

UNITED STATES GOVERNMENT

Memorandum

TO : Those listed below

DATE: October 9, 1967

FROM : P. A. Morris, Director *P. A. Morris*
Division of Reactor Licensing

SUBJECT: REVIEW OF INDIAN POINT UNIT NO. 3, DOCKET NO. 50-286
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

A meeting was held August 21, 1967, in my office to discuss the proposed review plan for Indian Point Unit No. 3. At this meeting, the principal areas to be considered, and those matters which require assistance from the various groups within DRL, were identified. The program agreed upon, and a general description of the review requirements, are outlined below. This project is assigned to Reactor Project Branch No. 1 (D. R. Muller). The project leader for Indian Point Unit No. 3 is J. A. Murphy.

- 1.0 Areas in which Reactor Technology will provide assistance to Reactor Projects.
 - 1.1 Determination of the adequacy of the codes used to calculate blowdown and core heatup following a loss-of-coolant accident.
 - 1.2 Determination of the adequacy of the codes and experimental data used by Westinghouse in predicting the lower hot channel factors.
 - 1.3 Evaluation of the adequacy of analysis of the consequences of pressure vessel thermal shock during safety injection.
 - 1.4 Evaluation of the containment structural design.
 - 1.5 Evaluation of the conformance of the control and safety systems to the proposed IEEE standards.
 - 1.6 Evaluation of the conformance of the emergency electrical power system to the Supplementary Criteria for Auxiliary Electrical Power System for Nuclear Power Plants and a determination of the adequacy of the normal electrical power system.
 - 1.7 Performance of a parameter study indicating the sensitivity of off-site doses to assumed efficiencies of the halogen scavenging system and assumed organic halide inventory.



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- 2.0 Areas in which Reactor Projects will conduct a review.
- 2.1 Determination of potential interactions between Unit 3 and Units 1 and 2.
- 2.2 Review of the acceptability of the fuel burnup proposed with regard to fuel integrity.
- 2.3 Evaluation of the adequacy of the engineering safety features proposed, including the safety injection, halogen removal, containment spray, containment isolation, and containment penetration systems.
- 2.4 Determination of the adequacy of the steam and power conversion system with regard to primary system and core integrity resulting from failures in this system.
- 2.5 Review of the applicable shielding criteria.
- 2.6 Review of the adequacy of the safety evaluation performed by the applicant for all accidents considered.
- 2.7 Review of the description and objectives of the proposed R&D program to determine the degree of assurance of acceptable results.
- 2.8 Determination of the degree of conformance of the proposed design to the July 11, 1967 Design Criteria.
- 3.0 The following areas will not be considered for in-depth review:
- 3.1 Aspects of the thermal and hydraulic design not affected by the reduced peaking factors.
- 3.2 Fuel and core internal mechanical design.
- 3.3 Reactor coolant system.
- 3.4 Chemical and volume control system.
- 3.5 Sampling system.
- 3.6 Fuel handling system.
- 3.7 Service water system.
- 3.8 Waste disposal system.

Meetings with the applicant have been held on August 22, 1967, September 8, 1967, and September 26, 1967. A list of the additional information required for the completion of our review is presently being prepared. Additions to this list should be provided to Reactor Projects (J. Murphy) by October 13.

A proposed review schedule follows:

Questions transmitted to applicant	October 16, 1967
Consultant reports	October 13, 1967 January 9, 1968
Technical meeting to discuss answers	February 5, 1968
ACRS Report	February 19, 1968
ACRS meeting	March 7, 1968
Safety Evaluation	April 5, 1968
Hearing	April 25, 1968
Issue Construction Permit	May 9, 1968

Addressees:

R. S. Boyd, A/D, RP
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