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POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NO. 3 NUCLEAR POWER PLANT

P. O. BOX 215 BUCHANAN, N. Y. 10511

TELEPHONE: 914-739-8200



November 28, 1979
IP-NAP-6300

Docket No. 50-286
License No. DPR-64

Boyce H. Grier, Director
Office of Inspection and Enforcement
Region 1
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

In accordance with the Technical Specifications of Facility Operating License No. DPR-64, the following describes Reportable Occurrence LER-016/01T-00. This event is the type defined in Technical Specification 6.9.1.7 (h).

On November 27, 1979, the Power Authority was notified by our Nuclear Steam Supplier, the Westinghouse Electric Corporation, that a detailed evaluation of the current LOCA/ECCS (October 1975) evaluation model indicates that a non-conservative feature could exist in the Appendix K LOCA analysis with respect to the portion of the calculation related to fuel rod burst. The potential non-conservative feature of the Westinghouse large break ECCS evaluation model is as follows. The model uses a curve which represents fuel clad burst conditions for clad heatup rates of 25°F/second and greater. The evaluation discussed above revealed that heatup rates could be less than 25°F/second. During the LOCA transient, the fuel clad burst curve is dependent on the clad heatup rate prior to burst and a reduction in heatup rate causes earlier clad burst. A shift in clad burst time can affect the peak clad temperature (PCT) calculated for the LOCA transient.

As determined from the most recent LOCA analyses, Indian Point 3, has heatup rates less than 25°F/second. However, Westinghouse believes that reanalysis with the most current Westinghouse LOCA/ECCS evaluation model (February 1978) would show that no change to F_b is necessary. That is, they believe margins available in this model will more than compensate for any effect associated with the change in the fuel clad burst curve. An analysis with the current model using the appropriate heatup rate dependent burst curves, is currently being performed. The results of this analysis will be available before the unit comes back on line after the current refueling outage.

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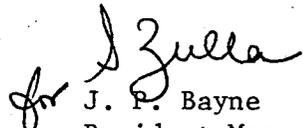
Boyce H. Grier

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Mr. J. Johnson, Resident Inspector, was notified on November 27, 1979, of the aforementioned circumstances. This letter constitutes the twenty-four hour written notification required by the Technical Specifications.

Very truly yours,


for J. P. Bayne
Resident Manager

NAP/bam

cc: Director of Nuclear Reactor Regulation (2 copies)
Office of Management Information and Program Control
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Resident Inspector T. Rebelowski