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CONTROLLER

April 13, 1978  
IPO-86

Director of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Attention: Mr. Albert Schwencer, Chief  
Operating Reactors Branch No. 1  
Division of Operating Reactors

Subject: Indian Point 3 Nuclear Power Plant  
Docket No. 50-286

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Dear Sir:

In response to a request from the Advisory Committee on Reactor Safeguards with regard to the Authority's application for a full power license, the Westinghouse Electric Corporation was requested to perform a loss of coolant accident emergency core cooling system analysis for the most limiting break condition for the subject facility.

This analysis utilized the October, 1975 approved ECCS Evaluation Model for the Indian Point 3 facility and included the correction for the calculation of the volumetric heat generation from the zirconium-water reaction. This calculation was performed for the upcoming Cycle 2 operating conditions.

The results of this analysis are included in the Attachment 1.

Very truly yours,  
*Lewis R. Bennett*  
Lewis R. Bennett  
Acting General Manager

Att.

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