

POWER AUTHORITY OF THE STATE OF NEW YORK  
INDIAN POINT NO. 3 NUCLEAR POWER PLANT

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June 13, 1980  
IP-WAJ-8924

Docket No. 50-286  
License No. DPR-64

Mr. Boyce H. Grier, Director  
U.S. Nuclear Regulatory Commission  
Region I, 631 Park Avenue  
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

Inspection Report 79-29 received on May 4, 1980, identified two items of non-compliance for failure to fully comply with Technical Specifications and Administrative Procedures.

The following actions have been taken to enhance compliance with the requirements of Technical Specifications 6.8.1.C, 4.4.H and AP-19.

- A. Personnel who review the surveillance tests have been instructed to be more observant of potential statements which have an effect on the completeness of the test.
- B. Where possible, temporary procedure changes will be hand-written into the test procedures prior to their issuance for performance.
- C. There will be a program instituted to formally follow-up on items noted during conduct of the surveillance test that effect equipment operability.
- D. The performance groups staffing will be increased to allow a more timely review of the surveillance tests.

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- 3PT-R16, Revision 0, Diesel Generator Full Load Test. Jacket cooling water sample tests on 32 and 33 generators, performed on September 20, 1979, shows that pH limits of 9.5 were exceeded.

After corrective action by the chemist the final "As Left" PH values were not entered in the original test procedure to document that these values were within the limits specified in the test procedures. They were, however, entered in the chemical inventory files.

Personnel reviewing the surveillance tests have been instructed to be more observant of potential statements which leave the completeness of the test in question. Also the establishment of a formal follow-up system for such problems will add assurances that this problem will not reoccur.

- 3PC-R21, Revision 1, BIT Recirculation Flow Indicator Calibration Check. Review of the test performed on October 18, 1979 revealed that the BIT recirculation flow indicator, F1-916, appeared to be reading erroneous values (60 GPM) until the gage was tapped and the value dropped to 39 GPM.

Considering the fact that this is only a local flow indicator, the noted condition was felt not to adversely affect the operability of the flow indicator. Once again the personnel reviewing the surveillance tests have been instructed to be more observant of potential statements which leave the completeness and/or the test results in question.

- 3PT-R3A, Revision 1, Safety Injection Test. Review of the test performed on November 6, 1979 revealed that procedure change SV-71-78 had not been incorporated in the procedure.

In the future temporary procedure changes of this nature will be more expeditious incorporated into the procedures. Temporary procedure change SV-71-78 has been incorporated.

- 3PT-R32C, Revision 1, June 20, 1979, Fan Testing Check List. Technical Specifications require that the five units of the containment air filters should be capable of removing 85% or more of Iodine at +20% accident design flow rate, with the inlet Iodine concentration of 5 15 mg/M<sup>3</sup>.

The tests, performed on June 8, September 28, September 29, October 1 and 2, 1979, by the Mine Safety Applcance Company, were conducted with the pollutant concentration of 0.15 mg/M<sup>3</sup>.

Subsequent to the inspection and prior to going on the line, and prior to the expiration of the allowable time period for the previously completed test with the proper concentration, the tests were repeated with the required concentration 5 to 15 mg/M<sup>3</sup> and documentation placed in the files. The filter specification has been revised to reflect the proper concentration of pollutants.

B. During the performance of 3PT-A2 "Containment Structural Integrity Inspection", four minor deficiencies were noted. These deficiencies were reviewed and determined not to constitute a significant deterioration of the integrity of the containment structure. This evaluation was documented in the comment section of the test procedure. Subsequent to this, at the request of the Nuclear Regulatory Commission Inspector, a more detailed evaluation was performed which substantiated that the structural integrity of the Containment Building was not degraded by the presence of the noted deficiencies. The final disposition of each of the noted deficiencies are as follows:

Item 1 - Subject holes have been repaired.

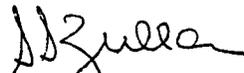
Item 2 - Engineering evaluations show that present conditions of liner plates are acceptable.

Item 3 - Hold-down nut has been installed.

Item 4 - Engineering evaluation shows that present conditions of paint are acceptable.

In the future the resolution of items of this nature will be more clearly documented and any associated repairs will be more expeditiously performed.

Very truly yours,



S. S. Zulla  
Resident Manager

WAJ/bam