Tier 2 Subsection	Implementation Milestone																			
14.4-3	Conduct of Test Program	14.4.3	Prior to initiating test program																			
6276	STD,VCS	PT10		LC#02, 19.59.10-01	<p>3. COLA Part 10, License Conditions and ITAAC VCS Proposed License Condition 2, COL Holder Items, first paragraph, will be revised from:</p> <p>As-Built SSC HCLPF</p> <table border="1"> <thead> <tr> <th>19.59.10-1</th> <th>Comparison to Seismic Margin Evaluation</th> <th>19.59.10.5</th> <th>Prior to initial fuel load</th> </tr> </thead> <tbody> <tr> <td></td> <td></td> <td></td> <td></td> </tr> </tbody> </table> <p>The Combined License holder referencing the AP1000 certified design will review differences between the as-built plant and the design used as the basis for the AP1000 seismic margins analysis prior to fuel load. A verification walkdown will be performed with the purpose of identifying differences between the as-built plant and the design. Any differences will be evaluated to determine if there is a significant adverse effect on the seismic margins analysis results. Spatial interactions are addressed by COL information item 3.7-3. Details of the process will be developed by the Combined License holder.</p> <p>To read:</p> <table border="1"> <thead> <tr> <th>19.59.10-1</th> <th>Comparison to Seismic Margin Evaluation</th> <th>19.59.10.5</th> <th>Prior to initial fuel load</th> </tr> </thead> <tbody> <tr> <td></td> <td></td> <td></td> <td></td> </tr> </tbody> </table> <p>The Combined License holder referencing the AP1000 certified design will review differences between the as-built plant and the design used as the basis for the AP1000 seismic margins analysis prior to fuel load. A verification</p>	19.59.10-1	Comparison to Seismic Margin Evaluation	19.59.10.5	Prior to initial fuel load					19.59.10-1	Comparison to Seismic Margin Evaluation	19.59.10.5	Prior to initial fuel load					RAI LTR 152 response to RAI 19-20 item 3 SER with Open Items Confirmatory Item 19.59-1
19.59.10-1	Comparison to Seismic Margin Evaluation	19.59.10.5	Prior to initial fuel load																			
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Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					walkdown will be performed with the purpose of identifying differences between the as-built plant and the design. Any differences will be evaluated and the seismic margins analysis modified as necessary to account for the plant specific design, and any design changes or departures from the certified design. Spatial interactions are addressed by COL information item 3-7-3. Details of the process will be developed by the Combined License holder.	
6277	STD,VCS	PT10		LC#02, 19.59.10-01	COLA Part 10, License Conditions and ITAAC, VCS Proposed License Condition 2, COL Holder Item 19.59.10-1, first paragraph, will be revised from: Any differences will be evaluated and the seismic margins analysis modified as necessary to account for the plant specific-design, and any design changes or departures from the certified design. To read: Any differences will be evaluated and the seismic margins analysis modified as necessary to account for the plant-specific design and any design changes or departures from the certified design.	Editorial revision to Qb 6276 - BLN RAI LTR 152 response to RAI 19-20 item 3 SER with Open Items Confirmatory Item 19.59-1
6278	STD,VCS	PT10		LC#02, 19.59.10-02	4. COLA Part 10, License Conditions and ITAAC, VCS Proposed License Condition 2, COL Holder Items, will be revised from: Evaluation of As-Built Plant Versus Design in AP1000 PRA 19.59.10-2 and Site-Specific PRA External 19.59.10.5 Prior to initial fuel load Events The Combined License holder referencing the AP1000 certified design will review differences between the as-built plant and the design used as the basis for the AP1000 PRA and Table 19.59-18 prior to fuel load. If the effects of the differences are shown, by a screening analysis, to potentially result in a significant increase in core damage frequency or large release frequency, the PRA will be updated to reflect these differences. To read: Evaluation of As-Built Plant Versus Design in AP1000 PRA 19.59.10-2 and Site-Specific PRA External 19.59.10.5 Prior to initial fuel load Events The Combined License holder referencing the AP1000 certified design will review differences between the as-built plant and the design used as the basis for the AP1000 PRA and Table 19.59-18 prior to fuel load. The plant-specific PRA-based insight differences will be evaluated and the plant specific PRA model modified as necessary to account for the plant specific design and, any design changes or departures from the design certification PRA.	RAI LTR 152 response to RAI 19-20 item 4 SER with Open Items Confirmatory Item 19.59-1
6279	STD,VCS	PT10		LC#02, 19.59.10-02	COLA Part 10, License Conditions and ITAAC, VCS Proposed License Condition 2, COL Holder Item 19.59.10-2, will be revised from: The plant specific PRA-based insight differences will be evaluated and the plant specific PRA model modified as necessary to account for the plant specific-design and, any design changes or departures from the design certification PRA. To read: The plant-specific PRA-based insight differences will be evaluated and the plant-specific PRA model modified as necessary to account for the plant-specific design and any design changes or departures from the design certification PRA.	Editorial revision to Qb 6278 - BLN RAI LTR 152 response to RAI 19-20 item 4 SER with Open Items Confirmatory Item 19.59-1
6280	STD,VCS	PT10		LC#02, 19.59.10-03	5. COLA Part 10, License Conditions and ITAAC, VCS Proposed License Condition 2, COL Holder Items, will be revised from: Internal Fire and Internal 19.59.10-3 Flood Analyses 19.59.10.5 Prior to initial fuel load The Combined License holder referencing the AP1000 certified design will review differences between the as-built plant and the design used as the basis for the AP1000 internal fire and internal flood analysis prior to fuel load. Differences will be evaluated to determine if there is significant adverse effect on the internal fire and internal flood analysis results. To read: Internal Fire and Internal	RAI LTR 152 response to RAI 19-20 item 5 SER with Open Items Confirmatory Item 19.59-1

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					19.59.10-3 Flood Analyses 19.59.10.5 Prior to initial fuel load The Combined License holder referencing the AP1000 certified design will review differences between the as-built plant and the design used as the basis for the AP1000 internal fire and internal flood analyses prior to fuel load. Plant specific internal fire and internal flood analyses will be evaluated and the analyses modified as necessary to account for the plant-specific design, and any design changes or departures from the certified design	
6281	STD,VCS	PT10		LC#02, 19.59.10-03	COLA Part 10, License Conditions and ITAAC, Proposed License Condition 2, COL Holder Item 19.59.10-3 will be revised from: Plant specific internal fire and internal flood analyses will be evaluated and the analyses modified as necessary to account for the plant-specific design, and any design changes or departures from the certified design To read: Plant-specific internal fire and internal flood analyses will be evaluated and the analyses modified as necessary to account for the plant-specific design and any design changes or departures from the certified design.	Editorial revision to BLN RAI LTR 152 response to RAI 19-20 item 5 SER with Open Items Confirmatory Item 19.59-1
6282	STD,VCS	PT10		LC#02, 19.59.10-04	2. COLA Part 10, License Conditions and ITAAC, VCS Proposed License Condition 2, Col Holder Items, will be revised from: 19.59.10-4 Develop and Implement Severe Accident Management Guidance 19.59.10.5 Prior to startup testing To read: 19.59.10-4 Implement Severe Accident Management Guidance 19.59.10.5 Prior to startup testing	RAI LTR 152 response to RAI 19-21 item 2 SER with Open Items Confirmatory Item 19.59-2
6284	STD,VCS	PT10		LC#03	COLA Part 10, Proposed License Conditions (Including ITAAC), Proposed License Condition 3, introductory statement, will be revised from: The licensee shall implement the programs or portions of programs identified in Table 13.4-201 on or before the associated milestones in Table 13.4-201. To read: The licensee shall implement the programs or portions of programs identified below on or before the associated milestones identified below.	Editorial revision to remove references to FSAR in proposed License Condition.
6285	STD,VCS	PT10		LC#03 C	5. COLA Part 10, Proposed License Condition 3, Operational Program Implementation, will be revised to add the following new milestones: C.2 - Fire Protection Program (applicable portions) C.3 - Non Licensed Plant Staff Training Program (applicable portions) C.4 - Emergency Planning (applicable portions) C.5 - Security Program (applicable portions)	COL-SER-OI-Ch01 response to OI 01.05-01 item 5
6287	STD,VCS	PT10		LC#06	COLA Part 10, proposed License Condition 6, will be revised from: 6. OPERATIONAL PROGRAM READINESS: The NRC inspection of operational programs will be the subject of the following license condition in accordance with SECY-05-0197. PROPOSED LICENSE CONDITION: The licensee shall submit to the appropriate Director of the NRC, a schedule, no later than 12 months after issuance of the COL, that supports planning for and conduct of NRC inspections of operational programs listed in the operational program FSAR Table 13.4-201. The schedule shall be updated every 6 months until 12 months before scheduled fuel loading, and every month thereafter until either the operational programs in the FSAR table have been fully implemented or the plant has been placed in commercial service, whichever comes first. a. This schedule shall include a submittal schedule for the emergency planning implementation procedures to the NRC consistent with 10 CFR Part 50, Appendix E, Section V. b. This schedule shall include a schedule for the development of the site specific Severe Accident Management Guidance. c. This schedule shall include a submittal schedule for the reactor vessel pressurized thermal shock evaluation at least 18 months prior to initial fuel load. d. This schedule shall include a submittal schedule for approved preoperational and startup test procedures in accordance with FSAR Section 14.2.3.	COL-SER-OI-Ch10 response to OI 10.01-01

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					<p>To read: 6. OPERATIONAL PROGRAM READINESS: The NRC inspection of operational programs will be the subject of the following license condition in accordance with SECY-05-0197. PROPOSED LICENSE CONDITION: The licensee shall submit to the appropriate Director of the NRC, a schedule, no later than 12 months after issuance of the COL, that supports planning for and conduct of NRC inspections of operational programs listed in the operational program FSAR Table 13.4-201. The schedule shall be updated every 6 months until 12 months before scheduled fuel loading, and every month thereafter until either the operational programs in the FSAR table have been fully implemented or the plant has been placed in commercial service, whichever comes first. This schedule shall include a submittal schedule for: a. the emergency planning implementation procedures to the NRC consistent with 10 CFR Part 50, Appendix E, Section V. b. the implementation of site specific Severe Accident Management Guidance. c. a reactor vessel pressurized thermal shock evaluation at least 18 months prior to initial fuel load. d. approved preoperational and startup test procedures in accordance with FSAR Subsection 14.2.3. e. an emergency response data system (ERDS) implementation program plan consistent with 10 CFR Part 50, Appendix E, Section V. f. a flow accelerated corrosion (FAC) program implementation schedule, including the construction phase activities.</p>	
6643	STD,VCS	PT10		LC#06	<p>COLA Part 10, proposed License Condition 6, will be revised from: This schedule shall include a submittal schedule for: a. the emergency planning implementation procedures to the NRC consistent with 10 CFR Part 50, Appendix E, Section V. b. the implementation of site specific Severe Accident Management Guidance. c. a reactor vessel pressurized thermal shock evaluation at least 18 months prior to initial fuel load. d. approved preoperational and startup test procedures in accordance with FSAR Subsection 14.2.3. e. an emergency response data system (ERDS) implementation program plan consistent with 10 CFR Part 50, Appendix E, Section V. f. a flow accelerated corrosion (FAC) program implementation schedule, including the construction phase activities.</p> <p>To read: This schedule shall address: a. the implementation of site specific Severe Accident Management Guidance. b. the reactor vessel pressurized thermal shock evaluation at least 18 months prior to initial fuel load. c. the approved preoperational and startup test procedures in accordance with FSAR Subsection 14.2.3. d. the flow accelerated corrosion (FAC) program implementation, including the construction phase activities.</p>	<p>Remove duplication of Emergency Planning ITAAC table items 3.2 and 9.1. This entry revises Qb 5243.</p>
6290	STD,VCS	PT10		LC#AppB	<p>4. COLA Part 10, Appendix B, is revised to include the following new site-specific ITAAC from: Add the following information to the information provided in the referenced DCD Tier 1 following Section 2.6.11: 2.6.12 Transmission Switchyard and Offsite Power System No entry for this system.</p> <p>To read: Add the following information to the information provided in the referenced DCD Tier 1 following Section 2.6.11: 2.6.12 Transmission Switchyard and Offsite Power System Inspection, Test, Analysis and Acceptance Criteria</p> <p>Table 2.6.12-1 provides a definition of the inspections, tests, and/or analyses, together with associated acceptance criteria for the offsite power system.</p> <p>[For table format and content see supplemental response to RAI LTR 027 - include Table 2.6.12-1 after Table 2.6.9-1.]</p> <p style="text-align: center;">Table 2.6.12-1</p>	<p>RAI LTR 027 S1 response to RAI 14.03-001 item 4 SER with Open Items Confirmatory Item 8.2A-1 SER with Open Items Confirmatory Item 14.3-1</p>

Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					Offsite Power System	

SUMMARY		
COLA Part REP	Chapter REP	Number of COLA Changes
PT01	(empty)	6
PT02	FSAR01	58
PT02	FSAR02	98
PT02	FSAR03	8
PT02	FSAR05	3
PT02	FSAR06	8
PT02	FSAR08	5
PT02	FSAR09	10
PT02	FSAR10	5
PT02	FSAR11	9
PT02	FSAR12	18
PT02	FSAR13	21
PT02	FSAR14	13
PT02	FSAR16	1
PT02	FSAR17	5
PT02	FSAR19	4
PT04	(empty)	7
PT05	(empty)	55
PT07	(empty)	1
PT10	(empty)	16
TOTALS (20 groups)		351