UNITED STATES
ATOMIC ENERGY COMMISSION
WASHINGTON, D.C. 20545

December 27, 1973

Docket No. 50-286

D. B. Vassallo, Chief, Light Water Reactors Project Branch 1-1, L

FORTHCOMING MEETING WITH CONSOLIDATED EDISON - INDIAN POINT 3

DATE & TIME: January 9, 1974
9:00 a.m.

LOCATION: Room 100
Woodmont Building

PURPOSE: Tech. Spec. Meeting

PARTICIPANTS: Con-Ed - R. Remshaw
AEC - H. Specter

H. Specter

H. Specter, Project Manager
Light Water Reactors
Project Branch 1-1
Directorate of Licensing

cc: Docket File ✓
AEC PDR
Local PDR
L Reading
RP Reading
LWR Group 1-1 Reading
LWR Group 1-1 File
E. G. Case
A. Giambusso
J. M. Hendrie
R. S. Boyd
TR Assistant Directors
TR Branch Chiefs
R. W. Klecker
R. F. Fraley, ACRS (16)
RO (4)
RS (3)
RP Assistant Directors
Receptionist (Bethesda)
H. Specter
RP Branch Chiefs
Attorney, OGC
D. Rosebaum
J. Lee

UNITED STATES
ATOMIC ENERGY COMMISSION
DIRECTORATE OF REGULATORY OPERATIONS
REGION 1
~~970 BROAD STREET~~
~~NEWARK, NEW JERSEY 07102~~
631 Park Avenue
King of Prussia, Pennsylvania 19406

H. D. Thornburg, Chief
Field Support & Enforcement Branch
Directorate of Regulatory Operations, HQ

November 30, 1973

ACRS MEETING - IP-3 OPERATING LICENSE

Attached for your information are Jack Allentuck's notes on the subject meeting which was held on November 9, 1973.

Robert T. Carlson

Robert T. Carlson, Chief
Facility Construction &
Engineering Support Branch

Enclosure: As stated.

cc: James B. Henderson
Ferd A. Dreher

50-286
ACRS

UNITED STATES
ATOMIC ENERGY COMMISSION
DIRECTORATE OF REGULATORY OPERATIONS
REGION 1
~~970 BROAD STREET~~
~~NEWARK, NEW JERSEY 07102~~
631 Park Avenue
King of Prussia, Pennsylvania 19406

Robert T. Carlson, Chief
Facility Construction & Enforcement Branch

November 20, 1973

SUBJECT: ACRS MEETING RELATIVE TO AN OPERATING LICENSE FOR IP-3

I attended a meeting of the Advisory Committee on Reactor Safeguards (ACRS) on November 9, 1973 relative to an operating license for IP-3. The highlights of the meeting were as follows:

The staff identified matters requiring additional effort by the staff:

a) Service water system, specifically the fact that a single 10-inch line provides cooling water to all three diesels. A possible solution involves supplying cooling water for one diesel from the conventional service water header while two remain on the nuclear service system. Appropriate interties and valving would be furnished so that the machine could be switched to the intact system in the event of failure of one system. A possible defect in this scheme lies in the fact that routing of service water lines past heat sources might be detrimental to jacket water cooling function. Another possible design revision would be to associate an accumulator with each jacket water system.

b) Problems related to effluent treatment -- this will be resolved by interties between IP-1 and IP-3 to affect treatment of blowdown.

c) Fuel densification -- this matter has been resolved.

d) Turbine overspeed -- (The discussion of this matter was by the licensee.) The problem arises from the fact that sufficient steam is trapped beyond the shutoff valve so that when the latter is closed on a turbine trip, the trapped steam expanding through the low pressure turbine causes an overspeed. It was calculated that with the unit operating at 965 MW(e), a speed of 135% synchronous speed could result. This exceeds the maximum allowable of 132%.

The fix described was as follows:

> Design is underway to provide a system so that on a turbine trip signal, steam leaving the high pressure turbine will be dumped directly to the condenser. It is hoped that this system will be operative at the time the unit goes on line. If there is a delay, the Technical Specifications will be used as an administrative device to limit the load to a level which will prevent overspeed from exceeding 132%.

The staff stated that although the actual design had not been reviewed, it was thought to be a feasible approach to the overspeed problem.

A discussion relative to inspection of the primary nozzle to vessel head welds in the steam generator ensued. The applicant stated that since the nozzles are cast together with the hemispherical head, there are no welds and the area need not receive an in-service inspection. The initial ACRS view (Okrent) was that in-service inspection was required (based on conventional boiler practice). The staff position on in-service inspection is that inspection, according to the '72 addendum to the code, requires volumetric NDT of 20% of welds where gross structural discontinuities occur.

The question of R.C. pump overspeed was discussed. A possible solution is to keep the R.C. pump tied to the generator. This has construction consequences in that if the LOCA is considered to be triggered by seismic events, the breaker must be qualified as seismic Class 1.

Finally, in a closed session, site security was discussed. I will not elaborate on this subject.

In conclusion, ACRS approval for an O.L. was initiated.

Jack Allentuck, Reactor
Inspector, FC&ESB

cc: R. F. Heishman