

Carl L. Newman
Vice President

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REGULATORY DOCKET FILE COPY

April 14, 1976

RE: Indian Point Unit No. 3
Docket No. 50-286
R. O. - 76-3-3-(A)

Mr. James P. O'Reilly, Director
Office of Inspection and Enforcement
Region 1
U.S. Nuclear Regulatory Commission
King of Prussia, PA 19406



Dear Mr. O'Reilly:

In accordance with the requirements of the Technical Specifications to Facility Operating License DPR-64, the attached report of Reportable Occurrence R.O. -76-3-3(A), is submitted. This report fulfills the requirement for a written report within 14 days of a Reportable Occurrence and is in accordance with the format set forth in Regulatory Guide 1.16, Rev. 4.

Three copies of this letter and the attachment are enclosed as required.



Very truly yours,

Carl L. Newman

Carl L. Newman
Vice President

Encl.
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Copies to: Director of Nuclear Reactor Regulation
Attn: Mr. Donald F. Knuth, Director (40 copies)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Director of Nuclear Reactor Regulation
Attn: Mr. William F. McDonald, Director (3 copies)
Office of Management Info & Program Control
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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LICENSEE EVENT REPORT

CONTROL BLOCK:

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(PLEASE PRINT ALL REQUIRED INFORMATION)

01	NY	I	P	S	3	00	-	0000	0	-	00	4	1	1	1	1	0	1						
7	8	9	14	15	25	26	30	31	32															
01	CONT			T	L	05	0	0	2	8	6	0	3	3	1	7	6	0	4	1	4	7	6	
7	8	57	58	59	60	61	68	69	74	75	80													

EVENT DESCRIPTION

02	During a reactor coolant system hydrostatic test inspection, a slight																							
7	8	9	14	15	25	26	30	31	32															
03	leak (approx. 1 drop/30 min) was discovered at incore thermocouple TE-7																							
7	8	9	14	15	25	26	30	31	32															
04	The thermocouple was capped off with a Swagelok cap. A bonnet seal weld																							
7	8	9	14	15	25	26	30	31	32															
05	leak of valve 565D, reported originally in the same 24 hr. report on																							
7	8	9	14	15	25	26	30	31	32															
06	April 1, 1976, was incorrectly classified as reportable and requires no																							
7	8	9	14	15	25	26	30	31	32															

07	I	F	E	I	N	S	T	R	U	N	R	3	7	0	N
7	8	9	10	11	12	17	43	44	47	48					

CAUSE DESCRIPTION

08	A Rosemount thermocouple (West. E-Spec. 676511) leaked through the																							
7	8	9	14	15	25	26	30	31	32															
09	sheathing, resulting in the reported reactor coolant leak.																							
7	8	9	14	15	25	26	30	31	32															
10																								
7	8	9	14	15	25	26	30	31	32															

11	B	0	0	0	NA	B	Routine Inspection																		
7	8	9	10	12	13	44	45	46																	

12	Z	Z	NA	NA																					
7	8	9	10	11	44	45																			

PERSONNEL EXPOSURES

13	0	0	0	Z	NA																				
7	8	9	11	12	13																				

PERSONNEL INJURIES

14	0	0	0	NA																					
7	8	9	11	12																					

PROBABLE CONSEQUENCES

15	NA																								
7	8	9	14	15	25	26	30	31	32																

LOSS OR DAMAGE TO FACILITY

16	Z	NA																							
7	8	9	10																						

PUBLICITY

17	NA																								
7	8	9	14	15	25	26	30	31	32																

ADDITIONAL FACTORS

18	(Event Descr. cont.) - report per discussion with I & E Project																								
7	8	9	14	15	25	26	30	31	32																
19	Inspector. (R.O.-76-3-3 (A)).																								
7	8	9	14	15	25	26	30	31	32																

NAME: Michael F. Shatkouski PHONE: 914-739-8823