

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 736.8001



Joseph E. Russell
Resident Manager

February 25, 1991
IP3-91-015
DJC-91-013B

Docket No. 50-286
License No. DPR-64

Mr. James Linville, Chief
Reactor Projects Branch No. 1
Division of Reactor Projects
U.S. Nuclear Regulatory Commission Region 1
475 Allendale Road
King of Prussia, PA 19406

Subject: Inspection No. 50-286/90-20 and Associated Notice of Violation (90-20-01)

Dear Mr. Linville:

This letter and Attachment I provide the Authority's response to Inspection Report No. 50-286/90-20 and the Notice Of Violation 90-20-01.

Should you or your staff have any questions concerning this matter please contact Mr. M. Peckham.

Sincerely,



Joseph E. Russell
Resident Manager
Indian Point 3 Nuclear Power Plant

JER:DJC
Attachment

cc: Document Control Desk (original)
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Resident Inspector's Office
Indian Point 3 Nuclear Power Plant
U.S. Nuclear Regulatory Commission
P.O. Box 337
Buchanan, N.Y. 10511

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PDR ADOCK 05000286
PDR

Handwritten initials: JER

ATTACHMENT I

RESPONSE TO VIOLATION 90-20-01

VIOLATION:

10 CFR 50, Appendix B, Criterion III requires, in part, that measures be established for the selection and review for suitability of application of parts that are essential to the safety-related functions of systems and components.

Contrary to the above, the emergency diesel generator jacket water pressure switches, United Electric J17A Model 156, were replaced in December 1990 with United Electric J54A Model 156, a switch with unsuitable DC current carrying capacity for the application.

This is a Severity Level IV Violation (Supplement I).

RESPONSE:

The Authority has reviewed in detail the notice of violation outlined in Appendix A of the NRC Inspection Report 90-20 and agrees that the events occurred as described.

We have completed the following corrective actions:

- 1) The malfunctioning jacket water pressure switches were replaced.
- 2) The Authority engineered and completed a modification of the emergency diesel generator control circuit to provide additional assurance of jacket water pressure switch reliability. An intermediate relay between the jacket water pressure switch and the field flash circuit was installed to reduce the direct current to a value within the rating of the switch contacts.
- 3) A full functional test was performed on each emergency diesel generator to verify operability.
- 4) The Authority will change the Modification Control Program, by July 1, 1991, to ensure that component function is carefully considered during design review for component replacement. The Authority has made its design engineers aware of this event and has reaffirmed to them the necessity of adequately considering system application when procuring replacement components.