

## POWER AUTHORITY OF THE STATE OF NEW YORK

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August 3, 1979  
IPN-79-55

Mr. Boyce H. Grier, Director  
Office of Inspection and Enforcement  
Region I  
U. S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Subject: Indian Point 3 Nuclear Power Plant  
Docket No. 50-286  
IE Bulletin 79-04

Dear Sir:

The Authority's June 1, 1979 letter to you (IPN-79-30) on the subject bulletin indicated in the response to Item 4 that the reanalysis of four (4) lines containing Velan valves were included in the Authority's activity associated with IE Bulletin 79-07, and that the analysis would be completed by July 1, 1979.

By letter dated June 29, 1979 (IPN-79-42) the Authority submitted its initial reanalysis of selected critical lines associated with IE Bulletin 79-07. Only three (3) of the four (4) lines mentioned in IPN-79-30 were included in this submittal.

The original commitment in IPN-79-30 has since proven to be incorrect in that only three of the four valves are contained in lines undergoing reanalysis per IE Bulletin 79-07. The fourth valve, namely valve number 374, is contained in a chemical and volume control system line (CVCS), line 19 which was not analyzed by a questioned computer program. Line 19 supports were designed on the basis of static spacing tables for deadweight and for the earthquake, and the stress analysis was performed using simplified stress charts.

The Authority has completed its review of the stress analysis, the pipe support and the piping penetration associated with valve number 374 and have found that the piping stress

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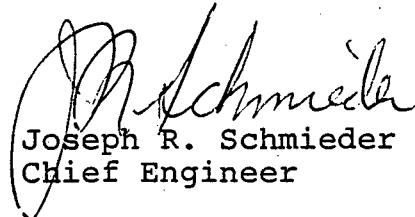
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levels are not significantly changed from the original calculation. The Authority also finds that the pipe support and piping penetration have adequate capability to carry the additional load imposed by the 13% increase in weight of the valve. The Authority, therefore, concludes that the increased weight does not adversely affect the analysis of the pipeline. The Authority further concludes that this satisfactorily addresses our complete response to IE Bulletin 79-04.

Very truly yours,

  
Joseph R. Schmieder  
Chief Engineer

cc: Office of Inspection and Enforcement  
Division of Reactor Construction Inspection  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555