

ATTACHMENT A

APPLICATION FOR AMENDMENT
TO OPERATING LICENSE

Technical Specification
Page Revisions

Consolidated Edison Company of New York, Inc.

Indian Point Unit No. 2
Docket No. 50-247
Facility Operating License No. DPR-26
December, 1979

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6.3.2 The Department Manager shall meet or exceed the minimum qualifications specified for Plant Manager in ANSI N18.1-1971.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Director and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Nuclear Training Director and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976 with the exception of the training program schedule.

6.5 REVIEW AND AUDIT

6.5.1 STATION NUCLEAR SAFETY COMMITTEE (SNSC)

FUNCTION

6.5.1.1 The Station Nuclear Safety Committee shall function to advise the Department Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Station Nuclear Safety Committee shall, as a minimum, be composed as follows:

Chairman:	Plant Manager
Member:	Technical Engineering Director
Member:	Quality Assurance Engineer
Member:	Chief Operations Engineer
Member:	Security Supervisor
Member:	Test and Performance Engineer
Member:	Instrument and Control Engineer
Member:	Maintenance Engineer
Member:	Chemistry and Radiation Safety Director
Member:	Reactor Engineer
Member:	Department Manager
Member:	Refueling Engineer
Member:	Assistant Chief Field Engineer (Eng. Dept.)
Member:	QA Field Office Manager (QA & R Dept.)

6.5.1.2.1 In addition, other technically competent individuals may be appointed by the SNSC Chairman to serve as SNSC members.

ALTERNATES

6.5.1.3 Alternate members shall be appointed in writing by the SNSC Chairman to serve on a temporary basis.

MEETING FREQUENCY

6.5.1.4 The SNSC shall meet at least once per calendar month and as convened by the SNSC Chairman.

QUORUM

6.5.1.5 A quorum of the SNSC shall consist of the Chairman or Vice Chairman and a majority of the committee members including no more than two alternates.

RESPONSIBILITIES

6.5.1.6 The Station Nuclear Safety Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, and 2) any other proposed procedures or changes thereto as determined by the Department Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and preparation and forwarding of a report covering evaluation and recommendations to prevent recurrence to the Manager, Nuclear Power Generation Department and to the Chairman of the Nuclear Facilities Safety Committee.
- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and the issuance of reports thereon as required by the Chairman of the Nuclear Facilities Safety Committee.
- h. Review of the Plant Security Plan and implementing procedures and submission of recommended changes to the Chairman of the Nuclear Facilities Safety Committee.
- i. Review of the Emergency Plan and implementing procedures and submission of recommended changes to the Chairman of the Nuclear Facilities Safety Committee.

AUTHORITY

6.5.1.7 The Station Nuclear Safety Committee shall:

- a. Recommend to the Department Manager, in writing, approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and any changes to them shall be reviewed and approved for implementation in accordance with a written administrative control procedure approved by the Manager, Nuclear Power Generation Department, with the concurrence of the Station Nuclear Safety Committee and the Nuclear Facilities Safety Committee. The administrative control procedure required by this specification shall, as a minimum, require that:

- a. Each proposed procedure/procedure change involving safety related components and/or operation of same receives a pre-implementation review by the SNSC except in case of an emergency.
- b. Each proposed procedure/procedure change which renders or may render the Final Safety Analysis Report or subsequent safety analysis reports inaccurate and those which involve or may involve potential unreviewed safety questions are approved by the SNSC prior to implementation.
- c. The approval of the Nuclear Facilities Safety Committee shall be sought if, following its review, the Station Nuclear Safety Committee finds that the proposed procedure/procedure change either involves an unreviewed safety question or if it is in doubt as to whether or not an unreviewed safety question is involved.

6.8.3 A mechanism shall exist for making temporary changes and they shall only be made by approved management personnel in accordance with the requirements of ANSI 18.7-1972. The change shall be documented, and reviewed by the SNSC within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Region I Office of Inspection and Enforcement unless otherwise noted.

ATTACHMENT B

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Safety Evaluation

Consolidated Edison Company of New York, Inc.

Indian Point Unit No. 2

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Safety Evaluation

The proposed technical specification revisions, contained in Attachment A to this Application, would effect certain administrative changes relative to the Station Nuclear Safety Committee (SNSC). Specifically, the proposed revisions to Section 6.5 would add the Assistant Chief Field Engineer and the QA Field Office Manager, both of whom are located on-site, to the minimum required membership of the SNSC. In addition, specification 6.5.1.2.1 has been added to permit the SNSC Chairman to appoint other technically competent individuals beyond the minimum required by specification 6.5.1.2 to serve as SNSC members. Specification 6.5.1.5 has also been revised to make the requirements for a SNSC quorum consistent with the above changes and more generally applicable. Finally, the revision to specification 6.8.3 proposes to change the required time for SNSC review of temporary procedure changes from within 7 days to within 14 days of implementation of the temporary change(s). This change to specification 6.8.3 is consistent with the NRC's model Standard Technical Specifications (STS) for Westinghouse Pressurized Water Reactors.

The proposed changes do not in any way alter the safety analyses performed for Indian Point Unit No. 2. The proposed changes have been reviewed by the Station Nuclear Safety Committee and the Consolidated Edison Nuclear Facilities Safety Committee. Both committees concur that these changes do not represent a significant hazards consideration and will not cause any change in the types or increase in the amounts of effluents or any change in the authorized power level of the facility.