

POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NO. 3 NUCLEAR POWER PLANT

P. O. BOX 215 BUCHANAN, N. Y. 10511

TELEPHONE: 914-739-8200



REGULATORY DOCKET FILE COPY

June 22, 1979
IP-NAP-5146

Docket No. 50-286
License No. DPR-64

Boyce H. Grier, Director
Office of Inspection and Enforcement
Region 1
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

In accordance with the Technical Specifications of Facility Operating License No. DPR-64, the following describes Reportable Occurrence (LER) 79-006/01T-0. This event is the type defined in Technical Specification 6.9.1.7 (h).

On June 21, 1979, the Power Authority was notified by our Nuclear Steam Supplier, the Westinghouse Electric Corporation, of an error in the less conservative direction in their analysis of high energy line breaks inside containment. Breaks of this type can result in heatup of the steam generator level measurement reference leg resulting in a decrease of the water column density with a consequent apparent increase in the indicated steam generator water level (i.e., apparent level exceeding actual level). This potential level bias could result in delayed protection signals (reactor trip and auxiliary feedwater initiation) which are based on low-low steam generator water level. In the case of a feedline rupture, this adverse environment could be present and could delay or prevent the primary signal arising from declining steam generator water level (low-low steam generator level). Containment pressure and safety injection provides a backup signal which initiates a trip before a containment temperature of 280° F. is reached. This trip is conservatively set at 3.0 psig. Westinghouse has calculated that raising the steam generator low-low level setpoint by 10 per cent of span would compensate for the effects of this temperature. Since the Power Authority's calibration procedures already use a value of 15 per cent of span as opposed to the Technical Specification value of 5 per cent, and containment pressure trip of 3. psig as opposed to Technical Specification value of 3.5 psig, the change in setpoint recommended by Westinghouse is already in effect.

Further evaluation will be conducted to determine final setpoint adjustments.

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Boyce H. Grier, Director

Page Two

June 22, 1979
IP-NAP-5146

For other high energy line breaks which could introduce a similar positive bias to the steam generator water level measurement, steam generator level does not provide the primary trip function and the potential bias would not interfere with needed protective system actuation.

The effects of the temperature increase on other level instrumentation in containment for post accident monitoring is being reviewed and appropriate guidance will be given to the operators.

The Site Resident Inspector, Walter Baunach, was verbally notified on this date by Salvatore Zulla. This constitutes the twenty-four (24) hour formal notification as required by Technical Specifications.

Very truly yours,


J. P. Bayne
Resident Manager

NAP/rbb

cc: Director of Nuclear Reactor Regulation (2 copies)
Office of Management Information & Program Control
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Resident Inspector Baunach