ABSTRACT

Service experience indicates that the probability of flaw propagation into the base metal underneath clad in PWR systems is remote. However, a two step ultrasonic inspection technique was developed in preference to a single step technique to provide maximum sensitivity and depth definition of any possible flaw propagation.

Acceptance criteria for continued operation will be within the limits of Section XI of the ASME Code with its built in safety factors viz. 3 on stress and 10 on flaw size.

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INTRODUCTION

Service experience with PWR systems indicate that flaws in stainless steel weld clad on carbon or low alloy steel will not propagate into the base metal underneath (Attachment I). However, it was requested that an ultrasonic method be developed to provide a means of in-service inspection of clad flaws to monitor possible growth into base metal. These flaws could then be evaluated, using rules defined in Section XI of the ASME Code to determine if they would affect the structural integrity of the vessesls.

ULTRASONIC INSPECTION PROGRAM

In order to obtain the best possible linear response and therefore the best flaw depth definition, a two step ultrasonic procedure has been developed and is described in Attachment II.

Step I of this ultrasonic procedure will accurately define flaws to a depth of 3/8 inch and will be used for the in-service inspections. Since the maximum clad thickness in the surveillance areas is 1/4 inch, Step I will define a flaw propagation into the base metal up to 1/8 inch. In the event that flaws exceed 3/8" then Step II will be used to further define depths up to 3/4 inch.

EVALUATION PROGRAM

The results of the ultrasonic inspections will be evaluated in accordance with the rules of Section XI of the ASME Code as follows:

- If no flaws are found which propagate into base metal no further evaluation is required.
- 2. If flaws are found which propagate into the base metal, they will first be evaluated to section XI IWB-3512.1 and, therefore, must stay within the limits of IWB-3512.1 (a). Justification is given in Attachment III. If no flaw depths exceed these limits no further evaluation will be required.
- 3. The critical flaw parameters with respect to flaw location may be calculated as outlined in Section XI IWB-3600. Attachment IV gives the calculations for the critical flaw sizes at the location of maximum stress in the surveillance area using specified cyclic duty.

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If plant experience has not exceeded specified cyclic duty these flaw aspect ratios may be used to evaluate all flaws that may have propagated into base metal beyond those listed in (2) above.

4. Larger flaws than those permitted by (3) above may be justified by fracture mechanics analyses on a case by case basis dependent on location, stress, and cyclic experience of the plant.

INSPECTION AND EVALUATION PROCEDURE

The ultrasonic inspection and evaluation procedures will be in accordance with NPT-61 and NPT-81 respectively, and are given in Appendix I of this report.

CONCLUSIONS

- An ultrasonic inspection procedure has been developed which will accurately define flaw depths greater than the critical flaw size.
- 2. Fracture mechanics analysis have been made to determine critical flaw size in the highest stress region.
- 3. Justification for continued operation will be determined by fracture mechanics analyses calculated using methods defined in Section XI of the ASME Code.

APPENDIX I

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INSPECTION AND EVALUATION PROCEEDURE NPT-61 AND NPT-81

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	Process Specification NUCLEAR POWER COMPONENTS METHOD OF SHEAR WAVE ULTRASONIC CLADDING SUF	NPT-61 Page 1 of 5 RVEILLANCE
G. Hughes	<u>PURPOSE</u> : Ultrasonic surveillance procedure f extent and possible growth of clad cracking f channel heads. The procedure includes the me baseline process and the fixturing necessary periodically	or periodically determining the In designated areas of steam generato eans to measure clad thickness as a to duplicate the test program
2/14/77 2 9/28/77 3	QUALIFICATION OF PERSONNEL: Personnel perfor procedure shall be qualified in accordance with Level II or III.	ming the examination under this th ASNT-TC-1A, Supplement "C",
	OPERATIONS:	
	Part I: Clad Measurement:	•
	 Measure clad thickness in the approximusing the procedure detailed below (Re#TR-71-24A). Record clad thickness determined in each surfaces in examination of the surface of the	nate center of designated areas of. Automation Industries' Report ach area.
	Equipment:	
	 a) UM 775 instrument "Automation" (m b) 10N db. pulser/receiver. c) Search unit fixture 57A6878. 57A3615 SIZ transmitter search unit. 57A2796 SIL receiver search unit. d) Reference standard S/N 1. e) Glycerine for couplant. 	ist fit through 18 inch manway).
	Calibration:	
	 a) Turn on instrument and allow properties b) Connect cables from transmitter an 57A6878 to their respective jacks c) Adjust the instrument controls to 	er warm up. nd receiver search units in fixture on the 10N db. pulser/receiver. the following preliminary settings:
	<u>UM 775</u>	10N db
12 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2	 Sweep Delay Push Button 5-50 Vernier - Max. (ccw) Sweep Range Push Button - 1 Material Vernier - Mid Range 	1) Sens. db control - 16 2) Reject - 1/4 cw
Jeek .	 3) Vertical - Adjust to locate baseline at 0 vertical position 4) Altn Max. (ccw) 5) Rate - Max. (cw) 	 3) Test - Through 4) Pulse Tune - Max. (ccw) 5) Pulse Length - Min. (cw)
1 1021	 6) Horizontal - Adjust to center baseline on scope screen 7) Mode - Out 	6) Freq 2.25 MH _z

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NPT-61 Page 2 of 5 Westinghouse Electric Corporation Tampa Division Nuclear Energy Systems Tampa, Florida, U. S. A.

issue 1	Process Specification NUCLEAR POL ^{ME} R COMPONENTS METHOD OF SHEAR WAVE ULTRASONIC CLADDING SURVEILLANCE			
9/29/75 C. Galyen G. Hughes Changed	 Set response from 3/8" deep notch to 80% of full screen height (FSH). Note response from 1/4" and 1/8" deep notches. Set recording gate "start" at peak point of 1/8" notch indication minus 1" of test metal distance. Set recording gate "END" at peak point of 1/8" notch indication plus 3/4" of test metal distance. 			
2/14/77 2 9/28/77 3	 Connect data potentiometer of scanner (5 inch scan length) to Y coordinate of X-Y recorder. Connect amplitude response to X coordinate of X-Y recorder. 			
	 Record calibration by scanning across reference block at appropriate (approx. 1/4") increments to record maximum responses and locations from reference notches. Set pen manually to duplicate scan spacings. 			
	8. Place scanner on selected area of channel head. Set search unit on left hand edge of area to be scanned. Move search unit to top of scan (5") and return to bottom of scan length. Move search unit 3/4" to right, move pen on recorder 3/4" and repeat 5" scan. Repeat process at 3/4" increments until specified width (circumferential direction on channel head) of selected area has been traversed.			
	9. Maintain records for comparison with periodic in-service recordings made in the same manner as described.			
	10. Clean and identify areas.			
_	. Follow the above procedure on channel head by placing the scanning fixture "feet" in drilled holes for each position.			
L.	12. Record U.T. results for each area scanned on channel head.			
	13. Maintain records for comparison with periodic in-service recordings made in the same manner as described.			
	Part III: Surveillance Procedure - 1/2 inch-3/4 inch range			
	a) IM 775 Reflectoscope (Automation)			
{ 	b) 5N Weld Pulser/Receiver			
	 c) Alarm and recording module (Transigate) d) Westinghouse Immersion/contact search fixture 16° + 1° -0° incident angle e) 1.0 MH 1-1/8" dia., heavy backing, SIZ transducer, Style #57A3453 f) Scanning fixture (WTD) 			
	 g) X-Y Recorder h) Clad Channel Head reference block with 3/4 in. long by 1/2", 5/8", and 3/4" deep Flox potches 			
	i) Couplant: 2 parts glycerine, 1 part water, 4-5 drops wetting agent per			
	pint			
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t	Westinghouse Electric Corporation Tampa Division Nuclear Energy Systems Tampa, Florida, U. S. A.NPT-61 Page 3 of 5			

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 METHOD OF SHEAR WAVE ULTRASONIC CLADDING SORVEILLANDE Calibration: a) Reject off b) Set response from the 3/4" deep Elox notch to 95% of full screen height (FSH) note and record response from the 1/2" and 5/8" deep notches - should be approximately 45% (FSH) and 70% (FSH), respectively. (Note response from 0.450" notch.) Procedure: Set "feet" on search unit fixture as follows: a) Place assembled "empty" search unit fixture on 0.040-0.045 inch thick shim on flat surface. 	9/29/75 C. Galyen G. Hughes Changed 2 2/14/77 3 9/28/77
 Calibration: 3 a) Reject off b) Set response from the 3/4" deep Elox notch to 95% of full screen height (FSH) note and record response from the 1/2" and 5/8" deep notches - should be approximately 45% (FSH) and 70% (FSH), respectively. (Note response from 0.450" notch.) Procedure: Set "feet" on search unit fixture as follows: a) Place assembled "empty" search unit fixture on 0.040-0.045 inch thick shim on flat surface. 	C. Galyen G. Hughes Changed 2 2/14/77 3 9/28/77
 a) Reject off b) Set response from the 3/4" deep Elox notch to 95% of full screen height (FSH) note and record response from the 1/2" and 5/8" deep notches - should be approximately 45% (FSH) and 70% (FSH), respectively. (Note response from 0.450" notch.) Procedure: Set "feet" on search unit fixture as follows: Place assembled "empty" search unit fixture on 0.040-0.045 inch thick shim on flat surface. 	Changed 2 2/14/77 3 9/28/77
Procedure: 1. Set "feet" on search unit fixture as follows: a) Place assembled "empty" search unit fixture on 0.040-0.045 inch thick shim on flat surface.	
 Set "feet" on search unit fixture as follows: a) Place assembled "empty" search unit fixture on 0.040-0.045 inch thick shim on flat surface. 	
 a) Place assembled "empty" search unit fixture on 0.040-0.045 inch thick shim on flat surface. 	
b) Adjust feet to touch flat surface.	
 Fill fixture with water and pressurize so that approximately 75-80% of flexible bladder width touches flat surface on which four "feet" are resting. 	
 Mount scanning fixture including search unit on reference block so that sound beam is projected perpendicular to notches. 	1
 Set response from 3/4" deep notch to 95% of full screen height (FSH). (Note response from 1/2" and 5/8" deep notches.) 	
5. Set recording gate "start" at peak point of 1/2" notch indication minus 1" of test metal distance. Set recording gate "END" at peak point of 1/2" notch indication plus 3/4" of test metal distance.	
 Connect data potentiometer of scanner (5 inch scan length) to Y coordinate of X-Y recorder. Connect amplitude response to X coordinate of X-Y recorder. 	
 Record calibration by scanning across reference block at appropriate (approx. 1/4") increments to record maximum responses and locations from reference notches. Set pen manually to duplicate scan spacings. 	
8. Place scanner on selected area of channel head. Set search unit on left hand edge of area to be scanned. Move search unit to top of scan (5") and return to bottom of scan length. Move search unit 3/4" to right, move pen on recorder 3/4" and repeat 5" scan. Repeat process at 3/4" increments until specified width (circumferential direction on channel head) of selected area has been traversed.	
9. Maintain records for comparison with periodic in-service recordings made in the same manner as described.	
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NPT-61 Page 4 of 5

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· · ·	Issue 1 /29/75	Process Specification NUCLEAR POWER COMPONENTS METHOD OF SHEAR WAVE ULTRASONIC CLADDING SURVEILLANCE Page 5 of 5
) G	. Galyen . Hughes	10. Clean and identify areas.
	Changed	11. Follow the above procedure on channel head by placing the scanning fixture "feet" in drilled holes for each position.
<u>_2</u> 9/	/14/77 2	12. Record U.T. results for each area scanned on channel head.
		13. Maintain records for comparison with periodic in-service recordings made in the same manner as described.
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NPT-61 Page 5 of 5

Issue 1	Procedure for ultrasonic in-fervice inspection and evalua- TION OF CHANNEL HEAD FLAW DEFECTS
K. POOLE Changed	<u>PURPOSE</u> The purpose of this specification is to give the proceedure for in-service inspec- tion of cladding flaws, evaluating the results, and determination if repairs are required. PART I BASE LINE INSPECTION
	At the time flaws are detected, a base line inspection shall be made and clad thickness in the surveillance region measured in accordance with NPT-61. PART II DETERMINATION OF TOTAL FLAW DEPTH
	Ultrasonically determine total flaw depth using Part II of NPT-61. Should the flaw depth exceed 3/8 inch, it must be further defined using Part III of NPT-61. <u>PART III CALCULATION OF DEPTH OF FLAW INTO BASE METAL</u> Using the clad thickness, as determined in the base line inspection, and the total flaw depth determined in Part II above, calculate to determine any flaw propagation into the base metal.
	 PART IV EVALUATION OF FLAWS If no flaws are found which propagate into the base metal no further evaluation is required and the steam generators may be returned to service. If flaws extend into the base metal and if their length vs depth fall under the curve as shown in the attached Figure 1, the steam generators may be placed back in service and the proper authorities and regulatory agencies shall be notified. If flaws are found which fall above the curve in Figure 1, and below the curve of Figure 2, the steam generators may be placed back in service with the concurance of the proper authorities and regulatory agencies provided the specified design cycles of the plant have not been exceeded. Further flaw propagations can be justified on a case by case basis depending on stresses at the location of the flaw and expected remaining cycles.

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NPT-81 Page 1 of 1





FIGURE 2 - ALLOWABLE FLAW DEPTH VS. LENGTH PER IWB-3600

DEPTH - a, in.

ATTACHMENT I

(SECTION 8 OF WESTINGHOUSE TAMPA DIVISION REPORT TD-MET-75-080)

SECTION 8

SERVICE EXPERIENCE

It has been established by examination that the cladding flaws do not extend into the carbon steel channel head base metal. It has further been established that structurally the flaws are insignificant and are within the specifications of Section XI. Finally, a nondestructive testing method has experimentally verified that flaws of a critical depth are determinable. It remains to be shown that the cladding will satisfy its original purpose of protecting the primary system coolant from excessive corrosion products contamination.

Service experience with cladding which shows similar problems has shown no adverse effect on performance. Over the past 20 years, cracking has occurred in the corrosion resistant cladding of several nuclear steam supply system vessels. For example, cracking has been found in roll-bonded clad, resistance welded clad, arc welded clad, (Sequoia I Rotterdam KE2020 and JPDR). These examples are not a complete compilation of all cracking incidents but are meant to be illustrative of the extent of the problem. Causes for the cracking have been attributed to corrosion, hot cracking, improper welding, metallurgical conditions, or various combinations of these. Extensive metallurgical and structural investigations were made for each cracking incident. In all cases, the cracking was confined to the corrosion resistant cladding, i.e., in no cases did the cracks extend into the underlying base metal. In some instances, attempts were made to propagate the cracks into the base metal by fatigue. These were singularly unsuccessful. In at least one example, the Yankee-Rowe pressurizer, the vessel containing cracked cladding has seen extensive service -- over ten years. Periodic inspections at refueling intervals have confirmed that crack growth has not taken place. Furthermore, this plant has successfully operated even though low alloy steel has been exposed to the primary coolant. The Yankee-Rowe inspection and service has been documented in periodic reports to the AEC.

In the case of the cracked cladding in the head of JPDR, a Japanese BWR, the cracking in the weld deposited cladding was interdendritic near the surface

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and transdendritic further inward. Base material, A302B, was exposed. After discovery of these cracks (\sim 1966), unrepaired areas were deliberately left for future observation. In 1968, a detailed metallographic examination revealed only minor pitting about 0.004" in the exposed base metal at the tip of some of the cracks. The environment was fairly aggressive since 35 ppm oxygen was present in the steam phase.⁽¹⁾

This service experience indicates that the flaws in the cladding are not detrimental, and lend credability to the stability of the weld deposit. The probability of "grain dropping", i.e., granular particle falling from the clad matrix cannot be ignored but is believed to be very low. "Grain dropping" would occur only if exposure to the corrodant continued. Since the primary system operates with an extremely low oxygen and chloride content deterioration of the cladding by this method is not anticipated.

The following corrosion data indicate that should cracks propagate to base metal, the corrosion rates in service are acceptable under all conditions.

۹.	Environment #1	Startup	Operating	Shutdown	Shutdown
	Temperature, ^O F	100 to 630	630	630	180
	Velocity, fps	14	14	14	14
	Chemistry:		• • •		
	Boron (as H ₃ BO ₃), ppm	0	1200-0	2000	2000
	Lithium (as LiOH), ppm	1 to 2	0.5 to 2	1 to 2	1.5 to 2
	Chloride, ppm	1.0 to <0.15	<0.15	<0.15	<0.15
	Fluoride, ppm	<0.15	<0.15	<0.15	<0.15
	Oxygen, ppm	7 to <0.1	<0.1	<0.1	2.0
	Hydrazine, ppm	21 to <0.1	<0.1	- '	-
	Hydrogen, cc/kg	0 to 30	30	30 to 0	0
	Exposure Time, day	15	365	28	21
	Material	SA-556-C2 Car	bon Steel		
	Corrosion Rate	$105 \text{ mg/dm}^2/42$	9 days	·	
		(0.6 mils/429)	dav)		

Reference:

Westinghouse corrosion data obtained from testing in simulated reactor coolant conditions.

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B.	Environment #2	Aerated Solutions Coupled to S/S	Deaerated Solution Coupled to S/S	
	Temperature, ^o F	70 to 140	70 to 140	
	Velocity, fps	Stirred	Stirred	
	Chemistry:	· .		
	Boron (as H ₃ BO ₃), ppm	2500	2500	
	Lithium as (LiOH), ppm	None	None	
	Chloride, ppm	,<0,15	<0.15	
	Fluoride, ppm	<0.15	<0.15	1.00
	Oxygen, ppm	7 to 3	<0.1	
	Hydrazine, ppm	None	7 to <0.1	
	Exposure Time, day	-120	-120	
	Material	SA-3028 Carbon Steel	: · ·	
	Corrosion Rate vs. Temp., mg/dm ² -mo/ ⁰ K	$-\frac{4.6 \times 10^6}{1} + 1.6 \times 10^4$	$\frac{-4.6 \times 10^4}{1} + 1.5 \times 10^{1}$	2

Reference: WCAP-7099, "Absorption of Corrosion Hydrogen by A3028 Steel at 70° to 500°F," December 1967.

C. Environment #3

Temperature, ^O F	650
Velocity, fps	Semistatic
Chemistry:	
Boron (as H ₃ BO ₃), ppm	1100
Lithium (as LiOH), ppm	None
Chloride, ppm	Not given
Fluoride, ppm	Not given
Oxygen, ppm	<0.1
Hydrogen, cc/kg	100
Exposure Time, day	28
Material	Carbon Steel Plate
Corrosion Rate	105 mg/dm ² (0.6 mils/yr)

Reference:

ce: TID-10028 Technical Progress Report, "PWR Program for Period October 7, 1954 to November 18, 1954, and WCAP-2855, "Evaluation of Yankee Vessel Cladding Penetrations," October 1965.



The corrosion data from Environment #1 should be directly applicable to the steam generator carbon steel corrosion since the test conditions represent a fuel cycle. The corrosion at low temperatures only are defined in Environment #2 while the corrosion at operating temperatures is defined in Environment #3.

Hydrogen release by corrosion reactions has been shown not to have an embrittlement effect on A302 steel and it is believed that no effect will be seen here.⁽²⁾

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REFERENCES

- T. Kondo, H. Nakajima, and R. Nagasaki, "Metallographic Investigation on the Cladding Failure in the Pressure Vessel of a BWR," Nuclear Engineering and Design, Vol. 16, No. 3, July 1971, pp. 205-222.
- Westinghouse Report WCAP-7099, "Absorption of Corrosion Hydrogen by A302B Steel at 70° to 500°F".

ATTACHMENT II

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(WESTINGHOUSE TAMPA DIVISION REPORT TD-MET-77-110)