MAY 22 1970

R. C. DeYoung, Assistant Director
for Pressurized Water Reactors
Division of Reactor Licensing (2)

CONSOLIDATED EDISON COMPANY - INDIAN POINT 3, CO REPORT NO. 286/70-3

The enclosed report of an inspection of the subject facility on March 24 and 25, 1970, is forwarded for information and for possible action.

The item suggesting a possible need for action is concerned with the lack of documentation in the application of the Wedco/UE&C QA-QC function. Current indications are that Wedco will assume all UE&C QC functions in the near future. Accordingly, we recommend that the licensee be required to update the application to reflect the Wedco QA-QC responsibility.

The previously identified containment liner out-of-round problem, CO Report No. 286/70-2, is being evaluated by the licensee and his contractors. Pouring of related concrete has been deferred pending satisfactory resolution. We will continue to follow this matter during future inspections.

J. P. O'Reilly, Chief
Reactor Inspection and
Enforcement Branch
Division of Compliance

Enclosure:
CO Report No. 286/70-3

cc w/enclosure:
E. G. Case, DRS (3)
P. A. Morris, DRL
R. S. Boyd, DRL (2)
D. J. Skovholt, DRL (2)
L. Kornblith, Jr., CO
J. G. Davis, CO:II
B. H. Grier, CO:III
D. I. Walker, CO:IV
R. W. Smith, CO:V
R. T. Carlson, CO:I w/o encl
REG Central File

50-286

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OFFICE ▶		CO	CO			
SURNAME ▶		FJNolan:kbj	JPO'Reilly			
DATE ▶		5/21/70	5/21/70			

UNITED STATES
ATOMIC ENERGY COMMISSION
DIVISION OF COMPLIANCE
REGION I
970 BROAD STREET
NEWARK, NEW JERSEY 07102

201 645-3944

May 14, 1970

J. P. O'Reilly, Chief, Reactor Inspection & Enforcement Branch,
Division of Compliance, Headquarters

CONSOLIDATED EDISON COMPANY (INDIAN POINT 3)
DOCKET NO. 50-286

The attached report of an inspection of the subject facility on March 24 and 25, 1970, is forwarded for action.

Operation of the primary concrete batch plant has been resumed. Concrete strength test results show that adequate concrete was placed from the substitute plant during the interim.

The QA-QC procedures noted to be lacking during previous inspections have been written and implemented.

The previously identified containment liner out-of-round problem is still being evaluated by the licensee and its contractors. Pouring of related concrete has been deferred pending satisfactory resolution. We will continue to follow this matter.

Our inspector indicates a belief that the QA-QC efforts being accomplished at present by Wedco, UE&C and Con Ed will continue and even get stronger as closer coordination is now evident. However, one area of growing concern to us (action item) is the lack of documentation in the application (IP-3) that speaks to the Wedco/UE&C QA-QC changes. I can appreciate the inherent difficulties for DRL in amending a PSAR following issuance of the CP. Conversely, you are aware of the difficulties not doing so imposes upon us in the field, specifically, a lack of current inspection criteria. In the case of IP-3, current indications are that Wedco will assume all UE&C QC functions in the near future. We are of the opinion that should that happen, it will be imperative to update the QA plan in the application. Therefore, it is suggested that CO:HQ request DRL to take some action in this area.

/S/ O'R/Staff 7/9/70
a/g
Cobb

R. T. Carlson
R. T. Carlson
Senior Reactor Inspector

Attachment:
CO Report No. 286/70-3
by R. F. Heishman, dated 5/7/70
(20 cys)

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no SDS

cpd