

OCT 17 1973

Dockets

Docket No. 50-286

R. C. DeYoung, Assistant Director for Pressurized Water Reactors, L

REQUEST FOR ADDITIONAL INFORMATION FOR INDIAN POINT NUCLEAR GENERATION UNIT 3

Plant Name: Indian Point Nuclear Generating Unit 3
Docket No.: 50-286
Licensing Stage: OL
NSSS Supplier: Westinghouse
Architect Engineer: United Engineers and Constructors
Containment Type: Dry
Responsible Branch & Project Manager: PWR #2; H. Specter
Requested Completion Date: N/S
Applicant's Response Date: Not Available
Review Status: Awaiting Information

Enclosed is a request for additional information. This question relates to the mass and energy release to the containment during the post-reflood period of a loss-of-coolant accident and the modeling of the heat sinks. We have had generic discussions of these problems with Westinghouse and they are currently conducting analyses for Indian Point #3 as well as for other plants.

We would strongly advise that the information that we are requesting be provided for our review before the November ACRS meeting. This information has been requested on other W PWR plants.

Robert L. Tedesco, Assistant Director
for Containment Safety
Directorate of Licensing

Enclosure:
As stated

cc: w/o encl.

A. Giambusso
W. McDonald

w/encl.

S. Hanauer
J. Hendrie
PWR Branch Chiefs
J. Glynn
H. Specter
J. Shapaker

Distribution:

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Docket File 50-286

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OFFICE ▶	L: CSB <i>[Signature]</i>	L: CSB <i>[Signature]</i> J. Shapaker		
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DATE ▶	10/16/73	10/16/73		

REQUEST FOR ADDITIONAL INFORMATION
INDIAN POINT NUCLEAR GENERATING UNIT 3
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1. The containment pressure response analysis should be expanded to include the effects of possible energy addition during the period when the core has been recovered with water. Considering that a break in a cold leg at the pump suction has occurred, the core will be cooled by boiling and a two-phase mixture of steam and water will exist in the core. Provide the results of an analysis showing the height as a function of time for the two-phase mixture level above the core. If any entrained liquid is determined to exit into the core and then enters the steam generators, provide the ensuing energy release rate into the containment and the results of an analysis of the containment pressure response for this release.