NOV 06 1973

Docket No. 50-286

D. B. Vassallo, Chief, Pressurized Water Reactors Branch No. 1, L

FORTHCOMING INDIAN POINT UNIT 3 FULL ACRS MEETING

DATE: November 9, 1973, Friday

TIME: 11:15 a.m.

LOCATION: Room 1046
1717 H Street
Washington, D. C.

PURPOSE: See attached agenda.

PARTICIPANTS: ACRS - All members

Con. Ed. - W. Cahill et al

AEC - D. Vassallo, H. Specter,
T. Novak et al

Original signed by
H. Specter

H. Specter, Project Manager
Pressurized Water Reactors
Branch No. 1
Directorate of Licensing

cc: Docket
AEC PDR
Local PDR
L Rdg
RP Rdg
PWR-1 Rdg
PWR-1 File
E. G. Case
A. Giambusso
J. M. Hendrie
R. S. Boyd
TR Asst. Directors
J. Lee
OGC, S. Treby
TR Branch Chiefs
R. W. Klecker
R. F. Fraley, ACRS (16)
RO (4)
RP Asst. Directors
RP Branch Chiefs
Receptionist (H Street)
T. Novak
R. Pollard
M. Dunenfeld
S. Pawlicki
F. Allenspach
W. Houston
H. Specter

memo

OFFICE ▶	PWR-1				
SURNAME ▶	HSpecter:ms				
DATE ▶	11/6/73				

AGENDA

INDIAN POINT UNIT 3 FULL ACRS MEETING

1. Site Description and Introductory Statement

2. Brief Project Status

3. Core Power Capability Analysis

   a. Compliance with Interim Acceptance Criteria
   b. Comparison with Zion and Indian Point 2
   c. ECCS review and independent analysis
   d. Sensitivity Analysis (ECCS)

4. Technical Specifications

   a. Administrative requirements for maintaining peaking factors, e.g., part length rod limitations, rod rate of motion limits
   b. Core Surveillance Requirements
   c. Alternative or Additional requirements to account for unanticipated effects, e.g., reductions in F_q to provide for LOCA combined with steam tube rupture

   d. Peaking Factor Monitoring - Fixed In-Cores vs Other Methods, Use of Thermocouples
   e. Inservice Inspection (Fracture Toughness) - Inservice Inspection of Steam Generators

5. Emergency Plan/Post Accident Monitoring

   a. What actions are specified beyond LPZ?
   b. Instruments provided to follow the course of an accident - Discuss adequacy as to instrument range, type of read out, facility of the operator to interpret.
   c. Does the stack monitor have a high enough range to detect a fuel element failure?

6. Generic Items

   a. Reactor Coolant Pump Overspeed
   b. ATWS
   c. Turbine Overspeed/Missiles
   d. Failure of Non Class I Components

      flooding
      electrical cable overheating

      isolation of low pressure and high pressure systems

7. Transportation Hazards - Barge Damage

8. Security Plan - Access to site from Hudson River