

U. S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
REGION I

IE Inspection Report No: 50-286/75-10

Docket No: 50-286

Licensee: Consolidated Edison Company of New York, Inc.

License No: CPPR-62

Indian Point 3

Priority: _____

Category: B1

Safeguards
Group: _____

Location: Buchanan, New York

Type of Licensee: PWR, 1050 MWe (W)

Type of Inspection: Routine, Announced

Dates of Inspection: April 23-25, 1975

Dates of Previous Inspection: April 8-9, 1975

Reporting Inspector: *J.T. Smith for*
D. F. Johnson, Reactor Inspector

5-9-75
DATE

Accompanying Inspectors: *A. N. Fasano*
A. N. Fasano, Reactor Inspector

5-9-75
DATE

DATE

DATE

Other Accompanying Personnel: None

DATE

Reviewed By: *E.C. McCabe Jr*
E. C. McCabe, Jr., Senior Reactor Inspector
Nuclear Support Section, Reactor Operations Branch

5/9/75
DATE

SUMMARY OF FINDINGS

Enforcement Action

None

Licensee Action on Previously Identified Enforcement Items

Item of Noncompliance, IE:I Inspection Report 50-286/74-07

The licensee's corrective actions with respect to the above referenced report were reviewed by the inspector and found to be completed.
(Detail 4.a)

Other Significant Findings

A. Current Findings

1. Acceptable Areas

The following areas were inspected on a sampling basis. Findings did not involve an Item of Noncompliance, Deviation or Unresolved Item.

- a. Administrative controls for revising, changing, review, approval and upgrading of plant procedures. (Detail 5)
- b. Format and content of facility procedures. (Detail 6)
- c. Scope of procedural coverage. (Detail 7)
- d. Administrative Procedures for procedure control, issuance and retention. (Detail 5)
- e. RCC pseudo ejection and RCC out of position, INT-TP-8.2. (Detail 8.b)
- f. Shutdown from outside the control room, INT-TP-11.1. (Detail 8.c)
- g. Unit trip and blackout at power, INT-TP-11.0. (Detail 8.d)

2. Unresolved Items

More information is required to determine the status of the following items.

- a. Preventive Maintenance Program. (Detail 2)
- b. Completion status of facility procedures program. (Detail 3)

B. Status of Previously Reported Unresolved Items

1. Facility Procedures

All previously identified unresolved items relative to facility procedures have been resolved. (Detail 4)

2. Test Procedures

The following test procedure items were resolved.

- a. Containment Integrity, INT-TP-5.2, "Prerequisite and Periodic Check Offs for Core Loading." (Detail 9)
- b. Unit 3 X-Y Xenon Stability Test. (Detail 10)
- c. Emergency Boration, INT-TP-5.1, "Initial Core Loading." (Detail 11.a)
- d. Reciprocal Multiplication Plots, INT-TP-5.1, "Initial Core Loading." (Detail 11.b)
- e. Control Rod Drop Test, INT-TP-4.9.5, "Rod Timing Checks." (Detail 12)
- f. Nuclear and Δ T Calibration and Thermal Power Measurement Procedure. (Detail 13)
- g. Predicted Boron Concentration and Rod Position at Criticality, INT-TP-7.2, "Initial Criticality." (Detail 14.a)
- h. I/M Plot Interpretation, INT-TP-7.2, "Initial Criticality." (Detail 14.b)
- i. Initial Criticality SUR Limit, INT-TP-7.2, "Initial Criticality." (Detail 14.c)
- j. Boron Dilution Rate. INT-TP-7.2, "Initial Criticality." (Detail 14.d)
- k. Manual Scram Test, INT-TP-7.2, "Initial Criticality." (Detail 14.e)

1. Procedural Details, INT-TP-7.2, "Initial Criticality.
(Detail 14.f)
- m. Acceptance Criteria for Power Coefficient of Reactivity,
INT-TP-8.1, "Power Level Change." (Detail 15)
- n. Motor Operated Accumulator Isolation Valve Testing.
(Detail 16)
- o. Turbine Overspeed Limit, INT-TP-9.7, "Generator Load Trip."
(Detail 17)
- p. SAO-104, Maintenance Work Request Procedure. (Detail 18)

Management Interview

A management meeting was held at the Indian Point Power Station on April 25, 1975, at the conclusion of the inspection.

Personnel in Attendance

Consolidated Edison Company

Mr. S. H. Cantone, Chief Engineer
Mr. W. Josiger, Start-up Test Engineer
Mr. S. S. Zulla, Operations Engineer, Unit 3

Summary of Items Discussed

1. Preventive Maintenance Program. (Detail 2)
2. Completion status of facility procedures program. (Detail 3)
3. Previously identified unresolved items and review of Item of Noncompliance. (Details 4, 10-19)
4. Administrative Controls for revising, changing, review, approval and upgrading of plant procedures. (Detail 5)
5. Facility procedure format and content. (Detail 6)
6. Scope of procedural coverage. (Detail 7)
7. Administrative procedures for procedure control, issuance and retention. (Detail 5)

DETAILS

1. Persons Contacted

Mr. S. H. Cantone, Chief Engineer
Mr. C. M. Caputo, Associate Engineer Test
Mr. W. Josiger, Start-up Test Engineer
Mr. L. J. Kawula, Test Engineer
Mr. J. Makepeace, Technical Operations Director
Mr. W. A. Monti, Plant Engineer
Mr. V. Perry, Site Construction Superintendent
Mr. J. J. Vignola, Preventive Maintenance Engineer
Mr. S. S. Zulla, Operations Engineer, Unit 3

2. Preventive Maintenance Program

a. Preventive Maintenance Procedures

The licensee provided the inspector with draft copies of proposed preventive maintenance procedures for his review.

Findings from review of the above procedures indicated discrepancies between the licensee's procedures and the format and content described in ANSI N 18.7, "Administrative Controls for Nuclear Power Plants."

In addition, the proposed preventive maintenance procedures were not in accordance with requirements of the licensee's Administrative Directive PE-AD-10, "Preventive Maintenance Program."

In a subsequent telephone conversation with the licensee on April 30, 1975, the licensee stated the draft preventive maintenance procedures were provided to the inspector erroneously, in that these procedures were not intended to be implemented in their present format and content. The procedures were merely submitted to the Plant Engineer as recommendations for implementation into the preventive maintenance program. The licensee further stated that, if utilized, the procedures would be revised, reviewed and approved in accordance with Administrative Directive PE-AD-10.

This item is unresolved pending licensee action on implementation of the Preventive Maintenance Program for Unit 3.

b. Preventive Maintenance Schedule

The inspector stated that a maintenance program should be developed to maintain safety related structures, systems and components at the quality required for them to perform their intended function. This program should contain general rules for the development of procedures, schedules and administrative controls that prescribe the format, content, review and approval for detailed maintenance procedures. This program should be completed and implemented prior to startup.

The licensee concurred with the inspector's comments and stated that existing maintenance procedures and Administrative controls for the Preventive Maintenance Program for Unit 2 will be used for Unit 3. Minor changes such as equipment designations, unit code numbers will be made and a schedule will be prepared. This interim program for Unit 3 will be completed and implemented prior to licensing.

This item is unresolved pending implementation of licensee action.

3. Facility Procedures Program Status

a. Completion Status

Information obtained during this inspection indicated the following status of completion regarding the Facilities Procedures Program.

	<u>% Written</u>	<u>% Approved</u>
Plant Operating Procedures	100	100
Plant Operating Instructions	100	100
System Operating Procedures	100	80
Preventive Maintenance Procedures	100	0
Emergency Procedures	100	80
Surveillance Test and Calibration Procedures	100	50
Chemistry Procedures	100	100
Health Physics Procedures	100	100
Administrative Procedures	100	100
Alarm Procedures	100	70

The following procedures remain to be approved.

b. Procedures to be Approved

(1) System Operating Procedures

- (a) SOP-CB-2 Containment Entry and Egress
- (b) SOP-CB-3 Containment Pressure Relief and Purge System
- (c) SOP-CC-1B Component Cooling System Operation
- (d) SOP-CM-3 Spent Fuel Pit Bldg Operation
- (e) SOP-CS-1 Containment Spray System Operation
- (f) SOP-CVCS-2 Charging, Seal Water and Letdown Control
- (g) SOP-EL-2 Instrument Bus Static Inverter Operation
- (h) SOP-EL-3 Battery Charger Operation
- (i) SOP-FW-4 Auxiliary Feedwater and Condensate System
- (j) SOP-PW-1 Primary Water System Operation
- (k) SOP-RCS-2 Pressurizer Pressure Control
- (l) SOP-RHR-1 Residual Heat Removal System Operation
- * (m) SOP-RP-8 Removal and Insertion of Movable and Fixed
In-Core Flux Mapping System Thimbles
- * (n) SOP-RP-10 Irradiation Sample Removal
- * (o) SOP-RP-11 Guide Tube Cover Removal and Replacement
- * (p) SOP-RP-12 Lower Internals Removal and Replacement
- * (q) SOP-RP-13 Guide Stud (Alignment Pin) Installation and Removal
- * (r) SOP-RP-18 Stud Tensioning and Detensioning
- * (s) SOP-RP-19 Full Length Rod Control Cluster Uncoupling
- (t) SOP-RPC-1 Reactor Protection System Operation
- * (u) SOP-SFP-1 Spent Fuel Pit Cooling and Purification System
Operation
- (v) SOP-SI-1 Safety Injection System Operation
- (w) SOP-TG-4 Turbine Generator Startup, Synchronizing Voltage
Control, and Shutdown
- (x) SOP-WDS-1 Liquid Waste Disposal System Operation
- (y) SOP-WDS-6 Liquid Waste Discharge Procedure
- (z) SOP-WDS-7 Gaseous Waste Discharge Procedure

* Refueling Procedures

(2) Plant Emergency Procedures

- (a) PEP-CB-1 Loss of Containment Integrity
- (b) PEP-CC-2 Leakage into the Component Cooling System
- (c) PEP-CVCS-1 Loss of Charging or Letdown
- (d) PEP-CVCS-2 Reactor Coolant Makeup Control System Malfunction
- (e) PEP-EL-1 Loss of Outside Power
- (f) PEP-FW-3 Feedwater Heater Tube Leak

- (g) PEP-RC-2 Rod Control System Malfunction
- (h) PEP-RCS-2 Malfunction of Pressurizer Pressure Control
- (i) PEP-SG-1 Steam Generator Tube Leak

(3) Alarm Procedures

- (a) SBF-1 Safeguards
- (b) SBF-2 Safeguards
- (c) SCF- Condensate and Boiler Feed
- (d) SLE- Weld Channel
- (e) SOF- Electric Heat Tracing and Deluge
- (f) Local- Steam and Water Analysis
- (g) Local- Waste Disposal - Boron Recycle
- (h) Local- Waste Evaporation - WDS

(4) Surveillance Test and Calibration Procedures*

- (a) 3PT-V1 Source Range
- (b) 3PT-V2 Intermediate Range
- (c) 3PT-V3 RCS Leakage
- (d) 3PT-V4 SF/FWF Mismatch Analog
- (e) 3PT-V5 Steam Generator Secondary Hydro
- (f) 3PT-V7A BFP Turbine Mechanical Trip
- (g) PC-M1 Axial Offset Calibration
- (h) 3PT-M1 Nuclear Power Range
- (i) 3PT-M2 OTAT and OPAT
- (j) 3PT-M14 Safety Injection System
- (k) 3PT-M16 S.I. Pump
- (l) 3PT-M18 RHR Pump
- (m) 3PT-M-20 Motor Driven ABFP
- (n) 3PT-M-22 Diesel Generator
- (o) 3PT-M-31 Instrument Air
- (p) 3PT-Q-2 Process Radiation Monitor
- (q) 3PT-SA2 Post Accident Cont. Atmo. Sampling

* Does not include refueling frequency test requirements.

4. Previously Reported Unresolved Items and Items of Noncompliance

a. Record Retention - Report 50-286/74-07 paragraph 7

The licensee's administrative procedures did not specify the individual(s) assigned the responsibility for maintaining records required by Technical Specifications.

The inspector verified, by review of Unit 3 Procedure 3AD-20 "Document Distribution and Retentions," that responsibility for record retention had been assigned to the Chief Engineer.

This item is resolved.

b. Review and Approval of Facility Procedures - Report 50-286/74-07 paragraph 2

The licensee's administrative controls for review and approval of safety related facility procedures were not in accordance with the requirements of 10 CFR 50, Appendix B, Criterion II.

The inspector verified, by review, that Station Administrative Order 102 has been revised to include requirements for a pre-implementation review of procedures/procedure changes involving safety related components and/or operation of same by the Station Nuclear Safety Committee (SNSC).

This item is resolved.

5. Administrative Controls

The inspector performed an audit, on a sampling basis, of the following procedures.

SAO-102 Procedure/Procedure Change Approval Policy
3AD-10 Document Review
3AD-14 Preparation of Procedures
3AD-19 Procedure/Procedure Change Approval Policy
3AD-20 Document Distribution and Retention
TEAD-2 Test Procedure Development and Approval Process
TEAD-5 Surveillance Test Procedure Issuance and Review Process
PE-AD10 Preventive Maintenance Program
PE-AD4 Procedures for Performing Corrective Maintenance

The inspector's findings, based on the review of the above procedures, verified that the licensee has administrative controls for changing, periodic review, revision, initial review, approval and updating of facility procedures, including temporary changes thereto.

No inadequacies were identified with the above procedures with respect to the requirements of Technical Specifications Section 6, "Administrative Controls," FSAR Chapter 12, "Conduct of Operations," and ANSI N18.7, "Administrative Controls for Nuclear Power Plants.

The inspector had no further questions on these items.

6. Format and Content of Facility Procedures

The inspector conducted an audit of the facility procedures program by a sampling review of the following procedures.

a. Administrative Procedures

- SAO-100 Station Administrative Order Policy
- SAO-101 Sub Section Management Policy
- SAO-103 Management Techniques and Control
- SAO-104 Maintenance Work Request Procedures
- SAO-105 Work Permits
- SAO-115 Recall of Standby Personnel
- SAO-123 Containment Entry Policy
- 3AD-1 Administrative Directive Policy
- 3AD-10 Document Review
- 3AD-17 General Administrative Procedures
- 3AD-23 Watch Relief
- 3AD-27 Conduct of Operations
- 3AD-28 Procedure Adherence

b. Plant Operating Procedures

- POP-1 Plant Startup from Cold Shutdown Condition to Hot, Critical Zero Power

- POP-2 Reactor Startup from Subcritical Condition to Critical, Zero Power Condition

c. System Operating Procedures

- SOP-CB-2 Containment Entry and Egress
- SOP-CC-1 Component Cooling System Operation
- SOP-CVCS-3 Reactor Coolant Boron Concentration Control
- SOP-NI-1 Excore Nuclear Instrumentation System Operation
- SOP-RCS-3 Pressurizer Level Control
- SOP-RM-2 Process Radiation Monitoring System Operation
- SOP-RPC-1 Reactor Protection System Operation
- SOP-RHR-1 Residual Heat Removal System Operation
- SOP-RC-1 Full Length Rod Control System Operation
- SOP-SG-1 Steam Generator and Blowdown System Operation
- SOP-WDS-1 Liquid Waste Disposal System Operation

d. Plant Emergency Procedures

PEP-C-1 Loss of Condenser Vacuum
PEP-CVCS-3 Emergency Boration
PEP-1A-1 Loss of Instrument Air
PEP-RCS-1 Loss of Reactor Coolant Flow
PEP-RPC-2 Control Room Inaccessibility

Alarm Procedures

SAF Reactor Coolant System
SFF Chemical and Volume Control System
SMF Safety Injection

e. Surveillance Test and Calibration Procedures

3PT-V6 Turbine Generator Trip
3PT-D1 Diesel Generator
3PT-BW1 Control Rod Exercise
3PT-M3 Reactor Coolant Flow
3PT-M5 Pressurizer Pressure
3PT-M14 Safety Injection System
3PT-Q3 Area Radiation Monitoring
3PT-Q7 Diesel Generator Overspeed Trip
3PT-A6 Charging Pump
3PT-R-17 Main Steam Stop Valves
3PT-R29A Station Battery Load

Findings from review of the above procedures indicate no discrepancies between the licensee's procedures and the format and content described in ANSI N18.7 "Administrative Controls for Nuclear Power Plants."

The inspector had no further questions in this area.

7. Scope of Procedural Coverage

Based upon review of procedure indices, examination of records, and discussions with the licensee, the inspector found no inadequacies in procedural scope or in conformance with Chapter 12 of the FSAR, Section 6 of the Technical Specifications, or discrepancies with the activities listed in Regulatory Guide 1.33, Appendix A.

The inspector had no further questions in this area.

8. Procedure Reviews

a. Scope

Procedures listed in this report detail (8) were reviewed to verify the following:

- (1) Standard procedure requirements;
- (2) Predicted values;
- (3) Precautions and limitations stipulated;
- (4) Initial condition established; and
- (5) Test conditions stipulated and found to be satisfactory.

b. RCC Pseudo Ejection and RCC Out of Position INT-TP-8.2

The purpose of this procedure is to measure hot channel factors with individual rods withdrawn to the out of bank position and to verify that the values measured are within expected limits.

c. Shutdown From Outside the Control Room INT-TP-11.1

The purpose of this procedure is to demonstrate that the plant can be safely shutdown and controlled from outside the control room.

d. Unit Trip and Blackout at Power INT-TP-11.0

The purpose of this test is to demonstrate satisfactory operation of the Emergency Power system following a Unit Trip combined with loss of incoming 138 KV power during unit operation equal to or greater than 10% generator output.

9. Containment Integrity, INT-TP-5.2, Prerequisite and Periodic Checkoff for Core Loading Approved 4/17/75

Reference: IE:I Report No. 50-286/75-03, Detail, Paragraph 4.b.(3)

The purpose of this test is to provide a list of prerequisite conditions which must be satisfied prior to the start of initial core loading; provide a list of conditions which must be verified at intervals of approximately 12 hours during the initial core loading and to provide a list of boron concentration samples that should be obtained at approximately 4 hour intervals during initial core loading.

Page 7, step 5.1.28 of TP-5.2 specifies that at least one isolation valve shall be operable or locked closed in each line penetrating containment which provides a direct path for containment atmosphere to the outside. This item is resolved.

10. Unit 3 X-Y Xenon Stability Test

Reference: IE:I Report No. 50-286/75-03 Detail, Paragraph 3.b.

Upon further evaluation it was determined that there is no requirement for conduct of the X-Y Xenon stability test at Indian Point 3. This item is resolved.

11. Initial Core Loading, INT-TP-5.1, Approved 4/2/75

The purpose of this procedure is to establish the conditions under which installation of the nuclear fuel charge is to be accomplished and to specify the sequence of events which constitute the initial core loading program.

- a. Emergency Boration (Reference IE Report No. 50-286/75-03, Detail, Paragraph 4.b.(4))

Reference page 2, step 3.3.9 of TP-5.1 which contains reference to "Emergency Boration," PEP-CVCS-3, Revision 0, approved 11/16/74. The emergency instruction describes the operations required to rapidly inject boric acid into the Reactor Coolant System. This item is resolved.

- b. Reciprocal Multiplication Plots (Reference IE Report No. 50-286/75-03, Detail, Paragraph 4.b.5.(b))

Reference page 4, steps 4.6, 4.7, 4.8 and 4.10 of TP-5.1 which address the count times, plots and requirements for the interpretation of the I/M plots which resolve the concern reported in 75-03. This item is resolved.

12. Control Rod Drop Test, INT-TP 4.9.4, "Rod Timing Checks" Approved 3/24/75

Reference: Report No. 50-286/75-03, Detail, Paragraph 4.d.(1)

Reference steps 5.5 and 5.6 of TP 4.9.5 which states in part:--select the rod having the fastest and slowest drop times and in addition any rod that exceeds 2.6σ (where σ is the standard deviation) in the hot full flow condition. Repeat the drop time measurements ten times

while maintaining plant conditions as in step 5.4 (hot $540^{\circ}\text{F} \pm 10^{\circ}$) full flow. Select the rod having the slowest and fastest drop time and in addition any rod that exceeds 2.6σ in the cold no flow condition and repeat the drop time measurement ten times in the cold no flow condition. This item is resolved.

13. IPP-SU-8.6, Nuclear and ΔT Calibration and Thermal Power Measurements Procedure Revision 3 July 23, 1973

Reference: IE Report No. 50-286/75-03, Detail, Paragraph 4.d.(2)(b)

The IE inspector reviewed the current IP-3 procedures for procedural organization. The program for IP-2, IPP-SU-8.6, has been changed and includes three separate procedures for IP-3. These procedures are titled: "Thermal Power Measurement" INT-TP-8.5.1 approved 4/20/75; "Operational Alignment of Nuclear Instrumentation" IWT-TP-8.5.2 approved 4/20/75, and Operating Alignment of Process Temperature Instrumentation" INT-TP-8.5.3, approved 4/20/75.

The purpose of TP-8.5.1 is to describe the method for determining core thermal power for use in other startup tests as required.

The purpose of TP-8.5.2 is to allow for a general alignment checkout of nuclear instrumentation prior to criticality and during power escalation. The determination of overlap ranges and the setting of correspondence of current to core power is addressed as is the setting of trip points and checking of flux deviations.

The purpose of TP-8.5.3 is to align the ΔT and Tav_g instrumentation under isothermal conditions prior to criticality; to align the ΔT and Tav_g process instrumentation at power levels up to 75% power and extrapolate values to 100% to check the alignment of ΔT and Tav_g instrumentation at full power.

This item is resolved.

14. Initial Criticality INT-TP-7.2, Approved 4/3/75

Reference: IE report No. 50-286/75-03, Detail, Paragraph 4.f.(1)

The purpose of this procedure is to achieve initial criticality and to establish the upper limit of neutron flux level for all zero power physics measurements.

a. Predicted Boron Concentration and Rod Position at Criticality

Reference page 3, step 4.4 which states that during the performance of this procedure, especially during the boron dilution phase, the Westinghouse Technical Advisor (test engineer) will monitor the changing core conditions and keep the licensee's Shift Watch Foreman or his designated alternate informed of his evaluations. The expected "just critical" conditions follow:

Boron Concentration: 1365 ± 50 ppm
RCC Bank Configuration:
 Control Bank D at 160 steps
 All other Banks at 228 steps
Tavg: 547°F (+0, -5)
RCS pressure: 2235 ± 50 psig.

This item is resolved.

b. I/M Plot Interpretation

Reference: IE Report No. 50-286/75-03, Detail, Paragraph 4.f.(2)

Reference pages 4, 5, and 6, steps 5.2, 5.3, 5.8 and 5.9 of TP-7.2. These steps specify the I/M plots during the criticality and include requirements for plot interpretation. This item is resolved.

c. Initial Criticality SUR

Reference: IE Report No. 50-286/75-03, Detail, Paragraph 4.f.(3)

Based on further review of the use of 1 DPM this startup rate appears to be acceptable for TP 7.2. This item is resolved.

d. Boron Dilution Rate

Reference: IE Report No. 50-286/75-03, Detail, Paragraph 4.f.(4)

Reference page 6, step 5.9 of TP-7.2 where the adjusting of the primary makeup valve FCV-111A to obtain dilution rates between 850 and 1250 pcm/hr is noted. Also, the reactivity addition is under the control of the test director through the Shift Watch Foreman.

The licensee has written a change for the FSAR to assure agreement between the test procedure and the FSAR Section 13, page 13.3-2.

This item is conditionally resolved pending submittal of the FSAR change.

e. Manual Scram Test

Reference: IE Report No. 50-286/75-03, Detail, Paragraph 4.f.(6)

Reference page 4, step 5.1 of TP-7.2 states that, prior to initial approach to criticality, ensure that the manual trip circuit function properly by depressing the reactor trip button on the control room console and verify that the reactor trip breakers open. This item is resolved.

f. Procedural Details

Reference: IE Report No. 50-286/75-03, Detail, Paragraph 4.f.(7)

The "Limit 3 Core Loading Prerequisite List," April 14, 1975 has been written. It contains the listing of system requirements to assure readiness of the plant for core loading.

Attachment 1 - "Periodic Checkout and Calibration of Reactivity Computer," March 21, 1975, was reviewed to cover special equipment requirements. This attachment has the purpose of assuring confidence in data obtained using the Reactivity Computer. Both a static and dynamic test of the complete computer system will be demonstrated daily. A record of the test will be kept on attached data sheets.

This item is resolved.

15. Acceptance Criteria for Power Coefficient of Reactivity - INT-TP-8.1, "Power Level Change" Approved 4/2/75

Reference: IE Report No. 50-286/75-03, Detail, Paragraph 4.h.(1)

TP-8.1 was reviewed. The purpose of this procedure is to measure the differential power, coefficient of reactivity and to measure the integral power defect. Reference is made to page 11 where acceptance criteria is specified as design value of 1415 ± 212 pcm. Extractions have been taken from reference documents and inserted as part of the procedure. This item is resolved.

16. Motor Operated Accumulator Isolation Valve Testing

Reference: IE Report No. 50-286/75-03, Detail, Paragraph 5

The Indian Point 3 Technical Specifications do not require power to the accumulator isolation valves below 1000 psig. Under this condition there is no requirement for testing the valves against a high differential pressure. This item is resolved.

17. Turbine Overspeed Limit - INT-TP-9.7, "Generator Load Trip"
Approved 4/17/75

Reference: IE Report No. 50-286/75-03, Detail, 4.h.(2)

The purpose of this procedure is to obtain a turbine overspeed reference point for comparison with calculated levels.

Reference page 11a of TP-97. The revised procedure contains the curve for limits 2 and 3 showing overspeed as a function of load. This item is resolved.

18. SAO-104, Maintenance Work Requests Procedure

Reference: IE Report No. 50-286/74-23, Detail, Paragraph 10.a.(3)

The revised procedure has been reviewed. It now contains directions for a maintenance package for retention where safety related maintenance is performed. This item is resolved.