

William J. Cahill, Jr.
Vice President

Regulatory

File Cy.

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, N Y 10003
Telephone (212) 460-3819

April 7, 1977

RE: Indian Point Unit No. 3
Docket No. 50-286

Director of Nuclear Reactor Regulation
ATTN: Mr. Robert W. Reid, Chief
Operating Reactors Branch No. 4
Division of Operating Reactors
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555



Dear Mr. Reid:

Your letter dated March 8, 1977 requested either revised technical specification limits for $F_{\Delta H}^N$, to account for the effects of fuel rod bowing on departure from nucleate boiling, or justification why such revised limits are not required for Indian Point Unit No. 3.

As discussed with members of the Regulatory Staff, additional time is required for the owner of the facility (PASNY) to review our proposed reply. Therefore, our Application For Amendment To Operating License proposing revised technical specification limits for $F_{\Delta H}^N$ will be submitted for your review by April 22, 1977.

Very truly yours,

A handwritten signature in cursive script, appearing to read "William J. Cahill, Jr.".

William J. Cahill, Jr.

Copy to:
Mr. George T. Berry
General Manager and Chief Engineer
Power Authority of the State of New York
10 Columbus Circle
New York, N.Y. 10019

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PDR ADDCK 05000286
PDR

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RE: Indian Point Unit No. 3
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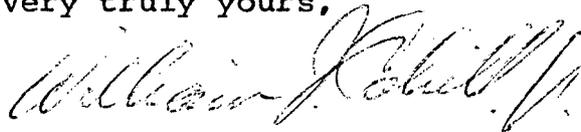
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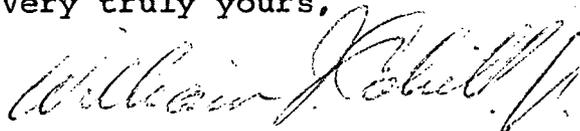
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William J. Cahill, Jr.

APR 1 1977

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DOCKET-2
~~NRC PDR-2~~
 LOCAL PDR-2
 ORB#4 Rdg.
 TJCarter
 RWReid
 RIngram
 PErickson
 GZech
 Attorney, OELD
 OI&E(3)
 DEisenhut
 TBAbernathy
 JRBuchanan
 ACRS(16)
 Gray file

Dockets Nos. 50-247
and 50-286 ✓

Consolidated Edison Company
 of New York, Inc.
 ATTN: Mr. William J. Cahill, Jr.
 Vice President
 4 Irving Place
 New York, New York 10003

Gentlemen:

RE: INDIAN POINT UNITS NOS. 2 AND 3

A pressure transient recently occurred at an operating PWR facility when a reactor coolant pump (RCP) was started while in a water-solid condition. At the time, the temperature of the steam generator in that loop was greater than the reactor coolant system (RCS) temperature and a pressure transient from about 400 psig to about 800 psig resulted.

In our January 1977 letters to PWR licensees we stated that if RCP starts while in a water-solid condition should be necessary, appropriate steps should be taken to determine and minimize the temperature profile within the RCS. Such action was not taken prior to the recently report event.

Although you indicated in your letter of October 25, 1976 that procedural changes have been made to prevent an occurrence such as described above, it is strongly recommended that you immediately conduct a critical review of your procedures to ensure that they are fully responsive to all of the positions stated in our letter of January 10, 1977, and compatible with the pertinent results of your RCS pressure response analysis.

Sincerely,

Original signed by *M. Sartile for*

Robert W. Reid, Chief
 Operating Reactors Branch #4
 Division of Operating Reactors

cc: See next page

OFFICE ➤	ORB#4:DOR	ORB#1:DOR	C-ORB#4:DOR		
SURNAME ➤	PErickson	GZech	RWReid		
DATE ➤	8/ /77	4/ /77	4/ /77		

Consolidated Edison Company
of New York, Inc.

cc: Mrs. Kay Winter, Librarian
Hendrick Hudson Free Library
31 Albany Post Road
Montrose, New York 10548

Leonard M. Trosten, Esquire
LeBoeuf, Lamb, Leiby & MacRae
1757 N Street, N. W.
Washington, D. C. 20036

Anthony Z. Roisman, Esquire
Berlin, Roisman & Kessler
1025 15th Street, N.W., 5th Floor
Washington, D. C. 20005

Paul S. Shemin, Esq.
Assistant Attorney General
State of New York
Department of Law
Two World Trade Center
New York, New York 10047

Sarah Chasis, Esq.
Richard M. Hall, Esquire
15 West 44th Street
New York, New York 10036

Director, Technical Development
Programs
State of New York
Energy Office
Swan Street Building
CORE 1 - Second Floor
Empire State Plaza
Albany, New York 12223