

U. S. ATOMIC ENERGY COMMISSION

DIRECTORATE OF REGULATORY OPERATIONS

REGION I

RO Inspection Report No: 50-286/74-11
Licensee: Consolidated Edison Company (Indian Point 3)
4 Irving Place
New York, New York 10003
Location: Buchanan, New York

Docket No: 50-286
License No: GPPR-62
Priority: _____
Category: B

Type of Licensee: PWR, 1050 MWe (Westinghouse)
Type of Inspection: Routine, Unannounced
Dates of Inspection: June 11-13, 1974
Dates of Previous Inspection: May 7-10, 1974

Reporting Inspector: S A Folsom / for
J, Allentuck, Reactor Inspector

6/26/74
Date

Accompanying Inspectors: S A Folsom
Seth A. Folsom, Reactor Inspector

6/26/74
Date

Date

Date

Date

Other Accompanying Personnel: None

8111110543 740626
PDR ADOCK 05000286
G PDR

Date

Reviewed By: R. F. Heishman
R. F. Heishman, Senior Reactor Inspector

6/26/74
Date

SUMMARY OF FINDINGS

Enforcement Action

- A. The failure of the constructor to require compliance with his site procedure, PS-597760 as it relates to controls for the clean area for the installation of upper internals is a violation of 10 CFR 50, Appendix B, Criterion V. (Details, Paragraph 3)
- B. A seismic restraint was located on the ground, with a pile of discarded spring-loaded hangers. This is a violation of 10 CFR 50, Appendix B, Criterion VIII. (Details, Paragraph 4)

Licensee Action on Previously Identified Enforcement Items

Not Applicable

Unusual Occurrences

None

Other Significant Findings

A. Current Findings

None

B. Status of Previously Reported Unresolved Items

- 1. The following previously reported outstanding items have been resolved:
 - a. Diagrammatic representations of 6.9 KV power system in the FSAR are compatible. (Details, Paragraph 5)
 - b. QA/QC documentation for spray nozzles are available. (Details, Paragraph 6)
 - c. Welds 1061 and 1062 have been repaired. (Details, Paragraph 7)
 - d. Required signatures have been included on QA/QC documents for installation of reactor coolant pump internals. (Details, Paragraph 8)

- e. QA/QC documentation for setting of reactor coolant pump casings indicate no apparent deficiencies. (Details, Paragraph 9)
 - f. Defective welds on diesel generator jacket water piping have been repaired. (Details Paragraph 10)
 - g. There are no requirements for QC Records for valves 505 A&B. (Details, Paragraph 11)
 - h. Constructor's audit of field-run tubing indicates no apparent defects. (Details, Paragraph 14)
 - i. Licensee (corporate) QA has accepted licensee (site) response to audit findings. (Details, Paragraph 15)
 - j. The licensee has revised the format of IPR3051 to provide for controlled distribution. (Details, Paragraph 16)
 - k. The licensee has revised the format for "Field Weld Records Audit Checklist." (Details, Paragraph 12)
 - l. Licensee has reinspected electrical terminations made prior to the implementation of WQA 4-0-5. (Details, Paragraph 13)
 - m. IPP test procedures for Foxboro instrumentation are available at the site and have been implemented. (Details, Paragraph 18)
 - n. Plugs in weld channel pressurization system have either been replaced where required, or openings protected. (No Details, See RO Report 50-289/74-09)
 - o. Check-lists associated with WEGR-0-44 include appropriate signature blocks. (Details, Paragraph 19)
2. The following previously unresolved item remains unresolved:
- a. Missing plugs and broken connections at the seal table. (Details, Paragraph 17)

Management Interview

An exit interview was conducted at the site with the following individuals:

Consolidated Edison Co

D. L. Hartsfield, Superintendent, Construction
G. I. Coulbourn, Mgr, IP3 Construction
J. S. White, QA Project Engr (HQ)
J. P. Dean Supervising QA Examiner
W. L. Geider, Chief Construction Inspector
H. W. Cairns, Chief Construction Inspector

WEDCO

M. L. Snow, Reliability Manager
S. R. Buckingham, Quality Assurance Manager
B. W. Garrow, Quality Assurance Engineer

- A. The inspector stated that the following previously outstanding items had been resolved:
1. Diagrammatic representations of 6.9 KV power system in the FSAR are compatible. (Details, Paragraph 5)
 2. QA/QC documentation for spray nozzles are available. (Details, Paragraph 6)
 3. Welds 1061 and 1062 have been repaired. (Details, Paragraph 7)
 4. Required signatures have been included on QA/QC documents for installation of reactor coolant pump internals. (Details, Paragraph 8)
 5. QA/QC documentation for setting of reactor coolant pump casings indicate no apparent deficiencies. (Details, Paragraph 9)
 6. Defective welds on diesel generator jacket water piping have been repaired. (Details, Paragraph 10)
 7. There are no requirements for QC Records for valves 505 A&B. (Details, Paragraph 11)
 8. Constructor's audit of field-run tubing indicates no apparent defects. (Details, Paragraph 14)
 9. Licensee HQ QA has accepted licensee site response to audit findings. (Details, Paragraph 15)

10. The licensee has revised the format of IPR3051 to provide for controlled distribution. (Details, Paragraph 16)
 11. The licensee has revised the format for "Field Weld Records Audit Checklist". (Details, Paragraph 12)
 12. The licensee has reinspected terminations made prior to the implementation of WQA 4-0-5. (Details, Paragraph 13)
 13. IPP test procedures for Foxboro instrumentation are available at the site and have been implemented. (Details, Paragraph 18)
 14. Plugs in weld channel pressurization system have either been replaced where required or openings protected. (No Details, See RO Report 50-289/74-09)
 15. Appropriate signature blocks are included in WEGR 0-44 checklists.
- B. The inspector stated that plugs at the seal table were still missing and that tube connections had been broken. The licensee stated that these matters would be corrected promptly. This matter is unresolved. (Details, Paragraph 17)
- C. The inspector stated that the following were violations of AEC requirements:
1. The failure of the constructor to require compliance with his site procedure, PS-597760 as it relates to controls for the clean area for the installation of upper internals is a violation of 10 CFR 50, Appendix B, Criterion V. (Details, Paragraph 3)
 2. A hydraulic seismic restraint was located on the ground, with a pile of discarded spring loaded hangers. This is a violation of 10 CFR 50, Appendix B, Criterion VIII. (Details, Paragraph 4)

DETAILS

1. Persons Contacted

Consolidated Edison Co

D. L. Hartsfield, Superintendent, Construction
G. I. Coulbourn, Manager, IP3 Construction
J. S. White, QA Project Engr (HQ)
J. P. Dean, Supervising QA Examiner
W. L. Beider, Chief Construction Inspector
H. W. Cairns, Chief Construction Inspector

WEDCO

M. L. Snow, Reliability Manager
S. R. Buckingham, Quality Assurance Manager
J. Smart, Quality Assurance Engineer
B. W. Garrow, Quality Assurance Engineer

2. Status of Construction

The licensee is re-evaluating the status of construction. This information will not be available until mid-July 1974.

3. Clean Area Control Procedures

The following was a violation of the licensee site requirements for clean area controls as referred to in PS-597760, the procedure for the installation of the upper internals:

- a. There was no sign posted at the entrance to the clean area prohibiting certain specified activities.
- b. A craftsman was observed in the clean area without a cap.
- c. A candy wrapper and a beverage can were observed in the clean area.

The above were noted by the inspector while installation activities were in progress.

4. Seismic Restraints Uncontrolled

The inspector observed a seismic restraint, untagged, lying on

the ground on the ramp leading to the vapor containment equipment hatch among discarded spring-loaded pipe hangers. This is a violation of 10 CFR 50, Appendix B, Criterion VIII.

5. 6.9 KV Power System

The licensee has revised FSAR Dwg. 8.2-2 and as it appears in Supplement 26, it is compatible with FSAR Dwg 8.2-3, Supplement 23. These now adequately describe the connections to the gas turbine units.

6. Spray Nozzles

The licensee has performed a 100% audit of QA/QC documentation of spray nozzles. Each nozzle was inspected so that serial numbers might be associated with required inspection certificates. This audit is documented in the licensee's internal memorandum dated May 21, 1974. There were no apparent deficiencies.

7. Welds 1061 and 1062

Required repairs to Weld 1061 and 1062 had been the subject of OIR 3P-136 and 3P-137 respectively. The inspector examined Field Weld Repair Records and Radiograph Inspection Reports all dated August 2, 1973 for these repairs. There were no apparent deficiencies.

8. Reactor Coolant Pump Internals

The inspector examined QA/QC documents which included the appropriate signature for hold points Step If and Step IId. The inspector examined records for the following internals:

S/N 2-618 J 711-G01
S/N 3-618 J 711-G02
S/N 3-618 J 711-G01

There were no apparent deficiencies.

9. Reactor Coolant Pump Casings

The inspector examined the data included in "Level Readings on Seal Surface of RC Pumps" which covered measurements made as follows:

<u>Pump</u>	<u>Date</u>
#31	1/23/73
#32	6/1/73
#33	1/17/73
#34	6/1/73

Acceptance criteria for these measurements are included in Westinghouse Dwg. No. 618J710. There were no apparent deficiencies.

10. Diesel Jacket Water Piping

The inspector examined OIR-3-P-222 which documented the buildup of welds on the diesel jacket water piping system accepted on February 12, 1974. There were no apparent deficiencies.

11. QC Records for Valves No. 505 A&B

Valve Nos. 505 A&B were received at the site prior to July 27, 1970 and hence the requirement that quality documents be maintained at the site is not applicable.

12. Field Weld Records Audit Checklist

The format of the "Field Weld Records Audit Checklist" has been revised to eliminate the requirement that the auditor include in his review certain welded joints which had been covered by WEDCO audits. This matter is resolved.

13. Electrical Termination Inspection

The constructor has reinspected the electrical terminations made prior to the implementation of WQA-4-0-5 for termination integrity, and WQA 4-0-6 for separations requirements. The inspector examined the re-inspection check-lists for the following cables:

GE-1WV7/2
AH-9M42
AH-3AI9/1
GE-1WV7/1

There were no apparent deficiencies.

14. Field Run Tubing

The inspector examined WEDCO Audit Report 5-112C for Rack 20 Class I

instrument tubing dated May 16, 1974, and noted no deficiencies.

15. Con Ed (HQ) Audit

The inspector examined a Con Ed (HQ) audit report of site activities dated June 12, 1974 which indicated acceptance of Con Ed (site) responses to outstanding items in the licensee's audit report relating to Weld/NDE dated March 7, 1974.

16. Controlled Distribution - IPR 3051

The inspector examined the revised IPR 3051, Section IX. Distribution and revision of these instruction has been added to effect the desired controls.

17. Seal Table - Connections

The inspector examined the seal table at Elevation 65'. Plugs were missing from tubing and certain tubing connections had been broken. Absence of required plugs was reported in RO Report No. 50-286/74-09.

18. Foxboro Instruemtns - Test Procedures

The inspector examined WEDCO letter EUP-930 which states in part "...IPP test procedures is described in Sect 2.23 of... Foxboro INT, WNES. System Manual" The inspector, in addition, examined Foxboro data sheet SP-11, Steam Generator Loops 1, 2, 3, 4, Racks 10, 18, 25 and noted no deficiencies.

19. WEGR-0-44 Sign-Off

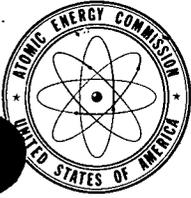
The inspector examined check-off sheets in WEGR-0-44 and noted that appropriate signature blocks were included.

HDT (AICF)

This problem
has been corrected
by Chuck Fitzgerald
after considerable discussion
This I believe I
discussed with you
at the time - The
original procedure
established resulted
in this kind of
screw-up -

FAD.

How come all the
copies?



UNITED STATES
ATOMIC ENERGY COMMISSION
WASHINGTON, D.C. 20545

OCT 2 1974

DOG
PAD
What's this all
abt.?
[Signature]

Note to H. D. Thornburg

RO INSPECTION RPT. NO. 50-286/74-12, CEC, IPS-3
(MEMO, E.J.BRUNNER TO H.D.THORNBURG, DTD. 8/15/74)

A review of the subject memorandum and related documents prompts the following comments:

1. The subject memorandum, dated August 15, 1974, was forwarded to the Facility Inspection Branch (FIB) as A/I H00255H1 on September 24, 1974.
2. The subject memorandum was also forwarded to the FIB as A/I H00172H1 on August 23, 1974.
3. A FIB memorandum, copy enclosed, was prepared and forwarded to DL on September 5, 1974. It is noted that both the FSEB and RO:I were listed on the distribution of this action memorandum.
4. Based on the above, it appears that further action on the part of the FIB is not warranted at this time.

As a general comment, we recommend that a control system be initiated to minimize or prevent the preparation of duplicate requests for the same action item.

J. H. Sniezek
J. H. Sniezek, RO

Enclosure:
Cy, memo dtd 9/5/74.

cc w/o encl:
JGDavis, RO (H00255H1)
BHGrier, RO (C&O:653)

10/9/74
not entered into systems
(H0255H1) however, closed out
or previous AI CF.
Ref 1. H00172H1
2. ~~H00172H1~~ F1.4003H0
[Signature] 10/9/74

SEP 5 1974

J. S. Vassallo, Chief, LWR Project Branch 1-1, DL

CONSOLIDATED EDISON COMPANY - INDIAN POINT 3 - DOCKET NO. 50-286

Enclosed for your consideration is a copy of RO inspection report No. 50-286/74-12 relating to a deficiency in the licensee's proposed pre-operational testing program for the emergency core cooling system (ECCS). As discussed in the report, the licensee does not plan to test the accumulator isolation valves under maximum differential pressure conditions in accordance with Regulatory Guide 1.79, paragraph C.3.c.(2). The failure to test these valves under maximum differential pressure conditions appears to be contrary to the Regulatory policy established in a memorandum, A. Giambusso to B. H. Grier, dated December 12, 1973, copy enclosed for your information. We recommend that the licensee be required to comply with the position stated in Mr. Giambusso's memorandum.

Details of this matter have been discussed between F. Nolan, RO and N. Aycock of your staff. If you require additional information or wish to discuss this matter further, please contact F. Nolan or me.

Original signed by *for*

F. J. Nolan

J. H. Sniezek, Acting Chief
Facility Inspection Branch, RO

Enclosures:

1. Insp. Rpt. No. 50-286/74-12
2. Memo, AGiambusso to BHGrier, dtd 12/12/73

cc w/encs:

RO Files (DN 50-286)
N. Aycock, DL

cc w/o encs:

B. H. Grier, RO (CSD:809)
H. D. Thornburg, RO (H00172H1) 8/23/74
E. J. Brunner, RO:I

RO/mib,7451 RO:FIb:AC

FJNolan

JHSniezek

9/4/74

9/ /74



UNITED STATES
 ATOMIC ENERGY COMMISSION
 DIRECTORATE OF REGULATORY OPERATIONS
 REGION I
 631 PARK AVENUE
 KING OF PRUSSIA, PENNSYLVANIA 19406

AUG 15 1974

H. D. Thornburg, Chief, Field Support and Enforcement Branch
 Directorate of Regulatory Operations, HQ

RO INSPECTION REPORT NO 50-286/74-12
 CONSOLIDATED EDISON COMPANY
 LICENSE NO. CPPR-62

The subject report is forwarded for action.

Paragraph 4.b of the Details section of this report addresses itself to the preoperational testing of the Emergency Core Cooling System (ECCS) at Indian Point 3. The extent of the testing included in the procedures reviewed by RO:I does not include testing of the accumulator isolation valves under maximum differential pressure conditions in accordance with Regulatory Guide 1.79, Paragraph C.3.c(2).

The licensee stated that he is not committed to Regulatory Guide 1.79 and does not feel this test is technically justified.

Failure to test the ECCS accumulator isolation valves under the differential pressure conditions under which they may be required to open does not appear to provide adequate assurance of operability. Also no alternative means of proving the valves capability was identified by the licensee, and no intent to modify the Technical Specifications to require this test has been identified.

It is requested that specific definition of the necessity for proving operability of the ECCS accumulator isolation valves be provided to RO:I and the licensee.

EJ Brunner
 Eldon J. Brunner, Chief
 Reactor Operations Branch

TRACK # F14003H0

OFFICE ▶	GRESS	<i>W. Kelly</i>	<i>W. Hannon</i>	<i>W. Davis</i>	<i>E. J. Brunner</i>	<i>F. ...</i>
SURNAME ▶	Stemberg/by	<i>W. Kelly</i>	Hannon	Davis	Brunner	F...
	8/14/74		8/14/74	8/14/74	8/10/74	9



UNITED STATES
ATOMIC ENERGY COMMISSION
DIRECTORATE OF REGULATORY OPERATIONS
REGION 1
631 PARK AVENUE
KING OF PRUSSIA, PENNSYLVANIA 19406

AUG 15 1974

H. D. Thornburg, Chief, Field Support and Enforcement Branch
Directorate of Regulatory Operations, HQ

RO INSPECTION REPORT NO 50-286/74-12
CONSOLIDATED EDISON COMPANY
LICENSE NO. CPPR-62

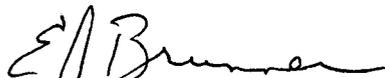
The subject report is forwarded for action.

Paragraph 4.b of the Details section of this report addresses itself to the preoperational testing of the Emergency Core Cooling System (ECCS) at Indian Point 3. The extent of the testing included in the procedures reviewed by RO:I does not include testing of the accumulator isolation valves under maximum differential pressure conditions in accordance with Regulatory Guide 1.79, Paragraph C.3.c(2).

The licensee stated that he is not committed to Regulatory Guide 1.79 and does not feel this test is technically justified.

Failure to test the ECCS accumulator isolation valves under the differential pressure conditions under which they may be required to open does not appear to provide adequate assurance of operability. Also no alternative means of proving the valves capability was identified by the licensee, and no intent to modify the Technical Specifications to require this test has been identified.

It is requested that specific definition of the necessity for proving operability of the ECCS accumulator isolation valves be provided to RO:I and the licensee.


Eldon J. Brunner, Chief
Reactor Operations Branch



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