

POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NO. 3 NUCLEAR POWER PLANT

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November 15, 1979
IP-WAJ-6180

Docket No. 50-286
License No. DPR-64

Boyce H. Grier, Director
Office of Inspection and Enforcement
Region 1
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Feedwater Line Inspection

Dear Mr. Grier:

We have conducted the examinations as required by I.E. Bulletin 79-13 dated June 25, 1979 during our 1979 refueling outage, which commenced on September 15, 1979, and have the following to report.

Examinations as required by Item 1 have been completed with the following results:

- A. Each of our four steam generator feedwater nozzles to piping welds were radiographically examined, including two wall thickness on either side. Evaluations were made to ASME Section III, Subsection NC, Article NC-5000 and no cracking or any other unacceptable code discontinuities were noted.
- B. Examinations required by this item were not performed as there was no cracking noted in item A examinations above.
- C. Visual inspection was performed on the feedwater system piping supports and snubbers in containment. Operability and conformance to design have been verified. No deviations from design drawings were noted in the installation of hangers and snubbers.

While still shutdown for our II to III refueling cycle, we elected to complete the balance of examinations required by Paragraph 2.A of the June 25, 1979 I.E. Bulletin 79-13. We have a common nozzle for both main and auxiliary feedwater systems and therefore performed radiographic examination of "all feedwater pipe weld areas inside containment."

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There are thirty-seven (37) feedwater welds inside containment, all of which were radiographically examined. Evaluations were made to ASME Section III, Subsection NC, Article NC-5000 and no cracking or any other code discontinuities were noted with the exception of BFD-143 weld on line 6 of steam generator no. 31, which contained two linear indications reported to you in our letter IP-WAJ-5926 dated October 12, 1979.

Radiographic examination revealed one 7/8" and one 3/8" indication in the weld root. A very scrutinous review of the original radiographs of this weld revealed that they were faintly present, but were undetected by the original construction evaluation because the original code required inspection (ANSI B31.1) was slightly less sensitive than the method used during the inservice inspection required by I.E. Bulletin 79-13.

We have evaluated the two linear indications which appear to be lack of penetration on the weld no. BFD-143, and have determined the following:

1. The indications were not picked up during the original radiographic inspection as required by ANSI B31.1 code as the sensitivity requirement of that code was 2-4T level of inspection with 2/8% equivalent penetrameter sensitivity, the bulletin requires 2-2T level of inspection with 2.0% equivalent penetrameter sensitivity. Therefore this weld was acceptable by ANSI B31.1 standards.
2. No cracks or sites of crack propagations are present in the weld in question.
3. The weld is not one of the high seismic stressed welds as identified by Westinghouse.
4. This weld is not in a high thermal area.

It has therefore been determined that repair of the weld to eliminate the two indications is not necessary at this time.

Further, the area of the indications will be re-examined by the same technique at the next refueling outage, to monitor it.

In addition we performed radiographic examination of four (4) welds containing auxiliary feedwater piping to main feedwater lines

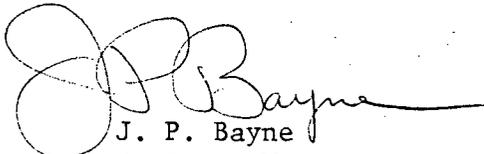
outside containment including 18" of base metal of the main feed-water line downstream of the connection. No cracking or other unacceptable code discontinuities were noted.

All radiography was performed to the 2T penetrameter sensitivity level, in lieu of table NC 5111-1, with systems void of water.

Upon comparing Revision 2 dated October 17, 1979 of the subject bulletin received several days after our completion of the inspections required by the June 25, 1979 issue of the same bulletin, we found that we have radiographically examined approximately twice the number of welds now required.

We therefore conclude that we have met the inspection requirements of I.E. Bulletin 79-13 Revision 2 as well.

Very truly yours,


J. P. Bayne
Resident Manager

WAJ/bam

cc: Office of Inspection and Enforcement
Division of Reactor Operations Inspection
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555