

ATTACHMENT A

Technical Specification  
Page Revisions

Consolidated Edison Company of New York, Inc.  
Indian Point Unit No. 2  
Docket No. 50-247  
December, 1981

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TABLE 3-5  
TABLE OF INDICATORS AND/OR RECORDERS AVAILABLE TO THE OPERATOR

PARAMETER	1	2	3
	NO. OF CHANNELS AVAILABLE	MIN. NO. OF CHANNELS REQUIRED <sup>(1)</sup>	INDICATOR/RECORDER <sup>(1)</sup>
1. Pressurizer Water Level	3	2	Indicator/One Channel is recorded
2. Reactor Coolant System Subcooling Margin Monitor <sup>(2)</sup>	1	1	Indicator
3. PORV Position Indicator (Limit Switch)	1/Valve <sup>(3)</sup>	1/Valve <sup>(3)</sup>	Indicator and alarm <sup>(3)</sup>
4. PORV Block Valve Position Indicator (Limit Switch)	1/Valve <sup>(3)</sup>	1/Valve <sup>(3)</sup>	Indicator <sup>(3)</sup>
5. Safety Valve Position Indicator (Acoustic Monitor)	1/Valve	1/Valve	Indicator
6. Auxiliary Feedwater Flow Rate	1/S.G.	1/S.G.	Indicator
7. Plant Vent Noble Gas Effluent Monitor(R-27)	1 <sup>(4)</sup>	1 <sup>(4)</sup>	Indicator Recorder <sup>(4)</sup>
8. Main Steam Line Radiation Monitor(R-28,R-29,R-30,R-31)	1/Steam line <sup>(4)</sup>	1/Steam line <sup>(4)</sup>	Indicator, Recorder and alarm <sup>(4)</sup>

Footnotes:

(1) Except as specified in another footnote, columns 2 and 3 may be modified to allow the instrument channels to be inoperable for up to 7 days and/or recorder(s) to be inoperable for up to 14 days. If the minimum number of channels required is not restored to meet the above requirements within the time periods specified, then:

- a. If the reactor is critical, it shall be brought to the hot shutdown condition utilizing normal operating procedures. The shutdown shall start no later than at the end of the specified time period.
- b. If the requirements of Columns 2 and 3 are not satisfied within an additional 48 hours, the reactor shall be cooled to below 350°F utilizing normal operating procedures. The shutdown shall start no later than the end of the 48 hour period.

TABLE 3-5 (Continued)

Footnotes (Cont'd):

- (2) If the subcooling margin monitor is inoperable for more than seven (7) days, plant operation may continue for an additional thirty (30) days provide that steam tables are continuously maintained in the control room and the subcooling margin is determined and recorded once a shift.
- (3) Except at times when the valve operator is deenergized in accordance with technical specification 3.1.A.4.c.
- (4) With the number of operable channels less than required by the minimum number of channels requirement, either restore the inoperable channel(s) to operable status within 72 hours, or:
  - a. Initiate an alternate method of monitoring the appropriate parameter(s), and
  - b. Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2.g within the next 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Region I Office of Inspection and Enforcement within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Each containment integrated leak rate test shall be the subject of a summary technical report including results of the local leak rate test since the last report. The report shall include analyses and interpretations of the results which demonstrate compliance in meeting the leak rate limits specified in the Technical Specifications.
- b. A report covering the X-Y xenon stability tests within three months upon completion of the tests.
- c. To provide the Commission with added verifications of the safety and reliability of the pre-pressurized Zircaloy-clad nuclear fuel, a limited program of non-destructive fuel inspections will be conducted. The program shall consist of a visual inspection (e.g., underwater TV, periscope, or other) of the two lead burnup assemblies in each region during the first, second, and third refueling shutdowns. Any condition observed by this inspection which would lead to unacceptable fuel performance may be the object of an expanded surveillance effort. If another domestic plant which contains pre-pressurized fuel of a similar design reaches fuel exposures equal to or greater than at Indian Point Unit No. 2, and if a limited inspection program is or has been performed there, then the program may not have to be performed at Indian Point Unit No. 2. However, such action requires approval of the Nuclear Regulatory Commission. The results of these inspections will be reported to the Nuclear Regulatory Commission.
- d. Inoperable fire protection and detection equipment (Specification 3.13).
- e. Sealed source leakage in excess of limits (Specification 4.15).
- f. The complete results of the steam generator tube inservice inspection (Specification 4.13.C).
- g. Inoperable radiation monitoring instrumentation (Specification 3.5).

ATTACHMENT B

Safety Evaluation

Consolidated Edison Company of New York, Inc.  
Indian Point Unit No. 2  
Docket No. 50-247  
December, 1981

## Safety Evaluation

By letters dated October 7, 1981 and November 19, 1981, the NRC Staff requested Consolidated Edison to propose technical specification revisions to incorporate requirements relative to NUREG-0737 Items II.F.1.1 and II.F.1.2.

The proposed technical specification changes are contained in Attachment A to this Application. These proposed changes will establish new requirements in the area of plant effluent monitoring. The proposed changes contained in this Application have been based on the model technical specifications provided with the NRC's October 7, 1981 and November 19, 1981 letters and will provide additional assurance that the Indian Point Unit No. 2 facility is operated within the limits determined appropriate following implementation of the TMI-2 Action Plan.

The proposed changes have been reviewed by both the Station Nuclear Safety Committee and the Consolidated Edison Nuclear Facilities Safety Committee. Both Committees concur that the proposed changes do not represent a significant hazards consideration and will not cause any change in the types or an increase in the amounts of effluents or any change in the authorized power level of the facility.

UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION

In the Matter of )  
 )  
CONSOLIDATED EDISON COMPANY ) Docket No. 50-247  
OF NEW YORK, INC. )  
(Indian Point Station, )  
Unit No. 2) )

STATE OF NEW YORK )  
 )  
COUNTY OF NEW YORK ) ss:

AFFIDAVIT OF SERVICE

Melissa B. Driscoll, being duly sworn, states: That she is an Assistant Engineer employed by Consolidated Edison Company of New York, Inc., and that she has served the foregoing document entitled, "Application for Amendment to Operating License," sworn to on December , 1981, by mailing a copy thereof, first class postage prepaid and properly addressed to the following person:

Hon. George V. Begany  
Mayor, Village of Buchanan  
236 Tate Avenue  
Buchanan, New York 10511

Melissa B. Driscoll  
Melissa B. Driscoll

Sworn to before me  
this 29 day December, 1981.

Thomas Love  
Notary Public

THOMAS LOVE  
Notary Public State of New York  
No. 31-2409638  
Qualified in New York County  
Commission Expires March 30, 1983